INTERIM RESPONSE ACTION (IRA) ADMINISTRATIVE RECORD FILE IR-0100

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IRA-100-101

DOE/OR/21548-154 CONTRACT NO. DE-AC05-860R21548

SCREENING LEVEL CHARACTERIZATION OF ELECTRICAL SUBSTATION 411

For the Weldon Spring Site Remedial Action Project Weldon Spring, Missouri

Prepared by MK-Ferguson Company and Jacobe Engineering Group

NOVEMBER 1990

REV. 0

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Weldon Spring Site Remedial Action Project

Screening Level Characterization of Electrical Substation 411

November 1990

Revision 0

Prepared by

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Prepared for

U.S. DEPARTMENT OF ENERGY Oak Ridge Operations Office Under Contract DE-AC05-860R21548

ABSTRACT

Substation 411 transformer dielectric fluids were sampled for uranium and PCBs. The exterior surfaces of the metal transformers were measured for radioactivity. Results of the sampling revealed that some transformer fluid contained PCBs, but no uranium in excess of naturally occurring amounts was detected. In addition, the exterior surface of the transformers were uncontaminated. Subject to measurement of the base of the transformers (which were inaccessible during the surface scans for radioactivity), the transformers and dielectric fluids may be removed from the site for disposal.

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1. INTRODUCTION

This report presents characterization data for the Number 411 Electrical Substation located on Department of Conservation properties south of the Weldon Spring Chemical Plant (WSCP). Sample collection methods, sampling equipment and analytical results are also presented in this report. The objective of this characterization was to supply data needed to prepare a subcontract package for the removal of all transformers and dielectric fluids contained in Substation 411.

2 SAMPLING

Actual sampling of Substation 411 was undertaken April 23 and 24, 1990. This included soil sampling throughout the area and retrieving dielectric fluids (oil) samples from each transformer. A sampling plan was developed previous to sampling. This plan contains detailed information on sampling locations, sampling equipment, sampling techniques and personal protective equipment used during the sampling of Substation 411. This sampling plan is provided in Appendix A.

As mentioned in the sampling plan, two pole mounted transformers located within Substation 411 could not be sampled due to safety concerns. On June 14, 1990, a subcontractor was employed to disconnect and remove the transformers from their respective poles and place them on the substation concrete pad. The transformers were sampled on June 20, 1990 using sampling techniques and equipment contained in the sampling plan.

3 RESULTS

Each group of analytical parameters for oil and soil samples is discussed in detail in the following sections. All interpretations made in this report are based on analytical results, field observations and historical data.

3.1 Substation 411 Transformer Dielectric Fluids

The summary of oil analysis for PCB's and uranium may be found in Table 3-1. Also contained in the table is physical information regarding the transformers.

3.1.1 Radiological

Oil from each transformer was sampled either individually or through composites for natural uranium. The highest level detected was 62 pCi/l. Because there is no regulatory limit for uranium in transformer fluids, and also no background level information available for uranium in transformer fluids, a second round of sampling took place on August 20 and 21, 1990 to evaluate sampling and/or analysis variability. A blank containing motor oil was sent to the lab to establish a sample analysis background concentration. Triplicate samples were obtained for each transformer and the motor oil. Results of the lab analysis for this event are presented in Table 3-2. On September 6, 1990 a transmittal was received by the lab stating that 62 pCi/l had been detected, but that the value should be questioned because of contamination of the lab equipment. After decontamination of the lab equipment the sample was rerun, obtaining a value of <0.68 pCi/l, which is below the detection limit. With this result noted, the next highest level detected for the first sampling event was 2 pCi/l. Concentrations of

TABLE 3-1 Electrical Substration 411 Information

				<u> </u>
Unit	Serial Number	Oil Volume	PCB Level of Fluid	Uranium Level of Fluid
Transformer	2715-1	1,285 gallons	<10 PPM	2 pCi/l
Transformer	2715-2	1,285 gallons	<10 PPM	2 pCi/l
Transformer	C-500897	362 gallons	157 PPM Arochlor 1260	2 pCi/1
3 Phase Oil Circuit Breaker	12696	270 gallons 90 gal/unit	<10 PPM	0.68 pC1/1
Pole Transformer	Unknown	Apprx. 40 gallons	<10 PPM	0.68 pCi/l
Single Phase Pole Transformer	2814-16	12 gallons	1100 PPM Arochlor 1260	*<0.68 pCi/l
Three Phase Metering Outfit	58E6844	24 gallons	6.1 PPM Arochlor 1260	*<0.68 pC1/1

Uranium sample composited from single phase transformer and metering outfit. The sample was originally reported as 62 pCi/l. Information received from the lab stated this level was due to equipment contamination. After decontamination, the sample was rerun, obtaining this result.

TABLE 3-2 Electrical Substation 411 Information for Uranium Samples - Collected August 20 & 21, 1990

Unit	Serial Number	Uranium Levels of 3 Confirmation Triplicate Samples (pCi/l)
Transformer	2715-1	1.1 - 1.5 + 1.3
Transformer	2715-2	12.1 - 0.72 - 1.1
Transformer	C-500897	0.7 - 2.0 - 0.65
3 Phase Oil Circuit Breaker	12696	0.7 - 1.1 - 5.0
Pole Transformer	Unknown	0.5 - 9.5 - 0.5
Single Phase Pole Transformer	2814-16	0.5 - 0.5 - 0.8
Three Phase Metering Outfit	58E6844	4.1 - 2.4 - 3.4
Motor Oil	EXXON	<0.4 - 1.8 - 10.0

2 pCi/l or less are within expected limits considering the natural distribution of uranium typically found in the environment. The results of the second round of sampling at locations identified in Table 3-2 indicate that the distribution of detected levels of transformer oils is not above the upper limits of the distribution of values for the background sample of motor oil.

3.1.2 PCB's

Oil from each transformer was sampled for PCBs. Lab analysis indicates that the General Electric transformer containing 362 gallons of oil has a level of 157 ppm of the PCB Arochlor 1260. The three-phase metering outfit containing 24 gallons of oil has a level of 6.1 ppm of the PCB Arochlor 1260. The single phase transformer containing 12 gallons of oil has a level of 1,100 ppm of the PCB Arochlor 1260.

Due to the concerns associated with a possible PCB spill, transformers containing detectable levels of PCBs were drained of oil on August 24, 1990. The oil was stored in Building 434. The transformer containing 1100 ppm PCB oil was also flushed with diesel fuel as specified for transformers containing greater than 500 ppm PCB's in 40 CFR 761.60(b)(l)(i)B.

3.2 SUBSTATION 411 SOILS

This section presents analytical lab results for Hazardous Substance List (HSL) metals, lithium, molybdenum, nitroaromatics and PCB's from soil samples taken in the area immediately surrounding Substation 411. These results are presented in Appendix B of this report.

3.2.1 Metals

Two composite soil samples were collected in the area immediately surrounding Substation 411 and analyzed for HSL metals, lithium and molybdenum. As the lab results in Appendix B indicate, these soil samples appear to be at levels considered background for this area. Documentation of background results can be found in Chemical Soil Investigation Report for the WSCP/RPs, Phase II (DOE/OR/21548-061, August 1989).

3.2.2 Nitroaromatics

Two composite soil samples were also collected in the substation area and analyzed for nitroaromatics. This was deemed important because the electrical Substation 411 was constructed directly over an Ordnance Works TNT line which had been abandoned. The composite samples taken for nitroaromatics were chosen to represent the entire area around Substation 411. Additional samples were taken near the abandoned TNT line. As the lab results in Appendix B demonstrate, the two samples had nitrobenzene concentrations of 1.3 and 3.7 ppm. Such low levels do not warrant any special precautions during removal of the transformer oils and carcasses contained in Substation 411.

3.2.3 PCB's

Individual and composite soil samples for PCB analysis were collected at eleven locations in the area surrounding and within Substation 411. One composite sample for PCB's was also taken near a pole transformer lying on the ground 100 yards due south of Substation 411. As the lab results in Appendix B indicate, only two samples had PCB concentrations above the detection

limit. These levels correspond to individual samples taken down gradient from the transformer containing 157 ppm PCB oil. Although these levels of 2.2 and 3.6 ppm are above detection limits, they are well below the 10 ppm clean up criteria for an uncontrolled access area (40 CFR 761.125(c)(4)).

3.3 EXTERIOR RADIOLOGICAL CONTAMINATION

The tops and sides of the transformer carcasses were scanned with a gamma-beta detector and swiped for radiological contamination. The units were determined to be slightly above background levels, but within the uranium surface contamination guidelines for unrestricted release. However, the bottoms of all electrical units will require survey prior to release. This will be performed during removal of the units, which will require heavy equipment. The on-site analysis of paint scrapings removed from the surfaces of the No. 411 Electrical Substation units indicates uranium to be the primary contaminant, thus uranium will be the focal point of our sampling of the transformer bases.

4 CONCLUSIONS/RECOMMENDATION

There is no data to suggest that the transformer oils from substation 411 contain uranium above naturally occurring levels and, therefore, should not be considered a mixed waste. This is supported by lab analysis presented in Section 3.1.1. Also with the bottoms of the electrical units withstanding, the units are within the uranium surface contamination guidelines for unrestricted release as mentioned in Section 3.3.

It is recommended that the oils and transformer carcasses be disposed of as non-radiological contaminated material.

APPENDIX A SUBSTATION 411 SAMPLING PLAN

No. 411 Electrical Substation Sampling Plan
April 1990

Prepared by

MK-FERGUSON COMPANY and JACOBS ENGINEERING GROUP 7295 Highway 94 South St. Charles, Missouri 63303

Prepared for

U.S. DEPARTMENT OF ENERGY Oak Ridge Operations Office Under Contract DE-AC05-86OR21548

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1 INTRODUCTION

This plan describes the sampling effort to be conducted in April 1990 to determine PCB and radiological content of individual transformers and area soil at the No. 411 Electrical Substation. Soil samples for nitroaromatics and HSL metals plus lithium and molybdium will also be retrieved for this exercise.

2 OBJECTIVES

This sampling effort will be performed to determine whether dielectric fluids in the No. 411 transformers are PCB and/or radiologically contaminated. The effort will also determine if the soil in and immediately surrounding No. 411 is contaminated by PCB's, nitroaromatics, HSL metals plus lithium and molybdeium. The data collected will then be used by the PMC prepare a subcontract package for transformer removal.

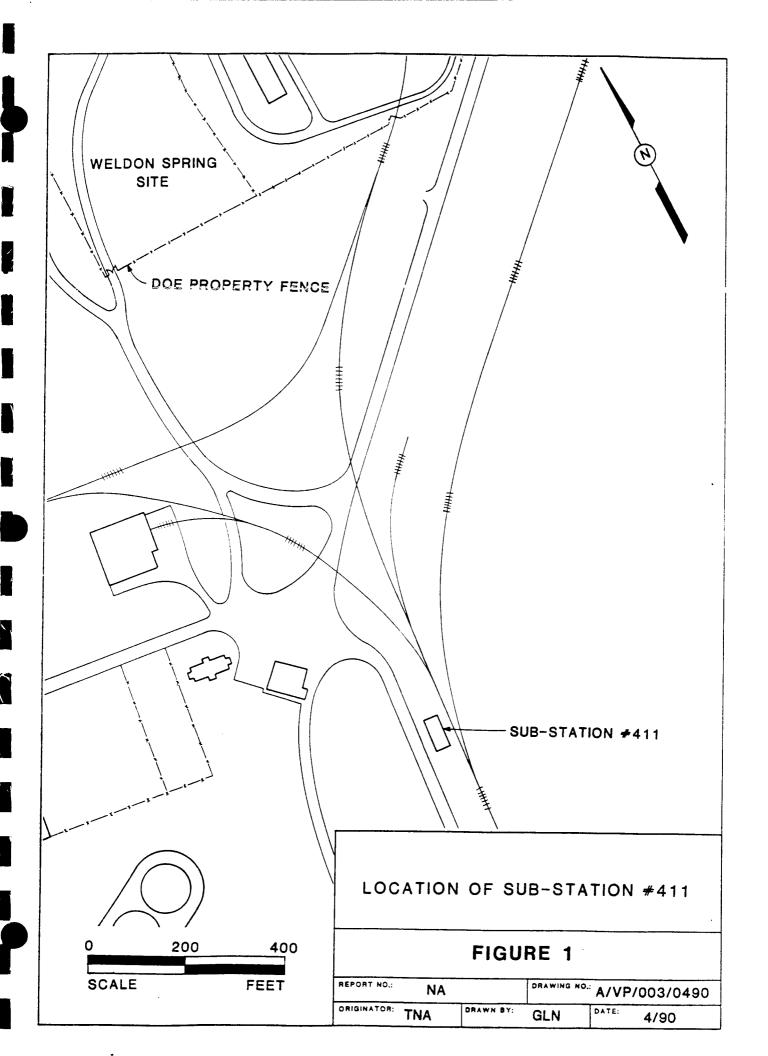
3 SAMPLING SCHEDULE

Samples will be collected during April 1990. The samples will be collected by Environmental Safety and Health (ES&H) personnel. Prior to the actual sampling, the following requirements will be fulfilled:

- The required sampling and safety equipment will be available.
- The respiratory protection program will be in operation.
- The Spill Prevention, Control and Containment Plan (SPCC Plan) will be in effect.
- Laboratories will have been selected to perform sample analysis.
- The Weldon Spring Site Remedial Action Project (WSSRAP)
 Operations Department will have certified that all transformers are electrically discharged and grounded.
- Access to all of the transformers will have been established.

3.1 SAMPLING LOCATIONS

The location of the No. 411 Electrical Substation is shown in Figure 1. Locations for proposed oil and soil sampling are presented in Figure 2. Tables 1 and 2 detail information for each sampling location and Table 3 lists physical information for each transformer. Individual oil samples for PCB's will be



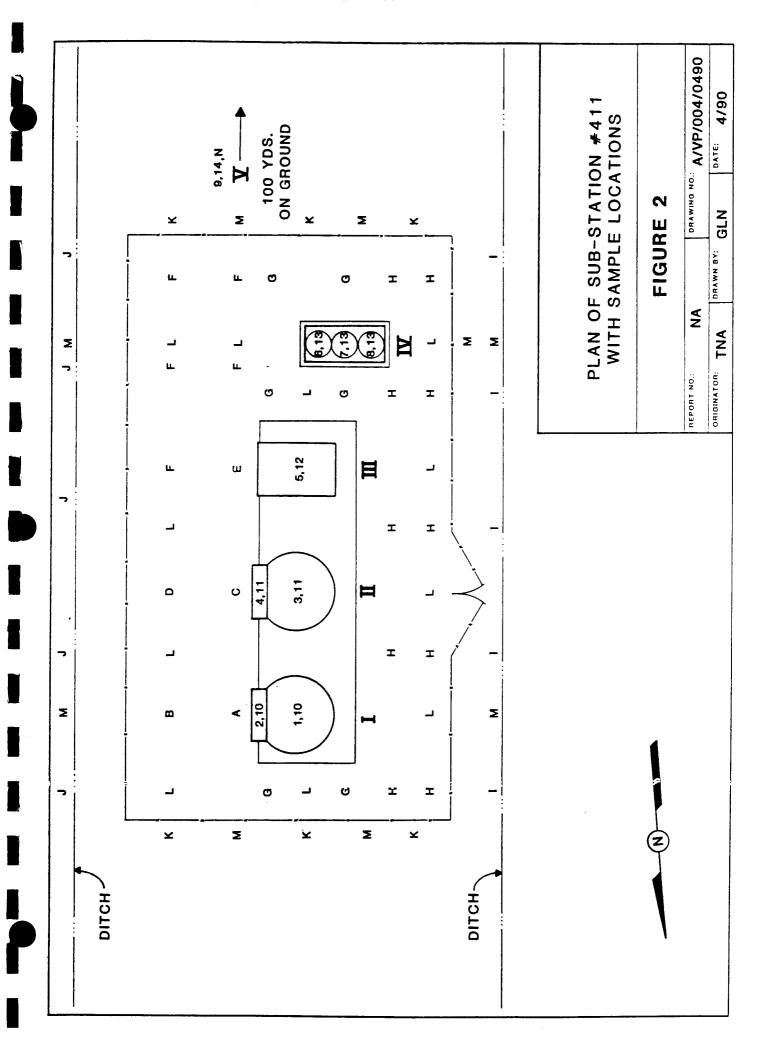


TABLE 1 Oil Sampling Locations

Number From Figure 2	Sample Type	Parameter	Sample Mumber	Des cription
1	Individual	PCB's	TO-0001-Date	Transformer I
2	Individual	PCB's	TO-0002-Date	Reservoir Tank on Transformer I
3	Individual	PCB's	TO-0003-Date	Transformer I
4	Individual	PCB's	TO-0004-Date	Reservoir Tank on Transformer II
5	Individual	PCB's	TO-0005-Date	Transformer III
6	Individual	PCB's	TO-0006-Date	Rastside vessel of 3-phase circuit breaker
7	Individual	PCB's	TO-0007-Date	Middle vessel of 3-phase circuit breaker
8	Individual	PCB's	TO-0008-Date	Westside vessel of 3-phase circuit breaker
9	Individual	PCB's	TO-0009-Date	Pole transformer lying 100 yards south of No. 411 on ground
10	Composite	Nat. Uranium, Total	TO-0010-Date	Transformer I and Reservoir Tank
11	Composite	Nat. Uranium, Total	TO-0011-Date	Transformer II and Reservoir Tank
12	Individual	Nat. Uranium, Total	TO-0012-Date	Transformer III
13	Composite	Nat. Uranium, Total	TO-0013-Date	All 3 vessels from 3-phase circuit breaker
14	Individual	Nat. Uranium, Total	TO-0014-Date	Pole transformer lying 100 yards south of No. 411 on ground

TABLE 2 Soil Sampling Locations

Letter				
From	_	Sample		Sample
Figure	2	Туре	Parameter	Number
A		Individual	PCB's	TS-000A-Date
В		Individual	PCB's	TS-000B-Date
C		Individual	PCB's	TS-000B-Date
D		Individual	PCB's	TS-000D-Date
E		Individual	PCB's	TS-000E-Date
F		Composite	PCB's	TS-000F-Date
G		Composite	PCB's	TS-000G-Date
H		Composite	PCB's	TS-000H-Date
I		Composite	PCB's	TS-000I-Date
J		Composite	PCB's	TS-000J-Date
K		Composite	PCB's	TS-000K-Date
L		Composite	Nitroaromatics HSL Metals and Li, Mo	TS-000L-Date
M		Composite	Nitroaromatics HSL Metals and Li, Mo	TS-000M-Date
N		Composite	PCB's	TS-000N-Date

TABLE 3 Physical Information on Transformers

Number From Figure 2	Serial Number	Manufacturer	Oil Capacity	Description
ī	2715-1	Pennsylvania	1,285 gallons	Transformer
11	2715-2	Pennsylvania	1,285 gallons	Transformer
III	C-500897	General Electric	362 gallons	Transformer
IA	12696	Pacific Electric	270 gallons 90 gal/unit	3 phase oil circuit breaker
v	Unknown	Unknown	approx. 40 gal	Pole transformer lying on ground

taken, however most radiological oil samples will be composited. Two composite soil samples will be retrieved to determine the presence or absence of nitroaromatics and HSL metals plus lithium and molybdeum. Individual PCB soil samples will be retrieved where visible oil leakage has occurred. PCB composite samples will be taken to screen the majority of the area. Two small pole transformers are contained above No. 411 but will not be sampled in this effort due to safety concerns. These units are to be treated as PCB-contaminated for purposes of subcontract development.

3.2 Sample Collection

Sampling personnel will use the personal protective equipment listed in Table 4 during the actual sampling operations. In addition the equipment listed in Table 4 will be available for use during sample collection. Spill response equipment will also be on hand.

Personal chemical contamination control must be practiced during and following the collection and shipment of these samples. Samples will be collected by a team of at least two persons as follows:

- Prepare a field data sheet for the sample to be collected. An example field data sheet is presented in Figure 3.
- Select the corresponding pre-labeled sample bottles for the sample to be collected at that location.
- Place plastic sheeting and containers under sample ports where applicable for spill control.

TABLE 4 Equipment Needed For Transformer Sampling

Item

Usage

Tool set Buckets Plastic sheeting Oil absorbent pillows and booms Peristaltic pump Peristaltic pump Generator Funnels Spoons Sample bottles Ziploc bags D.I. Water Hexane Camera Sample labels Field data sheets Log book Half-face respirator with organic vapor cartridges Disposable boot covers Hard hats Saranex or polycoated tyvek Face shields Nitrile gloves/surgical gloves Radiation monitoring equipment Two-way radio Flags PCB Field Test Kits

Transformer opening Spill control Spill control Spill control Sample drawing Sample drawing Sample drawing Sample drawing Sampling Sample shipping Sample shipping Decontamination Decontamination Documentation Documentation Documentation Documentation

Personal protection
Marking Soil Sample Locations
Sampling

FIGURE 3 Transformer Fluid and Soil Sampling - Field Data Sheet

Sample Number:	D	Date:
Sample Time:		
Sampling Location Descriptio	n:	
Sample Description:		
Radiation Level:		
Background		
Elevated A	mount:	cpm
Instrument:		
Samplers Signature		
Comments:		
4,44		

- Slowly open the valve with a sample bottle under the spigot where applicable. Fill the bottle only 3/4 full.
- If sample valves and spigots are inaccessible, entry will be gained at the top of each transformer. A peristaltic pump will then be used to collect samples.
- Seal the caps on the sample bottles with tape.
- Place the oil sample bottles in separate Ziploc bags.
- Clean up the surrounding area.
- Decontaminate equipment following the procedures in WSSRAP ES&H SOP 4.1.3.
- Containerize and store any contaminated articles which cannot be decontaminated.

RADIATION MONITORING

A member of the Health Physics Group will measure the radiation levels of the individual samples prior to shipment of the samples off site. Surface radiation levels for each transformer will be measured by Health Physics technicians.

QUALITY CONTROL

One duplicate sample will be taken for oil and soil for PCB's.

REPORTING OF ANALYTICAL RESULTS

The lab data, field data, and drawings showing the sampling locations will be incorporated in a final report by ES&H for submittal to the Engineering Department. This report will be completed approximately June 1990.

APPENDIX B
SUBSTATION 411 SOILS LAB REPORTS

Contract No. 3589/15-1000-1-11000 P.O. No. 3589-1002-1979 Request No. 84

TABLE 2

DATA SUMMARY

Client ID: TS-000A-042490

JTC ID: 90-04-062-01A

Location ID: WSSRAP

Matrix: Soil

Date Sampled: 4/24/90 Date Extracted: 5/04/90

PCB	Concentration	Units of Measure	Quantitation Limit	Date <u>Analyzed</u>
Arochlor-1016	BQL	$\mu_{ t g}/{ t Kg}$	1900	5/24/90
Arochlor-1221	BQL	$\mu_{ t g}/{ t K_{ t g}}$	1900	5/24/90
Arochlor-1232	BQL	μg/Kg	1900	5/24/90
Arochlor-1242	BQL	$\mu_{ t g}/{ t Kg}$	1900	5/24/90
Arochlor-1248	BQL	μg/Kg	1900	5/24/90
Arochlor-1254	BQL	μ g/Kg	1900	5/24/90
Arochlor-1260	BQL	μg/Kg	3800	5/24/90

Contract No. 3589/15-1000-1-11000 P.O. No. 3589-1002-1979 Request No. 84

TABLE 3

DATE SUMMARY

Client ID: TS-000B-042490

JTC ID: 90-04-062-02A

Location ID: WSSRAP

Matrix: Soil

Date Sampled: 4/24/90
Date Extracted: 5/04/90

PCB	<u>Concentration</u>	Units of <u>Measure</u>	Quantitation Limit	Date <u>Analyzed</u>
Arochlor-1016	BQL	μ g/Kg	95	5/24/90
Arochlor-1221	BQL	$\mu_{ t g}/{ t Kg}$	95	5/24/90
Arochlor-1232	BQL	$\mu_{ t g}/{ t Kg}$	95	5/24/90
Arochlor-1242	BQL	$\mu_{ t g}/{ t Kg}$	95	5/24/90
Arochlor-1248	BQL	$\mu_{ t g}/{ t Kg}$	95	5/24/90
Arochlor-1254	BQL	$\mu_{ t g}/{ t Kg}$	190	5/24/90
Arochlor-1260	BQL	$\mu_{ t g}/{ t Kg}$	190	5/24/90

Contract No. 3589/15-1000-1-11000 P.O. No. 3589-1002-1979 Request No. 84

TABLE 4

DATA SUMMARY

Client ID: TS-000C-042490

JTC ID: 90-04-062-03A

Location ID: WSSRAP Matrix: Soil

Date Sampled: 4/24/90

Date Extracted: 5/04/90

PCB	Concentration	Units of <u>Measure</u>	Quantitation <u>Limit</u>	Date <u>Analyzed</u>
Arochlor-1016	BQL	μg/Kg	10,000	5/24/90
Arochlor-1221	BQL	μ g/Kg	10,000	5/24/90
Arochlor-1232	BQL	$\mu_{ t g}/{ t Kg}$	10,000	5/24/90
Arochlor-1242	BQL	$\mu_{ t g}/{ t Kg}$	10,000	5/24/90
Arochlor-1248	BQL	$\mu_{ t g}/{ t Kg}$	10,000	5/24/90
Arochlor-1254	BQL	$\mu_{ t g}/{ t K_{ t g}}$	20,000	5/24/90
Arochlor-1260	BQL	$\mu_{ t g}/{ t Kg}$	20,000	5/24/90

Contract No. 3589/15-1000-1-11000 P.O. No. 3589-1002-1979 Request No. 84

TABLE 5

DATA SUMMARY

Client ID: TS-000D-042490

JTC ID: 90-04-062-04A

Location ID: WSSRAP

Matrix: Soil

Date Sampled: 4/24/90

Date Extracted: 5/04/90

PCB	Concentration	Units of <u>Measure</u>	Quantitation <u>Limit</u>	Date <u>Analyzed</u>
Arochlor-1016	BQL	μ g/Kg	980	5/24/90
Arochlor-1221	BQL	μg/Kg	980	5/24/90
Arochlor-1232	BQL	μ g/Kg	980	5/24/90
Arochlor-1242	BQL	$\mu_{ t g}/{ t Kg}$	980	5/24/90
Arochlor-1248	BQL	$\mu_{ t g}/{ t Kg}$	980	5/24/90
Arochlor-1254	BQL	$\mu_{ t g}/{ t Kg}$	2000	5/24/90
Arochlor-1260	2200	$\mu_{ t g}/{ t K_{ t g}}$	2000	5/24/90

Contract No. 3589/15-1000-1-11000 P.O. No. 3589-1002-1979 Request No. 84

TABLE 6

DATA SUMMARY

Client ID: TS-000E-042490

JTC ID: 90-04-062-05A

Location ID: WSSRAP

Matrix: Soil

Date Sampled:

4/24/90 Date Extracted: 5/04/90

PCB	Concentration	Units of <u>Measure</u>	Quantitation Limit	Date <u>Analyzed</u>
Arochlor-1016	BQL	μg/Kg	1500	5/24/90
Arochlor-1221	BQL	μg/Kg	1500	5/24/90
Arochlor-1232	BQL	μg/Kg	1500	5/24/90
Arochlor-1242	BQL	μg/Kg	1500	5/24/90
Arochlor-1248	BQL	μg/Kg	1500	5/24/90
Arochlor-1254	BQL	μg/Kg	3000	5/24/90
Arochlor-1260	3600	μg/Kg	3600	5/24/90

Contract No. 3589/15-1000-1-11000 P.O. No. 3589-1002-1979 Request No. 84

TABLE 7

DATA SUMMARY

Client ID: TS-000F-042490

JTC ID: 90-04-062-06A

Location ID: WSSRAP

Matrix: Soil

Date Sampled:

4/24/90 Date Extracted: 5/04/90

PCB	Concentration	Units of <u>Measure</u>	Quantitation Limit	Date <u>Analyzed</u>
Arochlor-1016	BQL	$\mu_{ t g}/{ t Kg}$	120	5/24/90
Arochlor-1221	BQL	$\mu_{ t g}/{ t K}_{ t g}$	120	5/24/90
Arochlor-1232	BQL	$\mu_{ t g}/{ t Kg}$	120	5/24/90
Arochlor-1242	BQL	$\mu_{ t g}/{ t K}_{ t g}$	120	5/24/90
Arochlor-1248	BQL	μ g/Kg	120	5/24/90
Arochlor-1254	BQL	μ g/Kg	230	5/24/90
Arochlor-1260	BQL	μ g/Kg	230	5/24/90

Contract No. 3589/15-1000-1-11000 P.O. No. 3589-1002-1979 Request No. 84

TABLE 8

DATA SUMMARY

Client ID: TS-000G-042490

JTC ID: 90-04-062-07A

Location ID: WSSRAP

Matrix: Soil

Date Sampled: 4/24/90 Date Extracted: 5/04/90

PCB	Concentration	Units of <u>Measure</u>	Quantitation <u>Limit</u>	Date <u>Analyzed</u>
Arochlor-1016	BQL	$\mu_{ t g}/{ t Kg}$	1200	5/24/90
Arochlor-1221	BQL	$\mu_{ t g}/{ t K}_{ t g}$	1200	5/24/90
Arochlor-1232	BQL	μg/Kg	1200	5/24/90
Arochlor-1242	BQL	$\mu_{ t g}/{ t K_{ t g}}$	1200	5/24/90
Arochlor-1248	BQL	μg/Kg	1200	5/24/90
Arochlor-1254	BQL	μg/Kg	2400	5/24/90
Arochlor-1260	BQL	μg/Kg	2400	5/24/90

Contract No. 3589/15-1000-1-11000 P.O. No. 3589-1002-1979 Request No. 84

TABLE 9

DATA SUMMARY

Matrix: Soil

Client ID: TS-000H-042490

JTC ID: 90-04-062-08A

Location ID: WSSRAP

Date Sampled: 4/24/90

Date Extracted: 5/04/90

PCB	Concentration	Units of <u>Measure</u>	Quantitation Limit	Date <u>Analyzed</u>
Arochlor-1016	BQL	μg/g	1000	5/24/90
Arochlor-1221	BQL	μg/g	1000	5/24/90
Arochlor-1232	BQL	μg/g	1000	5/24/90
Arochlor-1242	BQL	μg/g	1000	5/24/90
Arochlor-1248	BQL	μg/g	1000	5/24/90
Arochlor-1254	BQL	$\mu_{ t g}/{ t g}$	2100	5/24/90
Arochlor-1260	BQL	µg/g	2100	5/24/90

Contract No. 3589/15-1000-1-11000 P.O. No. 3589-1002-1979 Request No. 84

TABLE 10

DATA SUMMARY

Client ID: TS-000I-042490

JTC ID: 90-04-062-09A

Location ID: WSSRAP

Matrix: Soil

Date Sampled: 4/24/90 Date Extracted: 5/04/90

Units of Quantitation Date PCB Concentration <u>Measure</u> Limit Analyzed Arochlor-1016 BQL $\mu_{\rm g}/{\rm Kg}$ 1200 5/24/90 Arochlor-1221 BQL $\mu_{\rm g}/{\rm Kg}$ 1200 5/24/90 Arochlor-1232 BQL $\mu_{\rm g}/{\rm Kg}$ 1200 5/24/90 Arochlor-1242 BQL $\mu_{\rm g}/{\rm Kg}$ 1200 5/24/90 Arochlor-1248 BQL $\mu_{\rm g}/{\rm Kg}$ 1200 5/24/90 Arochlor-1254 BQL $\mu_{\rm g}/{\rm Kg}$ 2500 5/24/90 Arochlor-1260 BQL $\mu g/Kg$ 2500 5/24/90

Contract No. 3589/15-1000-1-11000 P.O. No. 3589-1002-1979 Request No. 84

TABLE 11

DATA SUMMARY

Client ID: TS-000J-042490

JTC ID: 90-04-062-10A

Location ID: WSSRAP

Matrix: Soil

Date Sampled:

4/24/90

Date Extracted: 5/04/90

PCB	Concentration	Units of <u>Measure</u>	Quantitation Limit	Date <u>Analyzed</u>
Arochlor-1016	BQL	μg/Kg	1200	5/24/90
Arochlor-1221	BQL	$\mu_{ t g}/{ t Kg}$	1200	5/24/90
Arochlor-1232	BQL	$\mu_{ t g}/{ t Kg}$	1200	5/24/90
Arochlor-1242	BQL	$\mu_{ t g}/{ t Kg}$	1200	5/24/90
Arochlor-1248	BQL	$\mu_{ t g}/{ t K_{ t g}}$	1200	5/24/90
Arochlor-1254	BQL	$\mu_{ t g}/{ t K_{ t g}}$	2500	5/24/90
Arochlor-1260	BQL	μg/Kg	2500	5/24/90

Contract No. 3589/15-1000-1-11000 P.O. No. 3589-1002-1979 Request No. 84

TABLE 12

DATA SUMMARY

Client ID: TS-000K-042490 JTC ID: 90-04-062-11A

Location ID: WSSRAP Matrix: Soil Date Sampled: 4/24/90
Date Extracted: 5/04/90

Units of Quantitation Date PCB Concentration Measure Limit <u>Analyzed</u> Arochlor-1016 BQL $\mu g/Kg$ 1000 5/24/90 Arochlor-1221 BQL $\mu_{\rm g}/{\rm Kg}$ 1000 5/24/90 Arochlor-1232 BQL $\mu_{\rm g}/{\rm Kg}$ 1000 5/24/90 Arochlor-1242 BQL $\mu g/Kg$ 1000 5/24/90 Arochlor-1248 BQL $\mu_{\rm g}/{\rm Kg}$ 1000 5/24/90 Arochlor-1254 BQL $\mu_{\rm g}/{\rm Kg}$ 2100 5/24/90 Arochlor-1260 BQL $\mu_{\rm g}/{\rm Kg}$ 2100 5/24/90

Contract No. 3589/15-1000-1-11000 P.O. No. 3589-1002-1979 Request No. 84

TABLE 13

DATA SUMMARY

Client ID: TS-000N-042490 JTC ID: 90-04-062-12A

Location ID: WSSRAP Matrix: Soil Date Sampled: 4/24/90

Date Extracted: 5/04/90

PCB	Concentration	Units of Measure	Quantitation <u>Limit</u>	Date <u>Analyzed</u>
Arochlor-1016	BQL	$\mu_{ t g}/{ t Kg}$	1400	5/24/90
Arochlor-1221	BQL	$\mu_{ t g}/{ t Kg}$	1400	5/24/90
Arochlor-1232	BQL	$\mu_{ t g}/{ t Kg}$	1400	5/24/90
Arochlor-1242	BQL	$\mu_{ t g}/{ t Kg}$	1400	5/24/90
Arochlor-1248	BQL	$\mu_{ t g}/{ t Kg}$	1400	5/24/90
Arochlor-1254	BQL	$\mu_{ t g}/{ t Kg}$	2700	5/24/90
Arochlor-1260	BQL	μg/Kg	2700	5/24/90

Contract No. 3589/15-1000-1-11000 P.O. No. 3589-1002-1979 Request No. 84

TABLE 14

DATA SUMMARY

Client ID: TS-000B-042490-DU

JTC ID: 90-04-062-13A

Location ID: WSSRAP Matrix: Soil

Date Sampled: 4/24/90

Date Extracted: 5/04/90

PCB	Concentration	Units of <u>Measure</u>	Quantitation <u>Limit</u>	Date <u>Analyzed</u>
Arochlor-1016	BQL	$\mu_{ t g}/{ t Kg}$	1000	5/24/90
Arochlor-1221	BQL	$\mu_{ t g}/{ t Kg}$	1000	5/24/90
Arochlor-1232	BQL	μ g/Kg	1000	5/24/90
Arochlor-1242	BQL	$\mu_{ t g}/{ t K_{ t g}}$	1000	5/24/90
Arochlor-1248	BQL	$\mu_{ t g}/{ t Kg}$	1000	5/24/90
Arochlor-1254	BQL	$\mu_{ t g}/{ t Kg}$	2000	5/24/90
Arochlor-1260	BQL	μ g/Kg	2000	5/24/90

Contract No. 3589/15-1000-1-11000 P.O. No. 3589-1002-1979 Request No. 84

TABLE 15

RESULTS OF NITROAROMATIC ANALYSES

Client ID: TS-000L-042490 JTC ID: 90-04-062-14A

Location ID: WSSRAP Matrix: Soil Date Sampled: 4/24/90
Date Extracted: 5/11/90

Date Analyzed: 5/30/90

Nitroaromatic Compound	Concentration	Units of <u>Measure</u>	Quantitation Limit
2,4-DNT	BQL	μg/g	0.344
2,6-DNT	BQL	μg/g	0.424
1,3-DNB	BQL	μg/g	0.401
1,3,5-TNB	BQL	μg/g	0.395
2,4,6-TNT	BQL	μg/g	0.369
Nitrobenzene	3.734	μg/g	0.443

Contract No. 3589/15-1000-1-11000 P.O. No. 3589-1002-1979 Request No. 84

TABLE 16

RESULTS OF NITROAROMATIC ANALYSES

Client ID: TS-000M-042490 JTC ID: 90-04-062-15A

Location ID: WSSRAP Matrix: Soil Date Sampled: 4/24/90

Date Extracted: 5/11/90
Date Analyzed: 5/30/90

Nitroaromatic Compound	<u>Concentration</u>	Units of Measure	Quantitation Limit
2,4-DNT	BQL	μg/g	0.353
2,6-DNT	BQL	μg/g	0.436
1,3-DNB	BQL	μg/g	0.412
1,3,5-TNB	BQL	μg/g	0.406
2,4,6-TNT	BQL	μg/g	0.380
Nitrobenzene	1.32	μg/g	0.456

Contract No. 3589/15-1000-1-11000 P.O. No. 3589-1002-1979 Request No. 84

TABLE 17 (REVISED)

RESULTS OF METALS ANALYSES

Client ID: TS-000L-042490 JTC ID: 90-04-062-14A

Location ID: WSSRAP Matrix: Soil Date Sampled: 4/24/90 Date Digested: 5/08/90

Hg Digested: 5/01/90

Parameter	Concentration	Units of Measure	Detection Limit	Methods Followed	Date <u>Analyzed</u>
Aluminum	3210	mg/Kg	39.2	200.7	5/23/90
Antimony	BQL	mg/Kg	15.2	200.7	5/23/90
Arsenic	BQL	mg/Kg	2.0	206.2	5/18/90
Barium	34.7	mg/Kg	3.0	200.7	5/23/90
Beryllium	BQL	mg/Kg	0.2	200.7	5/23/90.
Cadmium	1.86	mg/Kg	0.8	200.7	5/23/90
Calcium	102426	mg/Kg	106	200.7	5/23/90
Chromium	8.02	mg/Kg	1.6	200.7	5/23/90
Cobalt	2.44	mg/Kg	1.8	200.7	5/23/90
Copper	110.2	mg/Kg	3.4	200.7	5/23/90
Iron	5302	mg/Kg	16.8	200.7	5/23/90
Lead	36.8	mg/Kg	0.6	239.2	5/10/90
Magnesium	24500	mg/Kg	87	200.7	5/23/90
Manganese	202	mg/Kg	0.8	200.7	5/23/90
Mercury	BQL	mg/Kg	0.1	245.5	5/02/90
Nickel	6.14	mg/Kg	4.6	200.7	5/23/90
Potassium	690	. mg/Kg	158	258.1	5/20/90
Selenium	BQL	mg/Kg	1.0	270.2	5/11/90
Silver	BQL	mg/Kg	3.8	200.7	5/23/90
Sodium	326	mg/Kg	110	273.1	5/20/90
Thallium	BQL	mg/Kg	2.0	279.2	5/17/90
Vanadium	8.32	mg/Kg	2.2	200.7	5/23/90
Zinc	996	mg/Kg	. 2.8	200.7	5/23/90
Lithium	5.28	mg/Kg	4.0	200.7	5/24/90
Molybdenum	BQL	mg/Kg	4.0	200.7	5/24/90

Contract No. 3589/15-1000-1-11000 P.O. No. 3589-1002-1979 Request No. 84 (REVISED)

TABLE 18

RESULTS OF METALS ANALYSES

Client ID: TS-000M-042490 JTC ID: 90-04-062-15A

Location ID: WSSRAP Matrix: Soil Date Sampled: 4/24/90 Date Digested: 5/21/90

<u>Parameter</u>	Concentration	Units of Measure	Detection Limit	Methods Followed	Date <u>Analyzed</u>
Aluminum	7372	mg/Kg	39.2	200.7	5/23/90
Antimony	BQL	mg/Kg	15.2	200.7	5/23/90
Arsenic	2.88	mg/Kg	2.0	206.2	5/18/90
Barium	58.2	mg/Kg	3.0	200.7	5/23/90
Berginstein	0.4	mg/Kg	0.2	200.7	5/23/90
Cadnil Lii.	BQL	mg/Kg	0.8	200.7	5/23/90
Calcium	62782	mg/Kg	106	200.7	5/23/90
Chromium	11.4	mg/Kg	1.6	200.7	5/23/90
Cobalt	3.86	mg/Kg	1.8	200.7	5/23/90
Copper	18.7	mg/Kg	3.4	200.7	5/23/90
Iron	8432	mg/Kg	16.8	200.7	5/23/90
Lead	95	mg/Kg	0.6	239.2	5/10/ 90
Magnesium	2098	mg/Kg	87	20 0.7	5/23/90
Manganese	321	mg/Kg	0.8	200.7	5/23/90
Mercury	0.1	mg/Kg	0.1	2 45.5	5/02/90
Nickel	8.6	mg/Kg	4.6	20 0.7	5/23/90
Potassium	864	mg/Kg	158	258.1	5/20/90
Selenium	BQL	mg/Kg	1.0	270.2	5/11/90
Silver	BQL	mg/Kg	3.8	200.7	5/23/9 0
Sodium	214	mg/Kg	110	273.1	5/20/90
Thallium	BQL	mg/Kg	2.0	279.2	5/17/90
Vanadium	17.2	mg/Kg	2.2	200.7	5/23/90
Zinc	381	mg/Kg	2.8	200.7	5/23/90
Lithium	BQL	mg/Kg	4.0	200.7	5/23/90
Molybdenum	BQL	mg/Kg	4.0	200.7	5/23/90

MKE DOCUMENT NO. 5121-C: DW - D - 01-0180-00

ENERGY

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DEPARTMENT

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OAK RIDGE, TENNESSEE	
CHEMICAL PLANT	
CONSTRUCTION DRAWINGS	
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TRANSFORMER LOCATION PLA	アプア

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CP - PCB REMOVAL



Department of Energy

Oak Ridge Operations
Weldon Spring Site
Remedial Action Project Office
Route 2, Highway 94 South
St. Charles, Missouri 63303
December 11, 1990

Addressees:

SCREENING LEVEL CHARACTERIZATION OF ELECTRICAL SUBSTATION 411 REPORT

Enclosed for your information is the subject document. This report contains data necessary to prepare a subcontract package for the removal of all transformers and dielectric fluids contained in substation 411.

If you have any questions please call Ken Lawver at (314)441-8978.

Sincerely,

Stephen H. McCracken

Project Manager Weldon Spring/Site

Remedial Action Project

Enclosure: As stated

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Kansas City, Kansas 66101

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Weldon Spring Training Area
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St. Charles, Missouri 63303

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UNITED STATES ENVIRONMENTAL PROTECTION AGENCY

REGION VII 726 MINNESOTA AVENUE KANSAS CITY, KANSAS 66101

NOV 9 1987

Mr. Rodney R. Nelson
U.S. Department of Energy
Weldon Spring Site Remedial
Action Project/Office
Route 2, Highway 94, South
St. Charles, Missouri 63303

Dear Mr. Welson:



We have reviewed the material you provided on the following interim response actions proposed for the Weldon Spring site:

- 1. Dismantling of Building 401
- 2. Dismantling of Building 409
- 3. Removal of PCB Transformers
- 4. Debris Consolidation

These actions, as with the four other interim response actions we have reviewed, should be useful in preparing for long-term remedial actions and have positive effects on health and safety and the environment. The documents developed in support of these proposals represent a significant improvement over those developed for the previous actions. Generally, we believe that a more thorough analysis and screening of response alternatives would be appropriate. Also, the documents do not contain sufficient detail of the work to be done to stand alone without the support of the technical specifications and drawings. Other comments and recommendations regarding these interim response actions are discussed below.

Building Demolition

- More specifics regarding the handling, storage and ultimate disposal of radioactively contaminated waste should be presented.
- The specifications state that "..., if chemically hazardous or toxic material is suspected or encountered, the Contractor shall be immediately notified...". What guidelines will be used by the demolition subcontractor to determine the hazard potential of unknown materials?
- The responsibility for determining whether a pollution condition has or will be created should be clearly specified.
- The specification does not state the health and safety requirements for the subcontractor.

CC's: <u>×</u>Rlowis Råntesal LåStewalt

DOCUMENT NUMBER: 1.02

2

PCB Transformer Removal

In this case, more detail in the site characterization section of the text would be appropriate. For instance, the PCB transformers are catagorized as those containing PCBs at concentrations greater than the concentrations in these transformers are in excess of 350,000 ppm.

Disposal facilities under consideration for receipt of these wastes must provide certification that they meet the Superfund offsite policy.

Debris Consolidation

It is stated in the description of the response action that one of the response objectives is to "Perform a detailed chemical and radiological characterization of the debris...". The description of the response action and specifications document contain no guidelines, references or information which would allow the subcontractor to complete this objective.

More specifics regarding the handling, storage and ultimate disposal of radioactive contaminated waste are needed.

If there are any questions, please do not hesitate to call.

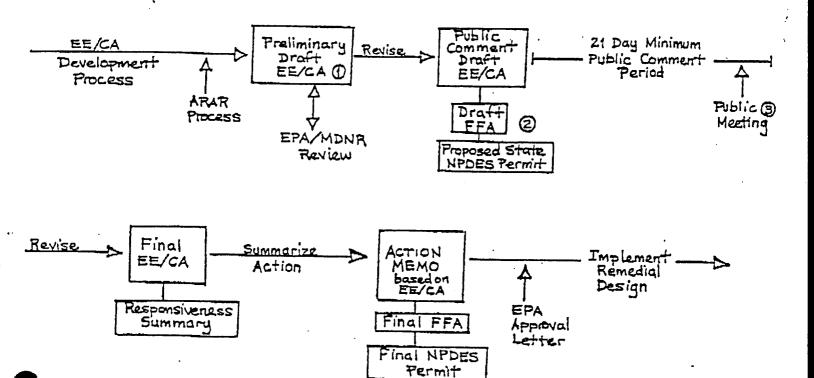
Sincerely yours,

B. Katherine Biggs

Chief, Environmental Review Branch

cc: Dave Bedan, MDNR

PROCESS FLOW CHART FOR DOE'S PROPOSED WASTEWATER TREATMENT AT THE WELDON SPRING SITE



- 1. Effluent limits are proposed by DOE based on BAT and/or ARARS.
- 2. It would be desirable to make the FFA and NPDES permit (if necessary) available for public comment along with the EE/CA.
- B. Timing of the public meeting can vary, however it is desirable to hold it after the EE/CA is made available for public comment so that it provides an opportunity for oral comment.

DRAFT

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IRA-100-102

MORRISON-KNUDSEN ENGINEERS, INC.

WSSRA PROJECT REVIEWS AND APPROVALS

Subject:	WSSRA PROJECT - CP	5121-C:SP-S-01-0217
	Specification Section 02090	(DOCUMENT NO
	Removal of Electrical Transformers	, , , , , , , , , , , , , , , , , , , ,
-		

Prepared:

Reviewed:

Signature Date

SEarch 5-19-89

Date

5-19-89

Approved, MKE:

- Site Design Engineer
- Engineering & Design Manager
- Chief Engineer, ESC Division

Alw Repport 5-19-87
Philip & Thuener 5-19-87

Z. Juan 22 May 87

- PMC Engineering Manager

QA Compliance <u>PAC 5-22-87</u>

O.A. REVIEW
ENTRY 190. <u>N/A</u>
INIT AL DATE GLOSET

DOCUMENT NUMBER: 4-100+102-1.01

SECTION 02090

REMOVAL OF ELECTRICAL TRANSFORMERS

PART 1 - GENERAL

O.A. REVIEW ENTRY NO. _WA_ INT. JAH DATE 4/25/27

1.1 SCOPE

- A. The work required to be performed under this Subcontract consists of controlled removal and disposal of selected electrical switches, transformers and capacitors from the project site, including many which are polychlorinated biphenol (PCB) containing or contaminated.
- B. The Subcontractor shall furnish all labor, supervision, materials, analysis, equipment, and transportation necessary for the complete removal and disposal of PCB contaminated fluid, three existing drums of previous PCB-contaminated sampling materials, electrical equipment with contents and subsequent contaminated materials resulting from cleanup operations. The electrical equipment consists of thirty-seven switches and transformers with capacitors located on concrete pads, mounted on poles, and one rooftop around the site. Three non-PCB containing transformers have a PCB contaminated capacitor attached. The capacitors are to be removed from the site as part of this contract. All electrical transformers, switches, and capacitors to be removed or drained are listed and located on project Drawing No. 5121E-CP-583.

1.2 CODES AND REGULATIONS

- A. All work on this Subcontract shall comply with applicable codes and regulations including, but not limited to, the following:
 - 1. Environmental Protection Agency (EPA)
 - 40 CFR 761
 - 15 USC 2601 et. seq. (TSCA)
 - U. S. Department of Labor, Occupational Safety and Health administration (OSHA)

- 29 CFR 1926
- 29 CFR 1910
- B. Posting Regulations: Where required by regulations, copies of applicable EPA and OSHA regulations shall be obtained by the Subcontractor and posted at the job site.

1.3 HEALTH AND SAFETY

- A. Training: All workers shall have received a forty-hour course per 29 CFR 1910.120 covering the following: health and safety regulations; environmental regulations; use and fitting of respiratory protective equipment; use of other personal protective equipment; procedures for work, spill, transport, disposal and decontamination.
- B. The Subcontractor shall provide all workers with personal protective equipment accepted by regulatory standards. This equipment shall generally include: Respirator per 29 CFR 1910.134; Full Saranex-Tyvek coveralls; PCB permeation resistant gloves and boot covers; goggles or face splash shield.

1.4 SUBMITTALS

A. Work Plan:

The Subcontractor shall submit a detailed plan of the work schedule and procedures to be used in removing the electrical equipment. The plan shall include details concerning sequencing, oil draining and flushing procedure, methods for removing the carcass from mounting structure, transportation to disposal, final disposal and procedures transportation of custody confirmation. The Subcontractor shall meet with the Contractor prior to beginning work to discuss and obtain acceptance of the Plan.

PART 2 - PRODUCTS

(None)

PART 3 - EXECUTION

3.1 WORK PRACTICES

- A. All PCB contaminated transformers whose volume prohibits placement in a 55-gallon DOT 17H drum, shall be drained and flushed on-site prior to shipment to an EPA-approved facility. Those transformers which can be shipped in drums may be transported undrained only if the EPA-approved facility is capable and intends to incinerate the oil, transformer and drum as a unit. If transformers are transported intact, the transformers shall be secured against overturning and a spill containment system provided.
- B. PCB contaminated liquids may be transported by either Department of Transportation (DOT) bulk container or DOT approved drum containers.
- C. Transformers drained on-site shall be flushed with non-PCB-contaminated Kerosene and rinsed in accordance with 40 CFR 761.60(b)(1)(i)B. During connecting and disconnection of hoses, spill collection pans shall be positioned to capture any liquid drips.
- D. Preparations for Handling PCBs: Barricade area to keep out unauthorized personnel. Construct a temporary holding area for drums and empty equipment; cover the floor and spill berm with polyethylene. Protect the polyethylene floor with a protective covering such as plywood. The holding area shall be leak-proof. Protect all surfaces over which PCB will be transported with polyethylene sheeting and spill berms. Seal all floor drains. All polyethylene sheeting shall have 6 mils minimum thickness.
- E. Any contaminated spill shall be immediately cleaned up in accordance with 40 CFR 761 Subpart G.
- F. Subcontractor shall verify that electrical equipment is de-energized prior to proceeding with any work.

3.2 DISPOSAL

A. Transport and disposal shall be in accordance with all Federal, State and local laws and regulations.

- B. PCB Containing Oil: Oil which is 50 parts per million or more PCB shall be drained from the equipment, taking care to avoid spills, drips and splashes. The thoroughly drained equipment shall be rinsed as required by 15 USC 2601 et. seq. (TSCA).
- C. The drained oil and rinse solution shall be properly labeled and sent for disposal at an EPA-approved PCB incineration facility. The drained and rinsed equipment shall be placed in a leak tight package, consisting of PCB impermeable plastic and wooden crating. The crated equipment shall be properly labeled and sent for disposal at an EPA-approved PCB facility.
- D. PCB-Containing Transformers: Transformers containing PCB contaminated oil shall be properly packaged and transported directly to an EPA approved PCB disposal facility.
- E. Waste Tracking: The Subcontractor shall establish and maintain a waste tracking system which shall as a minimum contain the following information: equipment identification, volume of oil, PCB contamination level, drum number (if any), date filled, date transported, manifest number, transporter, vehicle, destination and date arrived at destination for disposal and a letter of destruction.
- F. All PCB containing and contaminated oil and all equipment which was filled with PCB-contaminated oil shall be accompanied in transport by an EPA uniform hazardous waste manifest. If the PCB waste is being disposed of in a state which has more stringent manifesting requirements, a state manifest may be substituted.
- G. Non PCB oil shall be drained, removed from site and incinerated.
- H. All PCB containing and contaminated oil and all equipment which was filled with PCB containing oil shall be transported to the disposal site by a transporter licensed by the Missouri Department of Natural Resources.

END OF SECTION 02090

IRA-100-103

1.5



Department of Energy

Oak Ridge Operations
Weldon Spring Site
Remedial Action Project Office
Route 2, Highway 94 South
St. Charles, Missouri 63303

October 16, 1987

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Ms. Katherine Biggs
United States Environmental
Protection Agency
Region VII
726 Minnesota Avenue
Kansas City, Kansas 66101

Dear Ms. Biggs:

INTERIM RESPONSE ACTIONS (IRA'S)

Enclosed are six (6) copies of the documentation for the following four (4) Interim Response Actions:

- 1. Dismantling of Building 401
- 2. Dismantling of Building 409
- 3. Removal of PCB Transformers
- 4. Debris Consolidation

In addition, we are sending under separate cover, six (6) copies of the technical specifications and drawings from each of the four (4) proposed bid packages.

It is our intention to have copies of these documents in place in the repositories for public inspection, and to provide public notice of their availability on October 19, 1987. This will initiate the twenty one (21) day comment period.

If you have any questions, please give me a call.

Sincerely,

Rod Nelson

Project Manager Weldon Spring Site

Remedial Action Project

Enclosures: As stated

cc w/enclosures: D. Bedan, MDNR

REMOVAL OF PCB TRANSFORMERS

Site Background

The Weldon Spring site is located in St. Charles County, Missouri, about 48 km (30 mi) west of St. Louis. From 1941 to 1944, the U.S. Department of the Army operated the Weldon Spring Ordnance Works at the site for production of trinitrotoluene and dinitrotoluene. In the mid 1950s, a portion of the property was transferred to the U.S. Atomic Energy Commission (AEC), a predecessor of the U.S. Department of Energy (DOE).

From 1957 to 1966, the AEC operated a uranium processing facility at the Weldon Spring site. Impure uranium ore concentrates and some scrap uranium metal were processed at the chemical plant, and thorium-containing materials were also processed on an intermittent basis. Following closure by the AEC, the Army reacquired the chemical plant in 1967 and began converting the facilities to produce herbicides. The buildings were partially decontaminated and some equipment was dismantled. In 1969, prior to becoming operational, the herbicide project was canceled. Since that time, the plant has remained essentially unused and in caretaker status. The Army returned a portion of the Ordnance Works property to the AEC in 1971 but retained control of the chemical plant buildings. In 1984, the Army repaired several of these buildings; decontaminated some of the floors, walls, and ceilings; and removed some contaminated equipment to areas outside of the buildings. In 1985, custody of the chemical plant property was transferred to DOE. Currently, more than 70 inactive electrical transformers and switches are located in buildings and on external pads and poles throughout the Weldon Spring site.

Site Characterization

In order to characterize the potential hazards related to contamination by polychlorinated biphenyls (PCBs) at the Weldon Spring site, a sampling effort was conducted during March and April of 1987 to determine the types, locations, capacities, and levels of PCBs associated with transformers and switches. Sampling locations are shown in Fig. 1. Based on the results of this survey, on-site transformers and switches are grouped in Table 1 according to the EPA classification system for PCB-containing articles, as specified in 40 CFR Part 761.

Four on-site transformers were not sampled and therefore were not classified. Two were not sampled because of their proximity to energized electrical lines; the third was not sampled because it belongs to St. Charles County Water and is still in service; and the fourth was overlooked during the survey because it was lying on its side in the grass adjacent to Storage Building 436. Up to 76 liters (20 gallons) of PCB-containing dielectric fluid could be contained in this transformer.

Inspection of 31 other transformers indicated that they are air-cooled models. Follow-up surveys identified a number of additional air-cooled switches and transformers within scattered buildings and on the roof of Laboratory Building 407. These

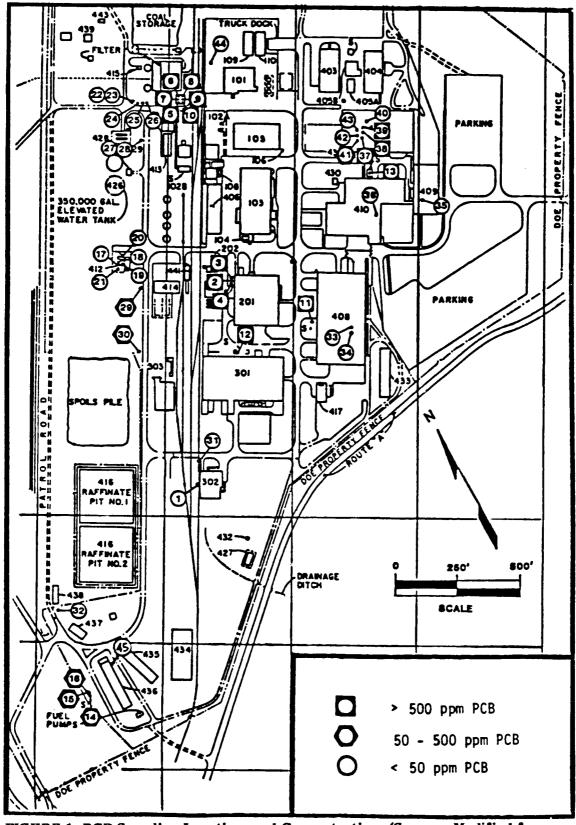


FIGURE 1 PCB Sampling Locations and Concentrations (Source: Modified from MK-Ferguson and Jacobs 1987)

TABLE 1 Summary of PCB Classification for Oil-Cooled Transformers and Switches at the Weldon Spring Site

	PCB Level (ppm)	Number	Estimated Volume		•	
Equipment			Liters	Gallons	Type of Mounting	
PCB transformer	>500	12	20,810	5,370	Pad	
PCB switch	>500	3	2,910	750	Pad	
PCB-contaminated transformer	50-500	5	1,290	334 ^a	Pad(3), pole(2)	
PCB-contaminated switch	50-500	1	1,270	334	Wall	
Non-PCB transformer	<50	12		2	Pad(5), pole(7)	
Non-PCB switch	<50	2	29,740	7,675 ^a	Wall	

^aRepresents combined volume from transformers and switches.

Source: Data from Meyer (1987).

transformers contain no dielectric fluids and therefore do not contribute to the potential PCB hazard on-site. In addition, none of the transformers recently installed to support the remedial action program contain PCBs because they were installed after PCB control regulations became effective.

As part of the effort to characterize on-site electrical equipment, an extensive radiological survey was conducted to determine whether the transformers and switches would meet residual surface contamination guidelines for unrestricted off-site disposal. Results of the survey indicate that all equipment meets DOE criteria for unrestricted release (MK-Ferguson and Jacobs 1987).

Threat to Public Health and the Environment

A public health and environmental hazard exists at the site due to the presence of abandoned electrical transformers and switches. The equipment has begun to show signs of deterioration that could result in leakage of PCB-containing oils. If this equipment remains on-site, its continued deterioration could result in a significant exposure hazard to site personnel. In addition, the safety of workers could be threatened by the deterioration of associated structural equipment (e.g., mounting supports).

Response Objectives

The objectives of this response action are as follows:

- 1. Reduction of the potential health hazard due to the presence of PCB-containing oils in electrical equipment on the Weldon Spring site; and
- 2. Removal of the potential safety hazard associated with structural deterioration of the equipment.

Proposed Response Action Alternatives

Interim response actions are designed to ensure the health and safety of on-site personnel and to minimize or preclude off-site releases of contamination. These actions are limited to those that can be performed under the Comprehensive Environmental Response, Compensation, and Liability Act/Superfund Amendments and Reauthorization Act and remain within the constraints of the Council on Environmental Quality's regulations for the National Environmental Policy Act (i.e., actions will be limited to those that do not have an adverse environmental impact nor limit the choice of reasonable alternatives).

Alternative response actions identified for inactive electrical equipment on the Weldon Spring site are:

- 1. No action;
- 2. Transport of intact switches and transformers off-site to a licensed treatment/disposal facility;
- 3. Draining and flushing of switches and transformers, and on-site storage of the empty units, PCB-containing oils, and flushing solutions;
- 4. Draining and flushing of switches and transformers, on-site storage of the empty units, and transport of PCB-containing oils and solutions off-site to a licensed treatment/disposal facility; or
- 5. Draining and flushing of switches and transformers, transport of the empty units off-site to a licensed landfill, and transport of PCB-containing oils and solutions off-site to a licensed treatment/disposal facility.

Analysis of Alternatives

Alternative 1 affords no reduction in potential threats to the health and safety of on-site personnel posed by PCB-containing electrical equipment at the Weldon Spring

site. There would be no improvement in environmental conditions at the site if no action were taken. This alternative presents no technical barriers and costs nothing in the short term. However, the equipment is scheduled for eventual disposal. The costs associated with deferred disposal would be higher than those for disposal at the current time, due to monitoring and maintenance activities required until future disposal. Most importantly, Alternative 1 is effectively precluded by institutional factors related to the community's strong desire for timely response actions at the Weldon Spring site.

Alternatives 2 through 5 are all technically feasible. Alternative 2 would be neither environmentally desirable nor cost-effective. Not only would there be an increase in costs related to the receiving facility, but bulk transport of the full containers would be less environmentally efficient than separation of hazardous from nonhazardous materials prior to treatment/disposal. In addition, the packaging, loading, transport, and unloading of the deteriorating equipment would entail considerable expense and effort to ensure minimization of the exposure threat to workers and the potential for PCB releases to the environment. Although the costs associated with Alternative 3 would be lower in the short term, this alternative would prove more expensive than Alternative 4 or 5 in the long term due to the monitoring and maintenance activities necessitated by controlled on-site storage of PCB-containing More importantly, Alternative 3 would be less desirable with regard to material. potential health and environmental effects than Alternative 4 or 5 because the PCBcontaining fluids would remain on-site. Alternatives 4 and 5 are both environmentally effective because each would involve the off-site transport of these fluids. Even though Alternative 4 would be less expensive than Alternative 5, it is not consistent with DOE's intention to dispose of all nonradioactive waste off-site. In addition, Alternative 4 does not fully address the public sentiment for expedited response action at the Weldon Spring site. Therefore, following the screening and analysis process for interim response action alternatives, Alternative 5 has been identified as the preferred alternative.

Description of Proposed Action

The proposed interim response action involves the following operations.

- 1. Draining PCB-containing oils from on-site switches and transformers:
- Flushing switches and transformers with an equal volume of a kerosene (or equivalent) solution;
- 3. Transporting the cleaned switch and transformer units off-site to a licensed facility; and
- 4. Transporting the PCB fluids and flushing solutions off-site to a licensed treatment/disposal facility.

The flushing and removal of PCB-containing electrical equipment from the Weldon Spring site will be performed in compliance with all applicable regulations and

procedures. This compliance will minimize the potential health threat to on-site personnel associated with exposure to PCBs and will also remove the safety hazards associated with the deteriorating electrical equipment. In addition, removal of the equipment will preclude the potential release of PCBs and will thus improve environmental conditions at the site. Finally, the proposed response action is consistent with DOE's goal to remove hazardous waste from the site.

The waste volumes associated with this effort are estimated to be 400 m^3 (500 yd³) of rinsed equipment and a combined volume of 109,500 liters (28,260 gallons) of PCB-containing oils and flushing solutions.

References

- Meyer, K., 1987, Transformer Sampling for PCB Fluids, prepared for MK-Ferguson Company, Weldon Spring, Mo. (April).
- MK-Ferguson Company and Jacobs Engineering Group, 1987, Radiological Survey Report for the Transformers at the WSS, prepared for U.S. Department of Energy, Oak Ridge Operations Office, Oak Ridge, Tenn. (Sept.).
- U.S. Atomic Energy Commission, 1960, Expansion Program at St. Louis Area Project No. 224-5066A Project History and Completion Report, Oak Ridge Operations Office, Oak Ridge, Tenn. (Oct.).

IRA-100-104



UNITED STATES ENVIRONMENTAL PROTECTION AGENCY

REGION VII 726 MINNESOTA AVENUE KANSAS CITY, KANSAS 66101

November 13, 1987

Mr. Rodney R. Nelson
U.S. Department of Energy
Weldon Spring Site Remedial
Action Project/Office
Route 2, Highway 94, South
St. Charles, Missouri 63303

Dear Mr (Ne Ison:



Enclosed is a copy of the comments we received from the Missouri Department of Natural Resources for the following four interim response actions:

- Dismantling of Building 401,
- 2. Dismantling of Building 409,
- 3. Removal of PCB Transformers, and
- 4. Debris Consolidation.

Any questions on these comments should be directed to David Bedan. To date, we have not received any other comments.

Sincerely yours,

P wather:

B. Katherine Biggs Chief, Environmental Review Branch

Enclosure

cc: David Bedan

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Division of Environmental Quality
Division of Environmental Quality
Division of Centings and Land Survey
Division of Management Services
Division of Parks, Recreation,
and Historic Preservation

JOHN ASHCROP

FREDERICK A. BRUNNER

STATE OF MISSOURI DEPARTMENT OF NATURAL RESOURCES

DIVISION OF ENVIRONMENTAL QUALITY P.O. Box 176 Jefferson City, MO 65102

November 12, 1987

Katie Biggs Environmental Review Branch U.S. EPA, Region VII 726 Minnesota Avenue Kansas City, KS 66101

Dear Ms. Biggs:

The Missouri Department of Natural Resources has reviewed four interim response actions (Group II) which the U.S. Department of Energy (U.S. DOE) has proposed for the Weldon Spring Site. These actions are:

- Dismanting and Disposal of Building 401
- Dismentling and Disposal of Building #09
- Removal of PCB transformers
- 4) Cebris Consolidation

MDNR supports the concept of interim response actions if they improve selety conditions or the environment, facilitate later remedial actions, and do not prejudice the decision on the final remedial action for the radioactive wastes. Based on the information provided by the U.S. DOE the MDNR concurs that these actions should be initiated immediately subject to the following comments and requirements:

1) <u>Dismentling and Dismosal of Cuilsing 401 and 409</u>

These activities are subject to both the Missouri Air Conservation Law and the Missouri Solid Waste Management Law. Missouri has adopted the federal Clean Air Act Standards for asbestos handling and has been delegated responsibility for implementing these standards. The Department of Energy and its contractors should develop and maintain close contact with the Missouri Air Pollution Control Program to Insure Compliance with these standards.

The Missouri Department of Natural Resources also regulates the disposal of asbestos and other demolition wastes under the Missouri Solid Waste Management Law. Demolition waste is required to be disposed of in a state permitted samitary or demolition landfill: aspestos waste is required to be disposed of in a permitted samitary landfill.

Ms. Katie Biggs November 12, 1987 Page 2

Because of the special concerns relating to the volume of waste and to the possible contamination of the aspestos and the other demolition material with hazardous wastes or radioactive wastes, the Department has determined that these materials should be handled as "special wastes". "Special Wastes" means solid wastes requiring handling other than that normally used for municipal wastes. Since radioactive wastes and hazardous wastes are excluded from disposal in sanitary landfills and demotition landfills in Missouri, the Department of Natural Resources cannot approve the disposal of the asbestos and other demolition wastes until a procedure is in place to assure us that no radioactive or hatardous materials are being disposed of in Missouri solid waste landfills. Also of concern is the large volume of material to be disposed of. Therefore, when the specific landfill has been chosen for disposal of this material, the generator of the waste (SOE) and the operator of the landfill must jointly make application to the Missouri Department of Natural Resources for special waste disposal approval. Please contact the Waste Management Program for further information on the approval process. If the waste is to be disposed of in another state. DOE must document that the action is in compliance with the waste management laws of that state.

Alternative 6 (on-site disposal) is rejected because "it is not consistent with DOE's intention to dispose of all non-radioactive waste off-site". DOE should provide justification for this off-site disposal policy.

2) Removed of PCB thensionmens

MDNR recommends that if Alternative #5 is used, during the "flushing" process care should be taken to contain aby spilled material. Also "flushing" should be continued until 9.2 reveis are less than 2 ppm. if transformer and switch carcasses are going to be disposed into a permitted sanitary landfill.

If the $i \in B$ liquids are being transported to a facility within Missouri, a licensed hazardous waste transporter must be used. If the PCBs are being transported to an out-of-state facility MONR recommends that a licensed transporter be used although it is not a requirement.

In the phromble to 40 (FR 761, unless otherwise tested, all dielectic transformers are assumed to contain 50-500 ppm PCB, therefore, untested transformers (22, 32, 3.45) should be "flushed" with other transformers.

External pad, poles, and adjacent areas should be tested to determine if ${\cal P}({\mathbb R})$ contamination exists.

Ma... Katle Biggs November 12, 1987 Page 3

3) Debris Consolidation

The storage of solid waste on site may be subject to the requirements of the Missouri Solid Waste Management Law. Please contact the Missouri Waste Management Program for assistance in determining whether these requirements apply.

Please contact me if you have any questions on these comments.

Sincerely.

DIVISION OF ENVIRONMENTAL QUALITY

David E. Bedan

Weldon Spring Site Work Group Coordinator

DEB/jtw

co: Rob Kucera, Deputy Director, DNR Caroly: deRous, legal Counsel Willium C. Ford, Director, DEQ Nick Di Pasquale, Director, WMP Nick NikVila, Director, APCP Don Maddox, SURO Bill Dieffenbach; MDCKI John Crellin, MDOH Rod Melson, WSSRAP, U.S. ECC



UNITED STATES ENVIRONMENTAL PROTECTION AGENCY

REGION VII 726 MINNESOTA AVENUE KANSAS CITY, KANSAS 66101

NOV 18 1987

OFFICE OF THE REGIONAL ADMINISTRATOR

Mr. Rodney R. Nelson
U.S. Department of Energy
Weldon Spring Site Remedial
Action Project/Office
Route 2, Highway 94, South
St. Charles, Missouri 63303

Dear Mr. Nelson:

We have reviewed the Department of Energy's (DOE) proposals for the following four interim response actions:

- Dismantling of Building 401,
- Dismantling of Building 409,
- Removal of PCB Transformers, and
- · Debris Consolidation.

Our comments on these proposals were sent to you earlier. You were also provided comments by the Missouri Department of Natural Resources (MDNR). No comments from the public were directed to the Environmental Protection Agency (EPA) and according to our records, there has been no public comment directed to MDNR or DOE.

We are in agreement these actions should proceed to ensure worker safety and reduce the further release of contaminants from this site. The EPA hereby approves these actions under the condition that the comments earlier provided by EPA and MDNR are adequately addressed. The MDNR has notified me they also concur with these actions. Please provide copies of any summary reports for these actions to EPA and MDNR.

We also received copies of the interim response action for construction of the Ash Pond Dike. We will provide any comments on this proposed action within the agreed upon 21-day comment period. We are most pleased to see that activities are underway to stabilize the site and reduce contaminant release.

Sincerely yours

Morris Kay

Regional Administrator

cc: Dr. Fred Brunner, MDNR

11. 23-87 Jape 11. 23-87 DEC 0 9 1987

Ms. B. Katherine Biggs
United States Environmental
Protection Agency
Region VII
726 Minnesota Avenue
Kansas City, Kansas 66101

Dear Ms. Biggs:

USEPA COMMENTS ON INTERIM RESPONSE ACTIONS (IRA'S)

Enclosed is our response to the comments contained in your letter of November 9, 1987, regarding the following interim response actions:

- 1. Dismantling of Building #401
- 2. Dismantling of Building #409
- 3. Removal of PCB Transformers
- 4. Debris Consolidation

We anticipate that this will adequately resolve the issues raised. We intend to proceed with action on these items in accordance with the enclosure.

If you have any questions, please give me a call.

Sincerely,

R. R. NELSON

Rod Nelson Project Manager Weldon Spring Site Remedial Action Project

Enclosure: As stated

cc: Dave Bedan, MDNR

PEER: JCoyne: x41: mw: 12/04/87: (c: EPA-IRA'.Ltr.)

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RESPONSIVENESS SUMMARY

B. Katherine Biggs letter to Rodney R. Nelson, dated November 9, 1987 re:

Interim Response Actions

- Dismantling of Building #401
- Dismantling of Building #409 Removal of PCB Transformers 2.
- 3.
- Debris Consolidation

General

Comment: Generally, a more thorough analysis and screening of response alternatives would be appropriate.

Response: This comment was reviewed with the EPA (telecon from Rod Nelson to Dan Wall dated 11-17-87). The EPA agreed that while additional analysis and screening is not required for the four (4) IRA proposals addressed herein, future proposals such as the Ash Pond Isolation Dike will present a more thorough analysis of response alternatives.

Comment: The documents do not contain sufficient detail of the work to be done to stand alone without the support of the technical specifications and drawings.

Response: Technical specifications and drawings will continue to accompany the IRA proposal packages submitted for review.

Building Demolition

Comment: Specifics of handling, storage, and ultimate disposal of radioactively contaminated waste should be presented.

Response: Radioactively contaminated waste from demolition of Buildings #401 and #409 will be segregated and stored on-site in a dry, concrete floored building, Building #434 and/or Building #406. Ultimate disposal will be in accordance with the RI Plan/EIS. Specifics of handling this waste will be covered in the Contractor's operational work plan which will integrate the specification and drawings, the WSSRAP Construction Safety and Health Management Program, applicable WSSRAP Standard Operating Procedures and Plans along with the subcontractor dismantling plan. This work plan will be finalized prior to the Subcontractor(s) starting demolition work.

response, txtsheil

Comment: What guidelines will be used by the demolition subcontractor to determine the hazard potential of unknown materials encountered in the work?

Response: Subcontractor personnel who will work on the site will be required to undergo a minimum of 40 hours of initial instruction in hazardous waste operations prior to starting work on site in accordance with 29CFR1910.120. In addition Subcontractor personnel will receive indoctrination training in the known hazards in the work area prior to start of work in accordance with the WSSRAP Construction Safety and Health Management Program and Special Conditions requirement of the subcontract. Unknown (unidentified or unmarked) chemical substances encountered in the work shall be considered potential hazards and shall be reported to the Contractor in accordance with the requirements of the specifications.

The Contractor will also provide health physics, construction safety and industrial hygiene surveillance on a routine basis during all stages of the work. This will include inspections of all work areas to identify potential hazards. Where required, the Contractor will collect bulk samples to identify any unknown or suspected substances. The Contractor will also perform air monitoring, as necessary and prudent, to assess exposure levels of hazardous substances in the workplace.

Comment: The responsibility for determining whether a pollution condition has or will be created should be clearly specified.

Response: The WSSRAP Construction Safety and Health Management Program which is an integral part of site subcontracts assigns responsibility for the identification of potential pollution (environmental) conditions to the Project Management Contractor. The Subcontractor is contractually required to comply with the requirements of the Clean Air Act and the Clean Water Act.

Comment: The specification does not state the health and safety requirements for the subcontractor.

Response: Subcontractor health and safety requirements are defined in the Special Conditions to the subcontract. The Special Conditions bind the Subcontractor to compliance with the WSSRAP Construction Safety and Health Management Program and all applicable Federal, State, and local health and safety regulations and standards listed therein. The Special Conditions are a supplement to the General Conditions and General Provisions which also contain basic health and safety requirements.

PCB Transformer Removal

Comment: In this case, more detail in the site characterization section of the text would be appropriate. For instance, the PCB transformers are categorized as those containing PCBs at concentrations greater than 500 ppm. It may be somewhat misleading not to indicate in the text that the concentrations in these transformers are in excess of 350,000 ppm.

Response: The final subcontract work package includes a table on the subcontract drawings listing each electrical component in the scope of work. This table includes the PCB concentration and volume capacity, in gallons, of each electrical component.

Comment: Disposal facilities under consideration for receipt of these wastes must provide certification that they meet the Superfund offsite policy.

Response: The Work Plan specified in Section 1.2A of Specification Section 02090 includes provision for meeting all requirements of 40CFR761. The Subcontractor's Work Plan in section 1.4A will be required to contain certification that the facilities selected for disposal of the waste material (1) have received written approval from the U.S. Environmental Protection Agency as required under 40CFR Part 761.70 or 761.75, as applicable, and (2) are not under a state or federal compliance order under CERCLA or RCRA.

Debris Consolidation

Comment: It is stated in the description of the response action that one of the response objectives is to "Perform a detailed chemical and radiological characterization of the debris...". The description of the response action and specifications document contain no guidelines, references or information which would allow the Subcontractor to complete this objective.

Response: This objective is to be completed by the Contractor and does not require any special activity by the Subcontractor. Radiological guidelines to be used by the Contractor in performing this characterization are as defined in Draft DOE Order 5480.11 and applicable WSS operating procedures. All debris will be visually inspected for potential chemical contamination. Where chemical contamination is observed or suspected, sampling and analyses will be performed to identify the characteristics of the chemical.

Comment: More specifics regarding the handling, storage and ultimate disposal of radioactive contaminated wastes are needed.

Response: Specifics of handling the radiologically contaminated debris will be finalized upon submittal of the Subcontractor's work plan. That plan will be integrated with applicable WSSRAP Standard Operating Procedures and Plans, the WSSRAP Construction Safety and Health Management Program, and the specifications and drawings. The debris will be consolidated for temporary storage in a materials staging area. Details of the materials staging area will be presented in a separate IRA under preparation. Ultimate disposal of radioactive contaminated debris waste will be





Department of Energy

Oak Ridge Operations Weldon Spring Site Remedial Action Project Office Route 2, Highway 94 South St. Charles, Missouri 63303

December 21, 1987

Ms. B. Katherine Biggs
United States Environmental
Protection Agency
Region VII
726 Minnesota Avenue
Kansas City, Kansas 66101

Dear Ms. Biggs:

MDNR COMMENTS ON INTERIM RESPONSE ACTIONS (IRA'S)

Enclosed is our responsiveness summary for the comments contained in Dave Bedan's letter of November 12, 1987, regarding the following interim response actions:

- 1. Dismantling of Building #401
- 2. Dismantling of Building #409
- 3. Removal of PCB Transformers
- 4. Debris Consolidation

We anticipate that this will adequately resolve the issues raised. We intend to proceed with action on these items in accordance with the enclosure.

If you have any questions, please give me a call.

Sincerely,

Rod Nelson

Project Manager

Weldon Spring Site

Remedial Action Project

Enclosure:
As stated

cc: Dave Bedan, MDNR, w/enclosure
 Jack Hammond, MK-F, w/o enclosure

RESPONSIVENESS SUMMARY

B. Katherine Biggs letter to Rodney R. Nelson, dated 11-13-87 re: MoDNR comments on:

Interim Response Actions

- 1. Dismantling and Disposal of Building #401
- 2. Dismantling and Disposal of Building #409
- 3. Removal of PCB Transformers
- 4. Debris Consolidation

1. Dismantling and Disposal of Buildings #401 and #409

Comment: The DOE and its Contractors should develop and maintain close contact with the Missouri Air Pollution Control Program to assure compliance with Missouri Air Conservation Law and Missouri Solid Waste Management Law in carrying out these activities.

Response: The DOE and its subcontractor(s) will continue to keep the DNR Air Pollution Control Program office apprised of plans for work at the site involving removal, handling, storage, and/or disposal of asbestos materials.

Comment: Missouri Solid Waste Management Law requires demolition waste to be disposed of in a permitted sanitary or demolition landfill. Asbestos waste must be disposed of in a permitted sanitary landfill.

Response: The Specifications for this work will require that asbestos and other demolition debris be disposed of in accordance with the requirements of the Missouri Solid Waste Management Law.

Comment: DNR maintains that because of the special concerns relating to the volume of waste and to the possible contamination of the asbestos and the other demolition material with hazardous wastes or radioactive wastes, these materials should be handled as "special wastes".

Response: The DOE concurs that there are special circumstances that require handling of asbestos as "special waste". Specifications for the asbestos subcontracts contain this provision.

The pending subcontracts contain the "special waste" forms which will be included in the

subcontract work packages as matter of comity. Should subcontract efforts, cost or progress on these IRA's be impacted by this provision, the DOE will revisit this issue with the MDNR.

Comment: The DNR cannot approve the disposal of the asbestos and other demolition wastes until a procedure is in place to assure us that no radioactive or hazardous materials are being disposed of in Missouri solid waste landfills.

Response: Release standards are in place for controlling release of the rubble off site. Radiological survey and release plans will be developed for each work package involving removal and off-site disposal of materials to insure compliance with the standards.

Comment: DOE should provide justification for its policy to dispose of all non-radioactive building waste off site.

Response: The DOE policy is based on volume reduction and cost effectiveness. By disposal of nonradiological material in a sanitary or demolition landfill, there is a reduction in the amount of material (Volume Reduction) that will be encapsulated in any disposal cell. Secondly, costs for on-site disposal cells are high in comparison to disposal in sanitary or demolition landfills. Also, as an aside to the technical and cost effectiveness issues, the DOE currently has funding available. The site is still to be fully characterized and to delay demolition and disposal of clean materials would not allow these funds to be utilized and also would lead to overall slipping of the schedule.

2. Removal of PCB Transformers

Comment: MDNR recommends that if Alternative #5 is used, during the "flushing" process care should be taken to contain any spilled material. Also, "flushing" should be continued until PCB levels are less than 2 ppm, if transformer and switch carcasses are going to be disposed into a permitted sanitary landfill.

Response: The subcontract specifications for this interim response action include spill control provisions for draining and flushing operations. Spill control pans are specified to collect any spilled liquids. PCB transformers and other electrical

equipment which have been drained and flushed, as stated in the specifications, will be disposed of at an EPA approved PCB disposal facility, not a sanitary landfill.

Comment: If the PCB liquids are being transported to a disposal facility within Missouri, a licensed hazardous waste transporter must be used. If the PCBs are being transported to an out-of-state facility MDNR recommends that a licensed transporter be used although it is not a requirement.

Response: The specifications state that the transporter of the PCB liquids-and drained electrical equipment shall be licensed.

Comment: In the preamble to 40 CFR 761, unless otherwise tested, all dielectric transformers are assumed to contain 50-500 ppm PCB, therefore untested transformers (22, 32, and 45) should be "flushed" with other transformers.

Response: The three transformers which have not been sampled for PCBs will be treated as PCB-contaminated units unless future sampling is performed to otherwise classify them as non-PCB transformers or PCB transformers. Irrespective of the classification, these units will be drained and flushed on site, unless the disposal facility intends to incinerate them as intact units, as indicated in the subcontract specifications.

Comment: External pad, poles, and adjacent areas should be tested to determine if PCB contamination exists.

Response: Additional sampling for PCB contamination, in areas from which the PCB-containing transformers and other electrical components are to be removed, is planned as part of future chemical characterization activities at the site.

3. <u>Debris Consolidation</u>

Comment: The storage of solid waste on site may be subject to the requirements of the Missouri Solid Waste Management Law. Please contact the Missouri Waste Management Program for assistance in determining whether these requirements apply.

Response: An interim response action proposal is being prepared which will present plans for materials staging and interim storage of solid waste on site. We will contact the Missouri Waste Management Program Office for assistance in determining applicability of the Missouri Solid Waste Management Law to this work.

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INTERIM RESPONSE ACTION (IRA) ADMINISTRATIVE RECORD FILE IR-0200

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IR-0200-201-1.03	ASH POND ISOLATION DIKE PLAN, REV. 4				
IR-0200-201-1.04	ASH POND ISOLATION DIKE SECTIONS AND DETAILS, REV. 4				
202 Sampling and A	Analysis Data/Chain of Custody Forms				
IR-0200-202.1.01	CHEMICAL SOIL DATA REPORT TO SUPPORT INTERIM RESPONSE ACTIONS, CONSTRUCTION STAGING AREA, AND ADMINISTRATIVE BUILDING				
203 Engineering Ev	Engineering Evaluation/Cost Analysis				
IR-0200-203-1.01	EE/CA FOR CONSTRUCTION OF ASH POND ISOLATION SYSTEM				
204 Engineering E Action Decision	Engineering Evaluation/Cost Analysis Approval or Remedial Action Decision Document				
IR-0200-204-1.01	EPA APPROVAL FOR ASH POND ISOLATION DIKE				
205 Amendments to	Decision Document				

IRA-200-201

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3589-87-I-DOE-191



Department of Energy

Oak Ridge Operations
Weldon Spring Site
Remedial Action Project Office
Route 2, Highway 94 South
St. Charles, Missouri 63303

November 9, 1987

Ms. Katherine Biggs
United States Environmental
Protection Agency
Region VII
726 Minnesota Avenue
Kansas City, Kansas 66101

Dear Ms. Biggs:

INTERIM RESPONSE ACTIONS (IRA'S)

Enclosed are six (6) copies of the documentation for the construction of the Ash Pond Dike, including copies of the relevant portions of the technical specifications and drawings from the proposed bid packages.

It is our intention to have copies of these documents in place in the repositories for public inspection, and to provide public notice of their availability on November 10, 1987. This will initiate the twenty one (21) day comment period.

If you have any questions, please give me a call.

Sincerely,

R. R. Nelson Project Manager Weldon Spring Site Remedial Action Project

Enclosure: As stated

cc: D. Bedan, MDNR (6 copies)

CONSTRUCTION OF ASH POND ISOLATION DIKE

Site Background

The Weldon Spring site is located in St. Charles County, Missouri, about 48 km (30 mi) west of St. Louis. From 1941 to 1944, the U.S. Department of the Army operated the Weldon Spring Ordnance Works at the site for production of trinitrotoluene and dinitrotoluene. In the mid 1950s, a portion of the property was transferred to the U.S. Atomic Energy Commission (AEC), a predecessor of the U.S. Department of Energy (DOE).

From 1957 to 1966, the AEC operated a uranium processing facility at the Weldon Spring site. Impure uranium ore concentrates and some scrap uranium metals were processed at the chemical plant, and thorium-containing materials were also processed on an intermittent basis. Following closure by the AEC, the Army reacquired the chemical plant in 1967 and began converting the facilities to produce herbicides. The buildings were partially decontaminated and some equipment was dismantled. In 1969, prior to becoming operational, the herbicide project was canceled. Since that time, the plant has remained essentially unused and in caretaker status. The Army returned a portion of the Ordnance Works property to the AEC in 1971 but retained control of the chemical plant buildings. In 1984, the Army repaired several of these buildings; decontaminated some of the floors, walls, and ceilings; and removed some contaminated equipment to areas outside of the buildings. In 1985, custody of the chemical plant property was transferred to DOE.

Ash Pond is located in the far northwest section of the Weldon Spring site and has the lowest surface elevation on the site (Fig. 1). Water in Ash Pond is recharged by intermittent surface runoff and overflow from the St. Charles County water tower, which is located on the Weldon Spring site (U.S. Department of the Army 1976). The watershed of Ash Pond includes the area around raffinate pits 1 and 2 as well as the western quarter (about 25 ha [62 acres]) of the chemical plant area. Discharge from Ash Pond flows northward to Lake 35, an impoundment on Schote Creek in the Busch Wildlife Area (U.S. Department of Energy 1987a). There is a hydraulic connection between the Ash Pond outflow and Burgermeister Spring, which is also located in the Busch Wildlife Area. Routine environmental monitoring of intermittent surface runoff resulting from precipitation events has identified substantial levels of uranium contamination in the runoff from Ash Pond.

Site Characterization

A preliminary radiological survey of the Weldon Spring site was performed in 1975. Analysis of water samples from Ash Pond indicated that the concentrations of radium, thorium, and uranium were less than their maximum permissible concentrations as specified in 10 CFR Part 20 (Jacobson 1976). (At the time of the survey, 10 CFR Part 20 was the appropriate regulation because control of the site was under cognizance of the U.S. Department of the Army.) Recent radiological sampling identified uranium

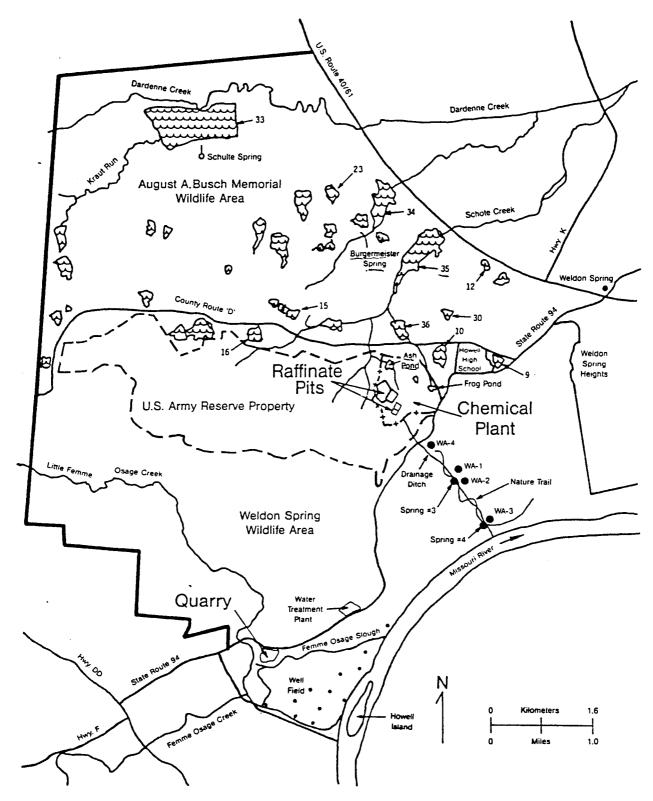


FIGURE 1 Map of the Weldon Spring Site and Vicinity (Source: Modified from U.S. Department of Energy 1987a)

concentrations as high as 4,000 pCi/L in surface runoff from Ash Pond and lower uranium levels of up to 400 pCi/L in surface waters upstream from Ash Pond (MK-Ferguson and Jacobs 1987).

A 1987 sampling of soils adjacent to Ash Pond identified the predominant radionuclides as radium and thorium. Contamination of soils in the Ash Pond area likely
derives from previous processing activities at the site, radionuclide migration from a
dump located adjacent to Ash Pond, and past discharges to the pond of decant liquids
from the area between raffinate pits 1 and 3 (MK-Ferguson and Jacobs 1987).
Background levels of radium-226, thorium-232, and uranium-238 — which were determined by sampling off-site locations — averaged 1.0, 0.8, and <1.9 (detection limit)
pCi/g, respectively.

A more extensive sampling of surface and subsurface soils was performed in areas adjacent to Ash Pond that were identified as potential borrow areas for the proposed dike construction project (Fig. 2). Radionuclide concentrations in these soil samples ranged from 0.3 to 2.9 pCi/g thorium and from 0.3 to 7.2 pCi/g uranium. All thorium measurements are below applicable DOE guidelines for unrestricted release (U.S. Department of Energy 1987b), and the uranium concentrations are below preliminary values specified for release for unrestricted use. Chemical analyses of area soil samples identified background concentrations of metals and nitroaromatics and only slightly elevated nitrate and sulfate concentrations. Although no known chemical hazards currently exist in the area, additional confirmatory soil sampling is planned.

Threat to Public Health and the Environment

A health and environmental hazard exists at the site due to high levels of uranium contamination in the surface waters of Ash Pond. The contamination poses a similar hazard off-site because at least a portion of the outflow from Ash Pond, which enters the subsurface just west of the site boundary, surfaces again at Burgermeister Spring in the Busch Wildlife Area. Lake 35 in the wildlife area also receives water from Ash Pond (MK-Ferguson and Jacobs 1987). Contamination of Lake 35 and Burgermeister Spring poses a potential health hazard to area personnel, the general public, and resident wildlife.

Response Objectives

The objectives of this response action are as follows:

- 1. Reduction of the potential on-site health hazard due to radiation exposure associated with uranium contamination of surface water in Ash Pond;
- 2. Reduction of the potential off-site health hazard due to radiation exposure associated with uranium contamination of receiving waters in the Busch Wildlife Area;

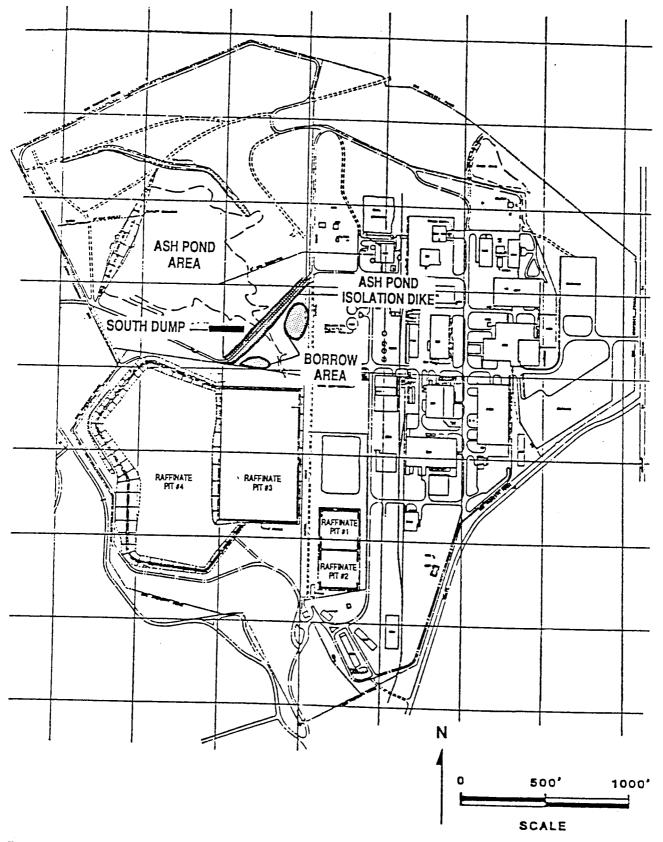


FIGURE 2 Location of Proposed Dike and Borrow Areas (Source: Modified from MK-Ferguson and Jacobs 1987)

- 3. Reduction of the potential for erosion and related resuspension and transport of the contaminated soils in Ash Pond;
- 4. Reduction of the surface water infiltration rate through contaminated soils in Ash Pond; and
- 5. Improvement in the quality of water being discharged from the site at Ash Pond.

Proposed Response Action Alternatives

Interim response actions are designed to ensure the health and safety of on-site personnel and to minimize or preclude off-site releases of contamination. These actions are limited to those that can be performed under the Comprehensive Environmental Response, Compensation, and Liability Act/Superfund Amendments and Reauthorization Act and remain within the constraints of the Council on Environmental Quality's regulations for the National Environmental Policy Act (i.e., actions will be limited to those that do not have an adverse environmental impact nor limit the choice of reasonable alternatives).

Alternative response actions identified for reducing radiological contamination of surface runoff from Ash Pond are:

- 1. No action;
- Excavation of contaminated soils in the Ash Pond location that are responsible for radiological contamination of surface flow through the area, with on-site storage of all material that exceeds the radiological criteria for unrestricted release (and on-site interim storage of any material that exceeds limits for chemical contamination, prior to its eventual transport off-site to a licensed treatment/disposal facility);
- 3. Construction of a dike at the site boundary downstream of the Ash Pond area to provide a retention basin for the contaminated water until its subsequent decontamination at an on-site water treatment plant; and
- 4. Construction of an isolation dike upstream of Ash Pond to prevent contact of the surface runoff with contaminated soils in the Ash Pond area, construction of diversion channels to route the water away from these contaminated locations for subsequent discharge at its current point, and concurrent or subsequent excavation of the contaminated soils with on-site storage of all material that exceeds the radiological criteria for unrestricted release (and on-site interim storage of any material that exceeds limits for chemical contamination prior to its transport off-site to a licensed treatment/disposal facility).

Analysis of Alternatives

Alternative 1 affords no reduction in the potential health threat posed by uranium contamination of surface runoff from Ash Pond. There would be no improvement in on-site or off-site environmental conditions if no action were taken. Although this alternative presents no technical barriers and costs nothing in the short term, Alternative 1 is effectively precluded by institutional factors related to the community's demand for timely response actions at the Weldon Spring site — in particular, a reduction in the release of contaminants off-site.

Alternatives 2 through 4 are technically feasible and would reduce the potential hazards associated with uranium contamination of surface runoff. Alternative 2 would be less expensive than Alternative 3 or 4 because it would not include costs associated with dike and channel construction. However, the location and extent of radiological contamination in the Ash Pond area has not yet been determined. Characterization of this area would have to be performed prior to the initiation of any excavation effort. Therefore, Alternative 2 would not satisfy institutional factors related to the community's strong desire for expedited response with regard to minimizing off-site releases of radiologically contaminated water.

Excavation of radioactive soils in the Ash Pond area following construction of the isolation dike would be less feasible for Alternative 3 than Alternative 4 because the retention pond would cover these soils, at least intermittently. Delayed decontamination of the soil (and potential resuspension and/or leaching of uranium) would prevent Alternative 3 from being fully responsive to institutional issues related to the need for timely cleanup action at the site. Costs associated with deferred excavation, and with the subsequent decontamination of stored water in a water treatment plant, would cause Alternative 3 to be more expensive than Alternative 4. In addition, factors related to the desire for minimizing the potential for off-site contaminant releases would not be completely addressed by Alternative 3. Ponding of water above areas of contaminated soil would increase the likelihood of infiltration through these areas and the resultant transport of radionuclides into the groundwater. In comparison, Alternative 4 would involve diversion of surface runoff away from contaminated areas and would effectively reduce the hydraulic head at Ash Pond, thereby reducing the potential for contaminant transport into groundwater. Finally, Alternative 3 is precluded by institutional factors related to construction of the treatment plant, i.e., approval for its construction has not yet been granted by the appropriate federal, state, or local agencies, and it could be a long time before the plant is built. In addition, because the allowable levels of radioactive and chemical contaminants in the treated water have not yet been determined, it is not possible to estimate the date by which a water treatment plant would be operational.

Therefore, following the screening and analysis process for interim response action alternatives, Alternative 4 has been identified as the preferred alternative.

Description of Proposed Action

The proposed interim response action involves restriction of flow across Ash Pond by construction of an upstream dike and diversion channels. The response action will include the following operations:

- Completion of the radiological and chemical characterization of the isolation dike area (radiological and chemical characterization of the remainder of the site will be performed at a later date following completion of the Soils Investigation Work Plan);
- Construction of an isolation dike upstream of Ash Pond measuring approximately 230 m (750 ft) in length and 3 m (10 ft) at its maximum height, containing about 5,400 m³ (7,000 yd³) of soil material, and creating a retention pond covering approximately 2.4 ha (6 acres);
- 3. Construction of diversion channels totaling approximately 610 m (2,500 ft) in length and measuring about 1 m (3 ft) in depth, which would circumvent Ash Pond and connect the dike to the current point of surface water discharge; and
- 4. Emplacement of a discharge monitoring station for intermittent measurement of water quality and continuous measurement of the quantity of surface water discharged from the Ash Pond area.

Under the proposed action, the dike and diversion channels will be constructed in full compliance with all applicable regulations and procedures. This compliance will ensure protection of the safety and health of on-site workers as well as limit off-site releases of contaminants. The proposed action would result in a decrease in the uranium concentration in discharged water from about 4,000 pCi/L to 400 pCi/L. The DOE uranium limit for water released off-site is 600 pCi/L. Hence, this action would reduce the uranium concentration in the water to levels below those allowed by DOE regulations. (The applicable limits to be used for the water treatment plant have not yet been determined.)

The dike and channels will be constructed of soil taken from adjacent borrow areas (Fig. 2), following final verification that the soil is neither chemically nor radio-logically contaminated. (Characterization efforts to date have identified negligible contamination of this soil.) If the results of the surface water monitoring indicate unacceptable levels of contamination at the point of discharge, the water could be pumped to a raffinate pit in lieu of being released off-site.

This interim response action is being taken to reduce the concentration of uranium in water leaving the site. The contaminated areas responsible for this contamination will be remediated in the future. All material that exceeds the radiological criteria for unrestricted use will be transported to a dry, concrete-floored building currently located at the Weldon Spring site or to an on-site material staging area

that may be constructed in the future. Material that exceeds the appropriate chemical contamination limits (to be developed), but is not radiologically contaminated, will be consolidated at an interim staging area on-site prior to its eventual transport off-site to a licensed treatment/disposal facility.

Implementation of the proposed response action at this time will minimize the potential adverse impacts on health and the environment resulting from continued runoff of highly contaminated surface water from the Ash Pond area.

References

Jacobson, J.R., 1976, Preliminary Radiological Survey of the Weldon Spring Chemical Plant, Memorandum Report No. 2610, prepared for the U.S.A. Ballistic Research Laboratories, Aberdeen Proving Ground, Md. (April).

MK-Ferguson Company and Jacobs Engineering Group, Inc., 1987, Characterization Report for the Ash Pond Isolation Dike Interim Response Action, prepared for U.S. Department of Energy, Oak Ridge Operations Office, Oak Ridge, Tenn. (Sept.).

- U.S. Department of the Army, 1976, Assessment of Weldon Spring Chemical Plant in St. Charles County, Missouri, Office of the Department of the Army Project Manager for Chemical Demilitarization and Installation Restoration, Aberdeen Proving Ground, Md. (March).
- U.S. Department of Energy, 1987a, Draft Environmental Impact Statement, Remedial Action at the Weldon Spring Site, Office of Remedial Action and Waste Technology, Washington, D.C. (Feb.).
- U.S. Department of Energy, 1987b, U.S. Department of Energy Guidelines for Residual Radioactive Material at Formerly Utilized Sites Remedial Action Program and Remote Surplus Facilities Management Program Sites (Revision 2, March).

PROPOSED INTERIM RESPONSE ACTION:

CONSTRUCTION OF ASH POND ISOLATION SYSTEM AT THE WELDON SPRING SITE

May 1988

PROPOSED INTERIM RESPONSE ACTION:

CONSTRUCTION OF ASH POND ISOLATION SYSTEM AT THE WELDON SPRING SITE

SITE BACKGROUND

The Weldon Spring site is located in St. Charles County, Missouri, about 48 km (30 mi) west of St. Louis. From 1941 to 1944, the U.S. Department of the Army operated the Weldon Spring Ordnance Works at the site for production of trinitrotoluene and dinitrotoluene. In the mid 1950s, a portion of the property was transferred to the U.S. Atomic Energy Commission (AEC), a predecessor of the U.S. Department of Energy (DOE).

From 1957 to 1966, the AEC operated a uranium processing facility at the Weldon Spring site. Impure uranium ore concentrates and some scrap uranium metals were processed at the chemical plant, and thorium-containing materials were also processed on an intermittent basis. Following closure by the AEC, the Army reacquired the chemical plant in 1967 and began converting the facilities to produce herbicides. Some of the buildings were partially decontaminated and some equipment was dismantled. In 1969, prior to becoming operational, the herbicide project was canceled. Since that time, the plant has remained essentially unused and in caretaker status. The Army returned a portion of the ordnance works property to the AEC in 1971 but retained control of the chemical plant buildings. In 1984, the Army repaired several of these buildings; decontaminated some of the floors, walls, and ceilings; and removed some contaminated equipment to areas outside of the buildings. In 1985, custody of the chemical plant property was transferred to DOE.

The Ash Pond area is located in the far northwest section of the Weldon Spring site and has the lowest surface elevation on the site (Fig. 1). Water is present only intermittently in Ash Pond and is recharged by surface runoff. The watershed of Ash Pond includes the area around the raffinate pits and and the western quarter (about 25 ha [62 acres]) of the chemical plant area (Fig. 2). Discharge from Ash Pond flows northward to Lake 35, an impoundment on Schote Creek in the August A. Busch Memorial Wildlife Area (U.S. Department of Energy 1987a). Based on the results of dye studies conducted at the site by the Missouri Department of Natural Resources in 1983, a hydraulic connection exists between the Ash Pond outflow stream and Burgermeister Spring, which is also located in the Busch Wildlife Area (Dean 1985). Routine environmental monitoring of intermittent surface runoff has identified substantial levels of uranium contamination in the runoff from Ash Pond.

In order to decrease the release of contaminants off-site, it is proposed that an isolation system (e.g., a dike and diversion channels) be constructed upstream of Ash Pond to limit the flow of surface water over the contaminated area. This report documents the proposed Ash Pond construction project as an interim response action.

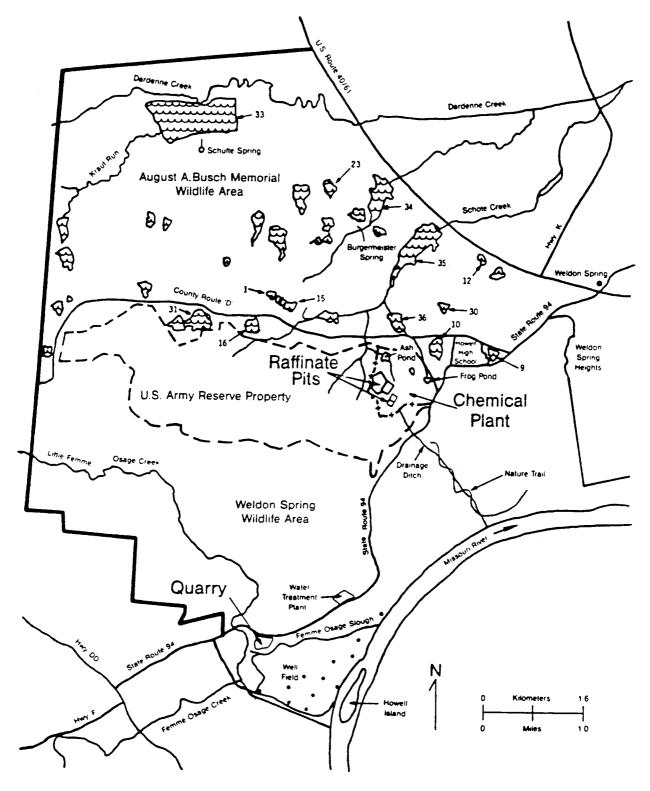


FIGURE 1 Map of the Weldon Spring Site and Vicinity (Source: Modified from U.S. Department of Energy 1987a)

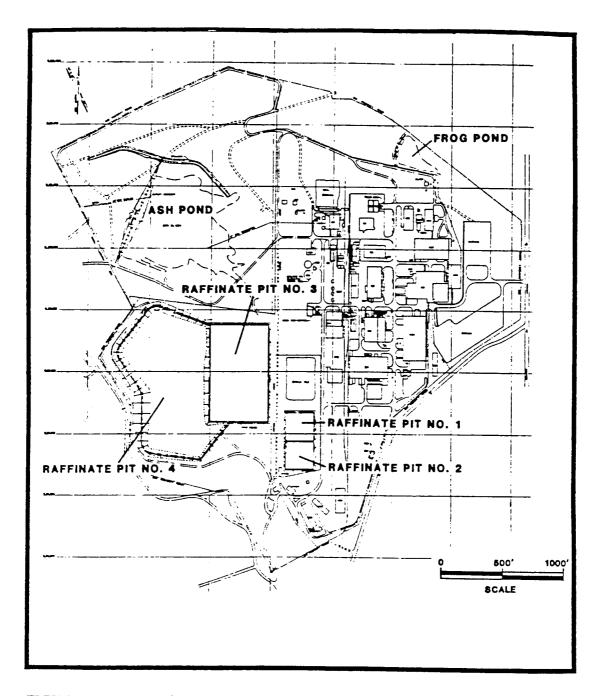


FIGURE 2 Layout of the Weldon Spring Raffinate Pits and Chemical Plant Area

SITE CHARACTERIZATION

Surface Water

A preliminary radiological survey of the Weldon Spring site was performed in 1975. Analyses of water samples from Ash Pond indicated that the concentrations of radium, thorium, and uranium were less than their maximum permissible concentrations (MPCs) as specified in 10 CFR Part 20 (Jacobson 1976; U.S. Department of the Army 1976). (At the time of the survey, 10 CFR Part 20 was the appropriate regulation because the site was under control of the U.S. Department of the Army.) Subsequent radiological sampling identified uranium concentrations in excess of the currently appropriate guideline, i.e., the DOE guideline for uranium-238 in water (600 pCi/L) (U.S. Department of Energy 1986). Levels as high as 4,000 pCi/L were detected in surface runoff from Ash Pond compared with levels up to 400 pCi/L in the watershed upstream from Ash Pond (Kleeschulte and Emmett 1986; MK-Ferguson and Jacobs Engineering 1987).

Recent characterization efforts at the Weldon Spring site have included more extensive sampling for uranium in surface runoff from the Ash Pond watershed. The locations of the sampling points, shown in Fig. 3, were selected because water passing between these points must cross over the known source of radioactive contamination in the watershed — i.e., the South Dump, which was used for disposal of contaminated material during both the uranium-processing period and the Army's decontamination effort at the site.

Results of the runoff sampling program are presented in Table 1. Because rainfall during the months of April, May, June, August, September, October, and November 1987 was insufficient to produce any flow from the watershed, no water samples were collected during those months. The variable results reflect the nature of the sampling method (i.e., grab samples) and the variable flow volumes. To permit the level of contamination to be more accurately determined, procedures and equipment for continuous monitoring and sequential sampling of surface runoff leaving the area were recently put in place; this effort was completed during May 1988.

Geology and Groundwater

During a recent comprehensive characterization of the Weldon Spring site, several boreholes were drilled in and near Ash Pond (see Fig. 4) to define the physical nature of the area. Analysis of these borehole samples indicated that layers of low-permeability clay are present in the area, with thicknesses ranging from 1.5 to 6 m (5 to 20 ft). The thinnest deposits are present in the existing drainage channel, where compacted fill would be placed during the proposed construction project. A cross section of the Ash Pond area is presented in Fig. 5.

Two piezometers placed in the overburden material immediately south of the proposed dike indicate that the local soil is unsaturated. Groundwater in the area occurs

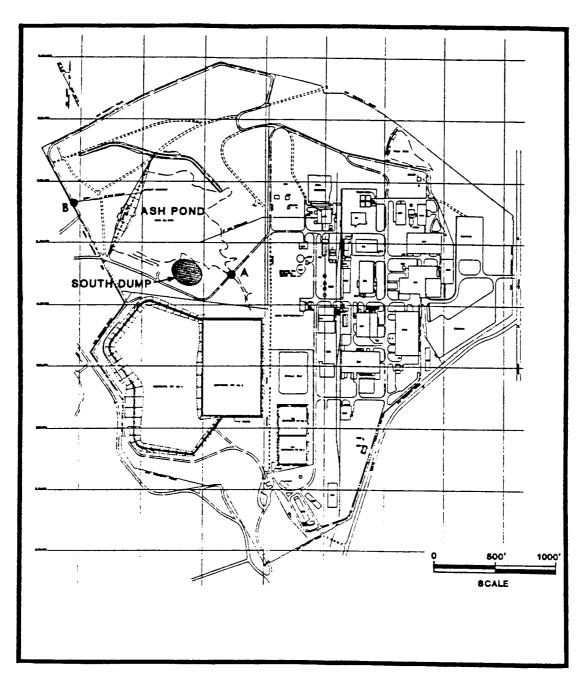


FIGURE 3 Sampling Locations for Uranium in Surface Water at Ash Pond

TABLE 1 Uranium Concentrations in Surface Runoff at the Ash Pond Sampling Points^a

	Natural Uranium (pCi/L)									
	1	987	1988							
Month	Point A	Point B	Point A	Point B						
January	_b	3,500	140 140 45 110 200	2,700 2,800 1,800 1,700 360						
February March	380	3,100 2,100	100 180	460 900						
July December	100 - - - -	250 960 1,500 1,200 1,800								

^aThe locations of Point A (upstream) and Point B (site boundary) are shown in Fig. 3.

in the bedrock, approximately 9 m (30 ft) below the ground surface. Groundwater recharge through this temporary impoundment would be minimal. However, should it occur, the underlying soils would be expected to adsorb contaminants and thus limit migration. (Soils in the area exhibit low hydraulic conductivity and favorable cation exchange properties.) In addition, the proposed upstream isolation dike and diversion channels would significantly reduce the amount of water entering the Ash Pond area, which is believed to be a shallow groundwater recharge area. The resultant decrease in hydraulic head would decrease the rate of infiltration through the contaminated locations in the Ash Pond area (e.g., the South Dump). Based on the thickness and nature of the soils in the affected area, the proposed Ash Pond dike and diversion system would not create a significant groundwater recharge zone. In addition, any water recharging the groundwater from this zone would contain lower levels of uranium than have been

^bA dash indicates that no data were collected.

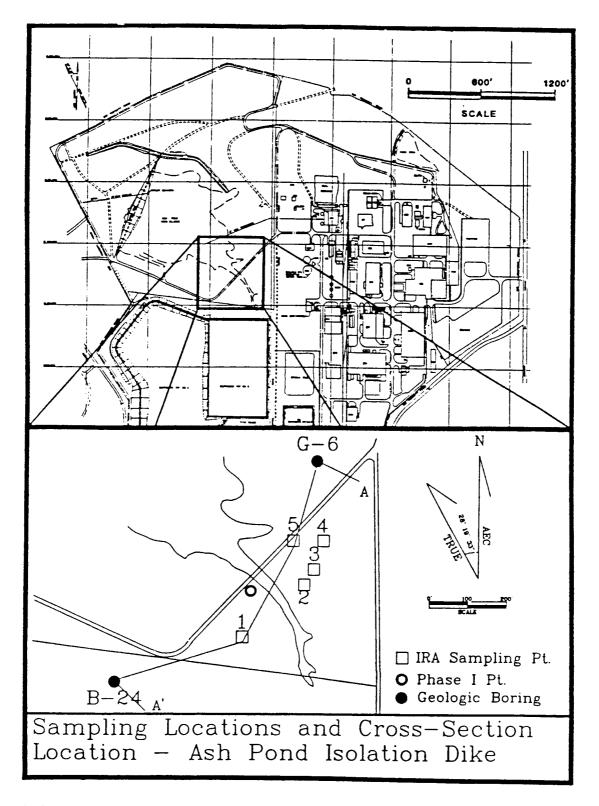


FIGURE 4 Location of Boreholes in the Ash Pond Area

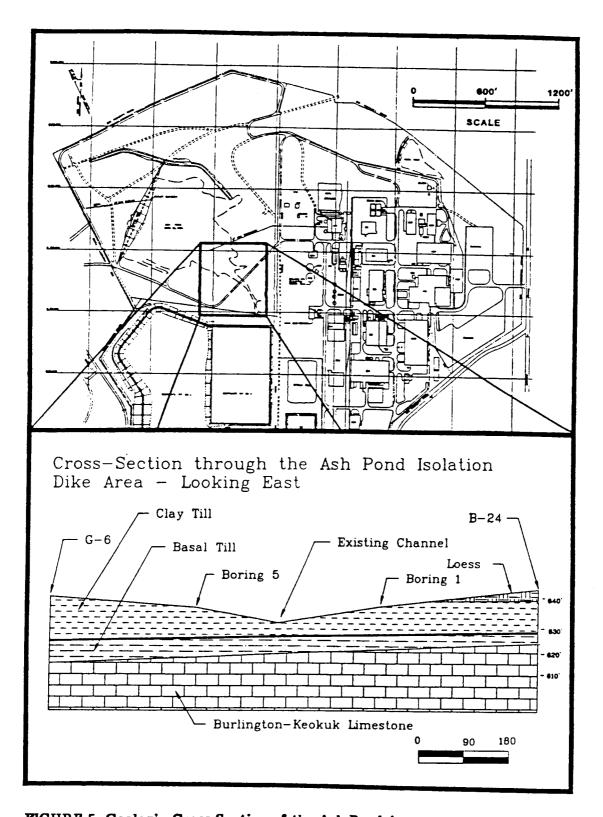


FIGURE 5 Geologic Cross Section of the Ash Pond Area

detected in the losing stream located immediately west of the site. Thus, an improvement in the quality of surface water leaving the Ash Pond area would also improve the quality of the subsurface system (MK-Ferguson and Jacobs Engineering 1988).

Soils

Soils in the Ash Pond area were probably radioactively contaminated as a result of previous processing activities at the Weldon Spring site, migration from the South Dump adjacent to Ash Pond, and past discharges to the pond of decant liquids from the area between raffinate pits 1 and 3 resulting from process line breakage. No known chemical hazards currently exist in the Ash Pond area (MK-Ferguson and Jacobs Engineering 1987).

The Phase I soil investigation program, consisting of a comprehensive radiological and chemical characterization of site soils, was recently completed at the Weldon Spring site (MK-Ferguson and Jacobs Engineering 1988). For the chemical characterization, subsurface soil samples were collected from several boreholes in and around the area proposed for the Ash Pond isolation system (see Fig. 4). These borehole samples Fig. 4) were analyzed for metals, nitroaromatics, inorganic ions (nitrate, sulfate, chloride, and fluoride), and moisture content. Select samples were also analyzed for semivolatile compounds, pesticides, and polychlorinated biphenyls. The results indicated that only nitrate and sulfate levels are slightly elevated, and no chemical hazards exist in the area proposed for the isolation system (MK-Ferguson and Jacobs Engineering 1988).

It is proposed that borrow material for construction of the Ash Pond isolation system be obtained from a spoils pile that is located north of raffinate pit 1 and east of raffinate pit 3 (Fig. 6). This pile probably resulted from the excavation of raffinate pit 4 and typically consists of clayey soils. The spoils pile was chemically characterized during the Phase I soil investigation program. Samples were collected from two locations in the pile and analyzed for metals, nitroaromatics, inorganic ions, and moisture content. No elevated concentrations of chemical contaminants were detected in the samples.

The Ash Pond and spoils pile areas were also surveyed for radiological contamination. The methods employed and values measured during this effort are described in detail in the radiological characterization report for the site (Marutzky et al. 1988). Sampling results for the spoils pile indicate that there is no uranium contamination and that concentrations of radium and thorium are below current DOE guidelines for residual radionuclides in soil (U.S. Department of Energy 1987b), which are provided in Appendix A. (Although DOE has established generic guidelines for radium and thorium in soil, there is no similar guideline for uranium. The guideline for uranium in soil is derived on a site-specific basis.) The pertinent results for the Ash Pond/South Dump area are summarized below.

The analyses of soil samples identified one area south of Ash Pond with a radium-226 concentration above the near-surface (i.e., upper 15 cm [6 in.]) soil guideline,

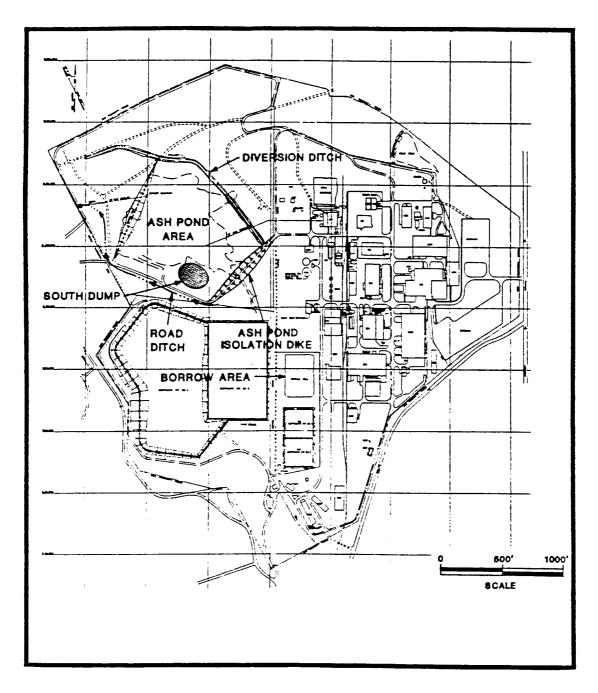


FIGURE 6 Location of Proposed Dike and Borrow Area (Source: Modified from MK-Ferguson and Jacobs Engineering 1987)

but there were no measurements above the guideline for thorium-232 in the area affected by the proposed isolation system. Uranium contamination was detected in the South Dump.

Spectrometric measurements identified two locations southeast of Ash Pond with radium-226 concentrations above the near-surface soil guideline, but no measurements of thorium-232 in the area exceeded the appropriate guideline. Exposure-rate measurements were above background levels in the South Dump.

The subsurface drilling and sampling effort identified the presence of elevated thorium-230 concentrations in the South Dump and elevated uranium concentrations in the Ash Pond/South Dump area. The near-surface soil limit of 5 pCi/g for thorium-230 was exceeded in the South Dump to a maximum depth of 1.2 m (4 ft).

Uranium was detected above 60 pCi/g at maximum depths of 1 m (3 ft) in the South Dump and at greater than 1 m (3 ft) in Ash Pond. Uranium concentrations of 15 pCi/g were detected to a maximum depth of 1.2 m (4 ft) in the South Dump and to a maximum depth of greater than 1 m (3 ft) in Ash Pond. In addition, of 217 boreholes drilled at the site, samples from only two boreholes drilled in the area of the proposed isolation system had radium-226 concentrations above the near-surface soil guideline of 5 pCi/g. A sample from the borehole located east of Ash Pond had elevated radium concentrations to a depth of 0.8 m (2.5 ft), with a maximum of 5.6 pCi/g at a depth of 0.3 m (1 ft). A sample from the borehole located in the South Dump was contaminated to 1 m (3 ft) below the ground surface, with a maximum concentration of 37.5 pCi/g at a depth of 0.3 m (1 ft) (Marutzky et al. 1988).

For comparative purposes, 9 boreholes were drilled off-site to establish background concentrations of radionuclides. The sampling locations (A, B, C, and 1 through 6) are shown in Fig. 7, and the analytical results are summarized in Table 2.

THREAT TO PUBLIC HEALTH AND THE ENVIRONMENT

A potential health and environmental hazard exists at the Weldon Spring site due to high levels of uranium in the outflow from the Ash Pond area. The contamination poses a similar hazard off-site because at least a portion of this outflow, which enters the subsurface just west of the site boundary, surfaces again at Burgermeister Spring in the Busch Wildlife Area. Lake 35 in the wildlife area also receives surface water directly from Ash Pond (MK-Ferguson and Jacobs Engineering 1987). Contamination of Lake 35 and Burgermeister Spring poses a potential health hazard to area personnel, the general public, and resident wildlife.

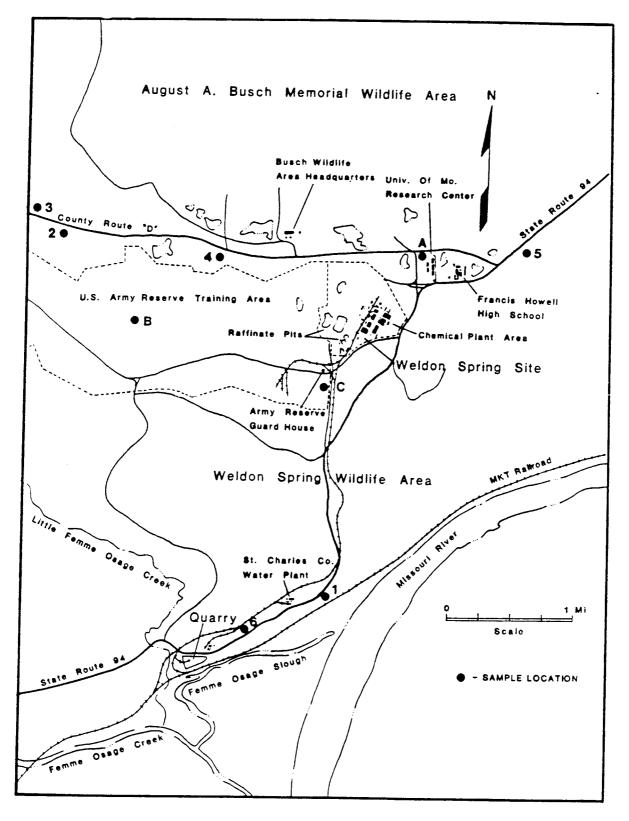


FIGURE 7 Off-site Radiological Sampling Locations (Source: MK-Ferguson and Jacobs Engineering 1987)

TABLE 2 Background Concentrations of Radionuclides in Surface Soil

000	Concentration (pCi/g)								
Off-site Location ^a	Radium-226	Thorium-232	Uranium-238						
1	0.8	0.9	< DLb						
2	1.1	0.9	< DL						
3	1.3	0.6	< DL						
4	0.8	0.8	< DL						
5	0.9	1.0	< DL						
6	1.1	1.0	< DL						
Α	0.9	0.7	< DL						
В	0.5	1.2	< DL						
С	1.2	0.4	< DL						

^aSamples from locations 1-6 were composited over 15 cm (6 in.); samples from locations A-C were composited over 1 m (3 ft).

Source: Data from MK-Ferguson and Jacobs Engineering (1987).

RESPONSE ACTION

Response Action Objectives

The objectives of the proposed response action are as follows:

- 1. Reduction of the potential on-site health hazard due to uranium contamination of surface water in the Ash Pond area;
- 2. Reduction of the potential off-site health hazard due to uranium contamination of receiving waters in the Busch Wildlife Area;
- 3. Reduction of the surface water infiltration rate through contaminated soils in the Ash Pond area; and
- 4. Improvement in the quality of water being discharged off-site from the Ash Pond area.

These objectives can be met by limiting surface water flow through the contaminated Ash Pond area by means of the proposed isolation system.

bDL = detection limit (about 1.9 pCi/g).

Proposed Response Action Alternatives

Interim (expedited) response actions are implemented to ensure the health and safety of on-site personnel and local populations and to minimize or preclude off-site releases of contamination. These actions are limited to those that can be performed under the Comprehensive Environmental Response, Compensation, and Liability Act, as amended by the Superfund Amendments and Reauthorization Act (SARA), and remain within the constraints of the Council on Environmental Quality's regulations for the National Environmental Policy Act (i.e., actions will be limited to those that do not have an adverse environmental impact nor limit the choice of reasonable alternatives).

The following alternatives have been identified for the proposed interim response action to reduce contamination of surface runoff from the Ash Pond area:

- 1. No action:
- 2. Excavation of contaminated material from the Ash Pond area, including the South Dump, which is responsible for radiological contamination of surface flow through the area, with on-site storage of all material that exceeds the radiological criteria for unrestricted release (and on-site interim storage of any material that exceeds limits for chemical contamination, if discovered, pending a disposal decision);
- Construction of a dike at the site boundary downstream of the Ash Pond area to provide a retention basin for the contaminated water until it can be decontaminated at an on-site water treatment plant; and
- 4. Construction of an isolation dike upstream of Ash Pond to prevent contact of surface runoff with contaminated material in the Ash Pond area (e.g., the South Dump) and construction of diversion channels to route the water away from these contaminated locations for subsequent outflow at its current off-site discharge point.

Screening and Analysis of Response Action Alternatives

The four alternatives that have been identified for the proposed action are screened and analyzed below on the basis of criteria identified in U.S. Environmental Protection Agency (EPA) guidance for removal actions. These criteria include technical feasibility, environmental impacts, cost, and institutional factors (e.g., timeliness, compliance with ARARs, and protectiveness of public health and welfare).

If no action were taken (Alternative 1), the potential health threat posed by uranium contamination of surface runoff from Ash Pond would not be reduced, nor would on-site or off-site environmental conditions be improved. Although Alternative 1 presents no technical barriers and costs nothing in the short term, it is effectively precluded by the potential for adverse environmental impacts and significant long-term

costs (e.g., for the cleanup of areas not currently contaminated but to which contaminants may migrate if no action is taken). It is also precluded by institutional factors related to the community's desire for timely response actions at the Weldon Spring site — in particular, for a reduction in the off-site release of contaminants.

The action alternatives (Alternatives 2 through 4) are technically feasible and would reduce the potential hazards associated with uranium contamination of surface runoff. Environmental conditions, both on-site and off-site, would be improved if any of these alternatives were implemented.

Alternative 2 is expected to be more expensive than Alternatives 3 and 4. The affected area would need to be protected from surface water intrusion during the excavation period, which would be reflected in costs for constructing an isolation system. In addition to these construction costs, which would be similar to those for Alternatives 3 and 4, Alternative 2 would incur costs associated with storage — i.e., for all material exceeding radiological release criteria and for chemically contaminated material, if encountered, pending a disposal decision. Thus, a material staging area would be required for Alternative 2; the plan for such a staging area is currently being addressed as a separate interim response action because of a separately identified need. The more extensive planning and documentation that would be required prior to the implementation of Alternative 2, because of its expanded scope as compared to Alternatives 3 and 4, would increase costs and delay the initiation of any mitigative action. Therefore, Alternative 2 would not satisfy institutional factors related to timeliness, i.e., the community's desire for expedited response with regard to minimizing off-site releases of radioactively contaminated water.

The excavation of contaminated material from the area of the proposed interim response action is not unique to Alternative 2; it is being addressed in remedial action plans for the Weldon Spring site and would occur subsequent to the implementation of either Alternative 3 or 4. The excavation would likely be included in the scope of the record of decision for remedial action at the Weldon Spring site. Thus, the selection of either Alternative 3 or Alternative 4 would preclude the need for interim storage of contaminated material because a decision on waste disposal would have been made by the time of excavation. An additional advantage of selecting Alternative 3 or Alternative 4 instead of Alternative 2 is the flexibility to initiate a timely response action at the Ash Pond area, without being tied to a decision that is within the broader scope of overall remedial action for the Weldon Spring site.

Although implementation of Alternative 3 would prevent surface water from leaving the Ash Pond area (i.e., by virtue of a downstream dike), it would do nothing to mitigate the contamination of this water (i.e., the contact of inflow with contaminated materials would continue). Thus, a water treatment plant would be required to treat the contaminated water prior to its release off-site. Costs associated with the construction and operation of a water treatment plant would make Alternative 3 more expensive than Alternative 4. In addition, institutional factors associated with public pressure to minimize off-site contaminant releases would not be completely addressed by Alternative 3. Ponding of water above areas of contaminated soil would increase the local hydraulic head, thereby increasing the potential for infiltration through these areas and the resultant transport of radionuclides into the groundwater. Finally, Alternative 3

would be unsatisfactory in terms of timeliness and other institutional factors related to construction of the water treatment plant. Because approval for this construction has not yet been addressed by the appropriate federal, state, or local agencies, considerable delays could occur prior to construction of the treatment plant.

In contrast to Alternative 3, Alternative 4 would involve diversion of surface runoff away from contaminated areas in the watershed. Not only would this preclude the contamination of surface runoff resulting from contact with these areas and obviate the need for a water treatment plant, it would also effectively reduce the hydraulic head at Ash Pond, thereby decreasing the potential for contaminant transport into the groundwater. Alternative 4 could be implemented in a timely and cost-effective manner and would be protective of the public and the environment by limiting the off-site release of contaminants.

As a result of the screening and analysis process for interim response action alternatives, Alternative 4 has been identified as the preferred alternative. Alternative 4 is consistent with and will contribute to the efficient performance of remedial action being planned for the Weldon Spring site.

Description of the Proposed Response Action

Implementation of the proposed interim response action to construct an upstream dike and diversion channels would result in restricting the flow of surface water across the contaminated areas of the Ash Pond watershed. The response action would include the following operations:

- Construction of an isolation dike upstream of Ash Pond -- measuring approximately 230 m (750 ft) in length and 3 m (10 ft) at its maximum height, containing about 5,400 m³ (7,000 yd³) of uncontaminated soil material, and creating a retention pond covering a maximum of 0.6 ha (1.5 acres) when full;
- 2. Construction of diversion channels totaling approximately 610 m (2,500 ft) in length and measuring about 1 m (3 ft) in height, which would circumvent the Ash Pond area and connect the dike to the current point of surface water discharge off-site; and
- Maintenance of the discharge monitoring station currently in place for intermittent measurement of water quality and continuous measurement of the quantity of surface water discharged from the Ash Pond area.

The proposed action would be conducted in accordance with all applicable or relevant and appropriate requirements (ARARs), to ensure protection of the safety and health of on-site workers and local populations and to limit off-site releases of contaminants. Section 121(d)(4) of SARA identifies six conditions under which a waiver from compliance with ARARs may be granted. One of these conditions is that the action

is only part of a total remedial action that will attain such levels or standards of control as identified by the specific ARAR when the total remedial action is completed. If it is determined that a waiver application is necessary, e.g., for uranium discharge limits, this condition is applicable to the proposed interim response action because isolation of the Ash Pond area is by definition an interim measure to minimize the off-site migration of contaminants. It is also important to note that, because the proposed action is an interim measure, the effected reduction in the uranium discharge level is not to be interpreted as an accepted discharge limit for the remedial action project at the Weldon Spring site. Instead, this level is specific to the response action and is dictated by the conditions of that intermediate action, the purpose of which is to improve near-term environmental and safety conditions in the Ash Pond area. The DOE will establish project-specific discharge limits and cleanup criteria for the Weldon Spring site in cooperation with the EPA and the Missouri Department of Natural Resources.

Borrow material for construction of the Ash Pond isolation dike and diversion channels would be obtained from a nearby spoils pile located outside the affected area. Results of characterization studies have indicated that this spoils pile poses no chemical hazard and is not radiologically contaminated.

This interim response action would be taken to reduce the concentration of uranium in water leaving the Ash Pond watershed. It is expected that the uranium concentration would be reduced from as high as 4,000 pCi/L to less than 400 pCi/L, which is below the current DOE uranium-238 limit of 600 pCi/L for release to uncontrolled areas (U.S. Department of Energy 1986). The isolated areas responsible for this contamination (i.e., locations in the Ash Pond area, including the South Dump) would be remediated in the future. Implementation of the proposed response action at this time would minimize the potential adverse impacts on health and the environment resulting from continued runoff of highly contaminated surface water from the watershed and would support the long-term response to contaminated conditions in the Ash Pond area.

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APPENDIX A

DOE GUIDELINES FOR RESIDUAL RADIOACTIVE MATERIAL

U.S. DEPARTMENT OF ENERGY GUIDELINES
FOR RESIDUAL RADIOACTIVE MATERIAL AT
FORMERLY UTILIZED SITES REMEDIAL ACTION PROGRAM
AND
REMOTE SURPLUS FACILITIES MANAGEMENT PROGRAM SITES

(Revision 2, March 1987)

A. INTRODUCTION

This document presents U.S. Department of Energy (DOE) radiological protection guidelines for cleanup of residual radioactive material and management of the resulting wastes and residues. It is applicable to sites identified by the Formerly Utilized Sites Remedial Action Program (FUSRAP) and remote sites identified by the Surplus Facilities Management Program (SFMP).* The topics covered are basic dose limits, guidelines and authorized limits for allowable levels of residual radioactive material, and requirements for control of the radioactive wastes and residues.

Protocols for identification, characterization, and designation of FUSRAP sites for remedial action; for implementation of the remedial action; and for certification of a FUSRAP site for release for unrestricted use are given in a separate document (U.S. Department of Energy 1986) and subsequent guidance. More detailed information on applications of the guidelines presented herein, including procedures for deriving site-specific guidelines for allowable levels of residual radioactive material from basic dose limits, is contained in "A Manual for Implementing Residual Radioactive Material Guidelines" (U.S. Department of Energy 1987), referred to herein as the "supplement".

"Residual radioactive material" is used in these guidelines to describe radioactive material derived from operations or sites over which DOE has authority. Guidelines or guidance to limit the levels of radioactive material and to protect the public and the environment are provided for (1) residual concentrations of radionuclides in soil,** (2) concentrations of airborne

^{*}A remote SFMP site is one that is excess to DOE programmatic needs and is located outside a major operating DOE research and development or production area.

^{**&}quot;Soil" is defined herein as unconsolidated earth material, including rubble and debris that may be present in earth material.

radon decay products, (3) external gamma radiation levels, (4) surface contamination levels, and (5) radionuclide concentrations in air or water resulting from or associated with any of the above.

A "basic dose limit" is a prescribed standard from which limits for quantities that can be monitored and controlled are derived; it is specified in terms of the effective dose equivalent as defined by the International Commission on Radiological Protection (ICRP 1977, 1978). The basic dose limits are used for deriving guidelines for residual concentrations of radionuclides in soil. Guidelines for residual concentrations of thorium and radium in soil, concentrations of airborne radon decay products, allowable indoor external gamma radiation levels, and residual surface contamination concentrations are based on existing radiological protection standards (U.S. Environmental Protection Agency 1983; U.S. Nuclear Regulatory Commission 1982; and DOE Departmental Orders). Derived guidelines or limits based on the basic dose limits for those quantities are used only when the guidelines provided in the existing standards cited above are shown to be inappropriate.

A "guideline" for residual radioactive material is a level of radioactivity or radioactive material that is acceptable if use of the site is to
be unrestricted. Guidelines for residual radioactive material presented
herein are of two kinds: (1) generic, site-independent guidelines taken from
existing radiation protection standards and (2) site-specific guidelines
derived from basic dose limits using site-specific models and data. Generic
guideline values are presented in this document. Procedures and data for
deriving site-specific guideline values are given in the supplement. The
basis for the guidelines is generally a presumed worst-case plausible-use
scenario for the site.

An "authorized limit" is a level of residual radioactive material or radioactivity that must not be exceeded if the remedial action is to be considered completed and the site is to be released for unrestricted use. The authorized limits for a site will include (1) limits for each radionuclide or group of radionuclides, as appropriate, associated with residual radioactive material in soil or in surface contamination of structures and equipment, (2) limits for each radionuclide or group of radionuclides, as appropriate, in air or water, and, (3) where appropriate, a limit on external gamma radiation resulting from the residual material. Under normal circumstances, expected to occur at most sites, authorized limits for residual radioactive material or radioactivity are set equal to guideline values. Exceptional conditions for which authorized limits might differ from guideline values are specified in Sections D and F of this document. A site may be released for unrestricted use only if site conditions do not exceed the authorized limits or approved supplemental limits, as defined in Section F.1, at the time remedial action is completed. Restrictions and controls on use of the site must be established and enforced if site conditions exceed the approved limits, or if there is potential to exceed the basic dose limit if use of the site is not restricted The applicable controls and restrictions are specified in (Section F.2). Section E.

DOE policy requires that all exposures to radiation be limited to levels that are as low as reasonably achievable (ALARA). For sites to be released for unrestricted use, the intent is to reduce residual radioactive material to levels that are as far below authorized limits as reasonable considering technical, economic, and social factors. At sites where the residual material is not reduced to levels that permit release for unrestricted use, ALARA policy is implemented by establishing controls to reduce exposure to levels that are as low as reasonably achievable. Procedures for implementing ALARA policy are discussed in the supplement. ALARA policies, procedures, and actions shall be documented and filed as a permanent record upon completion of remedial action at a site.

B. BASIC DOSE LIMITS

The basic limit for the annual radiation dose received by an individual member of the general public is 100 mrem/yr. The internal committed effective dose equivalent, as defined in ICRP Publication 26 (ICRP 1977) and calculated by dosimetry models described in ICRP Publication 30 (ICRP 1978), plus the dose from penetrating radiation sources external to the body, shall be used for determining the dose. This dose shall be described as the "effective dose equivalent". Every effort shall be made to ensure that actual doses to the public are as far below the basic dose limit as is reasonably achievable.

Under unusual circumstances, it will be permissible to allow potential doses to exceed 100 mrem/yr where such exposures are based upon scenarios that do not persist for long periods and where the annual lifetime exposure to an individual from the subject residual radioactive material would be expected to be less than 100 mrem/yr. Examples of such situations include conditions that might exist at a site scheduled for remediation in the near future or a possible, but improbable, one-time scenario that might occur following remedial action. These levels should represent doses that are as low as reasonably achievable for the site. Further, no annual exposure should exceed 500 mrem.

C. GUIDELINES FOR RESIDUAL RADIOACTIVE MATERIAL

C.1 Residual Radionuclides in Soil

Residual concentrations of radionuclides in soil shall be specified as above-background concentrations averaged over an area of 100 m^2 . Generic guidelines for thorium and radium are specified below. Guidelines for residual concentrations of other radionuclides shall be derived from the basic dose limits by means of an environmental pathway analysis using site-specific data where available. Procedures for these derivations are given in the supplement.

If the average concentration in any surface or below-surface area less than or equal to 25 m^2 exceeds the authorized limit or guideline by a factor of $(100/A)^{1/2}$, where A is the area of the elevated region in square meters,

limits for "hot spots" shall also be applicable. Procedures for calculating these hot spot limits, which depend on the extent of the elevated local concentrations, are given in the supplement. In addition, every reasonable effort shall be made to remove any source of radionuclide that exceeds 30 times the appropriate limit for soil, irrespective of the average concentration in the soil.

Two types of guidelines are provided, generic and derived. The generic guidelines for residual concentrations of Ra-226, Ra-228, Th-230, and Th-232 are:

- 5 pCi/g, averaged over the first 15 cm of soil below the surface
- 15 pCi/g, averaged over 15-cm-thick layers of soil more than 15 cm below the surface

These guidelines take into account ingrowth of Ra-226 from Th-230 and of Ra-228 from Th-232, and assume secular equilibrium. If either Th-230 and Ra-226 or Th-232 and Ra-228 are both present, not in secular equilibrium, the appropriate guideline is applied as a limit to the radionuclide with the higher concentration. If other mixtures of radionuclides occur, the concentrations of individual radionuclides shall be reduced so that (1) the dose for the mixtures will not exceed the basic dose limit or (2) the sum of the ratios of the soil concentration of each radionuclide to the allowable limit for that radionuclide will not exceed 1 ("unity"). Explicit formulas for calculating residual concentration guidelines for mixtures are given in the supplement.

C.2 Airborne Radon Decay Products

Generic guidelines for concentrations of airborne radon decay products shall apply to existing occupied or habitable structures on private property that are intended for unrestricted use; structures that will be demolished or buried are excluded. The applicable generic guideline (40 CFR Part 192) is: In any occupied or habitable building, the objective of remedial action shall be, and a reasonable effort shall be made to achieve, an annual average (or equivalent) radon decay product concentration (including background) not to exceed 0.02 WL.* In any case, the radon decay product concentration (including background) shall not exceed 0.03 WL. Remedial actions by DOE are not required in order to comply with this guideline when there is reasonable assurance that residual radioactive material is not the cause.

^{*}A working level (WL) is any combination of short-lived radon decay products in one liter of air that will result in the ultimate emission of 1.3×10^5 MeV of potential alpha energy.

C.3 External Gamma Radiation

The average level of gamma radiation inside a building or habitable structure on a site to be released for unrestricted use shall not exceed the background level by more than 20 $\mu R/h$ and shall comply with the basic dose limit when an appropriate-use scenario is considered. This requirement shall not necessarily apply to structures scheduled for demolition or to buried foundations. External gamma radiation levels on open lands shall also comply with the basic dose limit, considering an appropriate-use scenario for the area.

C.4 Surface Contamination

The generic surface contamination guidelines provided in Table 1 are applicable to existing structures and equipment. These guidelines are adapted from standards of the U.S. Nuclear Regulatory Commission (NRC 1982)* and will be applied in a manner that provides a level of protection consistent with the Commission's guidance. These limits apply to both interior and exterior surfaces. They are not directly intended for use on structures to be demolished or buried, but should be applied to equipment or building components that are potentially salvageable or recoverable scrap. If a building is demolished, the guidelines in Section C.1 are applicable to the resulting contamination in the ground.

C.5 Residual Radionuclides in Air and Water

Residual concentrations of radionuclides in air and water shall be controlled to levels required by DOE Environmental Protection Guidance and Orders, specifically DOE Order 5480.1A and subsequent guidance. Other Federal and/or state standards shall apply when they are determined to be appropriate.

D. AUTHORIZED LIMITS FOR RESIDUAL RADIOACTIVE MATERIAL

Authorized limits shall be established to (1) ensure that, as a minimum, the basic dose limits specified in Section B will not be exceeded under the worst-case plausible-use scenario consistent with the procedures and guidance provided or (2) be consistent with applicable generic guidelines, where such guidelines are provided. The authorized limits for each site and its vicinity properties shall be set equal to the generic or derived guidelines except where it can be clearly established on the basis of site-specific data—including health, safety, and socioeconomic considerations— that the guidelines are not appropriate for use at the specific site. Consideration should also be given to ensure that the limits comply with or provide a level of protection equivalent to other appropriate limits and guidelines (i.e., state or

^{*}These guidelines are functionally equivalent to Section 4 -- Decontamination for Release for Unrestricted Use -- of NRC Regulatory Guide 1.86 (U.S. Atomic Energy Commission 1974), but they are applicable to non-reactor facilities.

TABLE 1 SURFACE CONTAMINATION GUIDELINES

	Allowable Total Residual Surface Contamination (dpm/100 cm ²) ^a							
Radionuclides ^b	Average ^{c,d}	Maximum ^d ,e	e Removable ^{d, f}					
Transuranics, Ra-226, Ra-228, Th-230, Th-228, Pa-231, Ac-227, I-125, I-129	100	300	20					
Th-Natural, Th-232, Sr-90, Ra-223, Ra-224, U-232, I-126, I-131, I-133	1,000	3,000	200					
U-Natural, U-235, U-238, and associated decay products	5,000 a	15,000 α	1,000 α					
Beta-gamma emitters (radionuclides with decay modes other than alpha emission or spontaneous fission)	5 000 0	15 000 0	1 000 0					
except Sr-90 and others noted above	5,000 β-γ	15,000 β-γ	1,000 β-γ					

As used in this table, dpm (disintegrations per minute) means the rate of emission by radioactive material as determined by correcting the counts per minute measured by an appropriate detector for background, efficiency, and geometric factors associated with the instrumentation.

b Where surface contamination by both alpha- and beta-gamma-emitting radionuclides exists, the limits established for alpha- and beta-gamma-emitting radionuclides should apply independently.

^c Measurements of average contamination should not be averaged over an area of more than 1 m². For objects of less surface area, the average should be derived for each such object.

d The average and maximum dose rates associated with surface contamination resulting from beta-gamma emitters should not exceed 0.2 mrad/h and 1.0 mrad/h, respectively, at 1 cm.

^e The maximum contamination level applies to an area of not more than 100 cm².

f The amount of removable radioactive material per 100 cm² of surface area should be determined by wiping that area with dry filter or soft absorbent paper, applying moderate pressure, and measuring the amount of radioactive material on the wipe with an appropriate instrument of known efficiency. When removable contamination on objects of surface area less than 100 cm² is determined, the activity per unit area should be based on the actual area and the entire surface should be wiped. The numbers in this column are maximum amounts.

other Federal). Documentation supporting such a decision should be similar to that required for supplemental limits and exceptions (Section F), but should be generally more detailed because the documentation covers the entire site.

Remedial action shall not be considered complete unless the residual radioactive material levels comply with the authorized limits. The only exception to this requirement will be for those special situations where the supplemental limits or exceptions are applicable and approved as specified in Section F. However, the use of supplemental limits and exceptions should be considered only if it is clearly demonstrated that it is not reasonable to decontaminate the area to the authorized limit or guideline value. The authorized limits are developed through the project offices in the field and are approved by the headquarters program office.

E. CONTROL OF RESIDUAL RADIOACTIVE MATERIAL AT FUSRAP AND REMOTE SFMP SITES

Residual radioactive material above the guidelines at FUSRAP and remote SFMP sites must be managed in accordance with applicable DOE Orders. The DOE Order 5480.1A and subsequent guidance or superceding Orders require compliance with applicable Federal and state environmental protection standards.

The operational and control requirements specified in the following DOE Orders shall apply to interim storage, interim management, and long-term management.

- a. 5000.3, Unusual Occurrence Reporting System
- b. 5440.1C, Implementation of the National Environmental Policy Act
- c. 5480.1A, Environmental Protection, Safety, and Health Protection Program for DOE Operations, as revised by DOE 5480.1 change orders and the 5 August 1985 memorandum from Vaughan to Distribution
- d. 5480.2, Hazardous and Radioactive Mixed Waste Management
- e. 5480.4, Environmental Protection, Safety, and Health Protection Standards
- f. 5482.1A, Environmental, Safety, and Health Appraisal Program
- g. 5483.1A, Occupational Safety and Health Program for Government-Owned Contractor-Operated Facilities
- h. 5484.1, Environmental Protection, Safety, and Health Protection Information Reporting Requirements
- i. 5820.2, Radioactive Waste Management

E.1 Interim Storage

- a. Control and stabilization features shall be designed to ensure, to the extent reasonably achievable, an effective life of 50 years and, in any case, at least 25 years.
- b. Above-background Rn-222 concentrations in the atmosphere above facility surfaces or openings shall not exceed (1) 100 pCi/L at any given point, (2) an annual average concentration of 30 pCi/L over the facility site, and (3) an annual average concentration of 3 pCi/L at or above any location outside the facility site (DOE Order 5480.1A, Attachment XI-1).
- c. Concentrations of radionuclides in the groundwater or quantities of residual radioactive material shall not exceed existing Federal or state standards.
- d. Access to a site shall be controlled and misuse of on-site material contaminated by residual radioactive material shall be prevented through appropriate administrative controls and physical barriers active and passive controls as described by the U.S. Environmental Protection Agency (1983—p. 595). These control features should be designed to ensure, to the extent reasonable, an effective life of at least 25 years. The Federal government shall have title to the property or shall have a long-term lease for exclusive use.

E.2 Interim Management

- a. A site may be released under interim management when the residual radioactive material exceeds guideline values if the residual radioactive material is in inaccessible locations and would be unreasonably costly to remove, provided that administrative controls are established to ensure that no member of the public shall receive a radiation dose exceeding the basic dose limit.
- b. The administrative controls, as approved by DOE, shall include but not be limited to periodic monitoring as appropriate, appropriate shielding, physical barriers to prevent access, and appropriate radiological safety measures during maintenance, renovation, demolition, or other activities that might disturb the residual radioactive material or cause it to migrate.
- c. The owner of the site or appropriate Federal, state, or local authorities shall be responsible for enforcing the administrative controls.

E.3 Long-Term Management

Uranium, Thorium, and Their Decay Products

- a. Control and stabilization features shall be designed to ensure, to the extent reasonably achievable, an effective life of 1,000 years and, in any case, at least 200 years.
- b. Control and stabilization features shall be designed to ensure that Rn-222 emanation to the atmosphere from the wastes shall not (1) exceed an annual average release rate of 20 pCi/m²/s and (2) increase the annual average Rn-222 concentration at or above any location outside the boundary of the contaminated area by more than 0.5 pCi/L. Field verification of emanation rates is not required.
- c. Prior to placement of any potentially biodegradable contaminated wastes in a long-term management facility, such wastes shall be properly conditioned to ensure that (1) the generation and escape of biogenic gases will not cause the requirement in paragraph b. of this section (E.3) to be exceeded and (2) biodegradation within the facility will not result in premature structural failure in violation of the requirements in paragraph a. of this section (E.3).
- d. Groundwater shall be protected in accordance with appropriate Departmental Orders and Federal and state standards, as applicable to FUSRAP and remote SFMP sites.
- e. Access to a site should be controlled and misuse of on-site material contaminated by residual radioactivity should be prevented through appropriate administrative controls and physical barriers active and passive controls as described by the U.S. Environmental Protection Agency (1983—p. 595). These controls should be designed to be effective to the extent reasonable for at least 200 years. The Federal government shall have title to the property.

Other Radionuclides

f. Long-term management of other radionuclides shall be in accordance with Chapters 2, 3, and 5 of DOE Order 5820.2, as applicable.

F. SUPPLEMENTAL LIMITS AND EXCEPTIONS

If special site-specific circumstances indicate that the guidelines or authorized limits established for a given site are not appropriate for a portion of that site or for a vicinity property, then the field office may request that supplemental limits or an exception be applied. In either case, the field office must justify that the subject guidelines or authorized limits are not appropriate and that the alternative action will provide adequate

protection, giving due consideration to health and safety, the environment, and costs. The field office shall obtain approval for specific supplemental limits or exceptions from headquarters as specified in Section D of these guidelines and shall provide to headquarters those materials required for the justification as specified in this section (F) and in the FUSRAP and SFMP protocols and subsequent guidance documents. The field office shall also be responsible for coordination with the state or local government of the limits or exceptions and associated restrictions as appropriate. In the case of exceptions, the field office shall also work with the state and/or local governments to ensure that restrictions or conditions of release are adequate and mechanisms are in place for their enforcement.

F.1 Supplemental Limits

The supplemental limits must achieve the basic dose limits set forth in this guideline document for both current and potential unrestricted uses of a site and/or vicinity property. Supplemental limits may be applied to a vicinity property or a portion of a site if, on the basis of a site-specific analysis, it is determined that (1) certain aspects of the vicinity property or portion of the site were not considered in the development of the established authorized limits and associated guidelines for that vicinity property or site and, (2) as a result of these unique characteristics, the established limits or guidelines either do not provide adequate protection or are unnecessarily restrictive and costly.

F.2 Exceptions

Exceptions to the authorized limits defined for unrestricted use of a site or vicinity property may be applied to a vicinity property or a portion of a site when it is established that the authorized limits cannot be achieved and restrictions on use of the vicinity property or portion of the site are necessary to provide adequate protection of the public and the environment. The field office must clearly demonstrate that the exception is necessary and that the restrictions will provide the necessary degree of protection and will comply with the requirements for control of residual radioactive material as set forth in Section E of these guidelines.

F.3 Justification for Supplemental Limits and Exceptions

Supplemental limits and exceptions must be justified by the field office on a case-by-case basis using site-specific data. Every effort should be made to minimize use of the supplemental limits and exceptions. Examples of specific situations that warrant use of the supplemental standards and exceptions are:

a. Where remedial action would pose a clear and present risk of injury to workers or members of the general public, notwithstanding reasonable measures to avoid or reduce risk.

- b. Where remedial action -- even after all reasonable mitigative measures have been taken -- would produce environmental harm that is clearly excessive compared to the health benefits to persons living on or near affected sites, now or in the future. A clear excess of environmental harm is harm that is long-term, manifest, and grossly disproportionate to health benefits that may reasonably be anticipated.
- c. Where it is clear that the scenarios or assumptions used to establish the authorized limits do not, under plausible current or future conditions, apply to the property or portion of the site identified and where more appropriate scenarios or assumptions indicate that other limits are applicable or necessary for protection of the public and the environment.
- d. Where the cost of remedial action for contaminated soil is unreasonably high relative to long-term benefits and where the residual radioactive material does not pose a clear present or future risk after taking necessary control measures. The likelihood that buildings will be erected or that people will spend long periods of time at such a site should be considered in evaluating this risk. Remedial action will generally not be necessary where only minor quantities of residual radioactive material are involved or where residual radioactive material occurs in an inaccessible location at which site-specific factors limit their hazard and from which they are costly or difficult to remove. Examples include residual radioactive material under hard-surface public roads and sidewalks, around public sewer lines, or in fence-post foundations. A site-specific analysis must be provided to establish that it would not cause an individual to receive a radiation dose in excess of the basic dose limits stated in Section B, and a statement specifying the level of residual radioactive material must be included in the appropriate state and local records.
- e. Where there is no feasible remedial action.

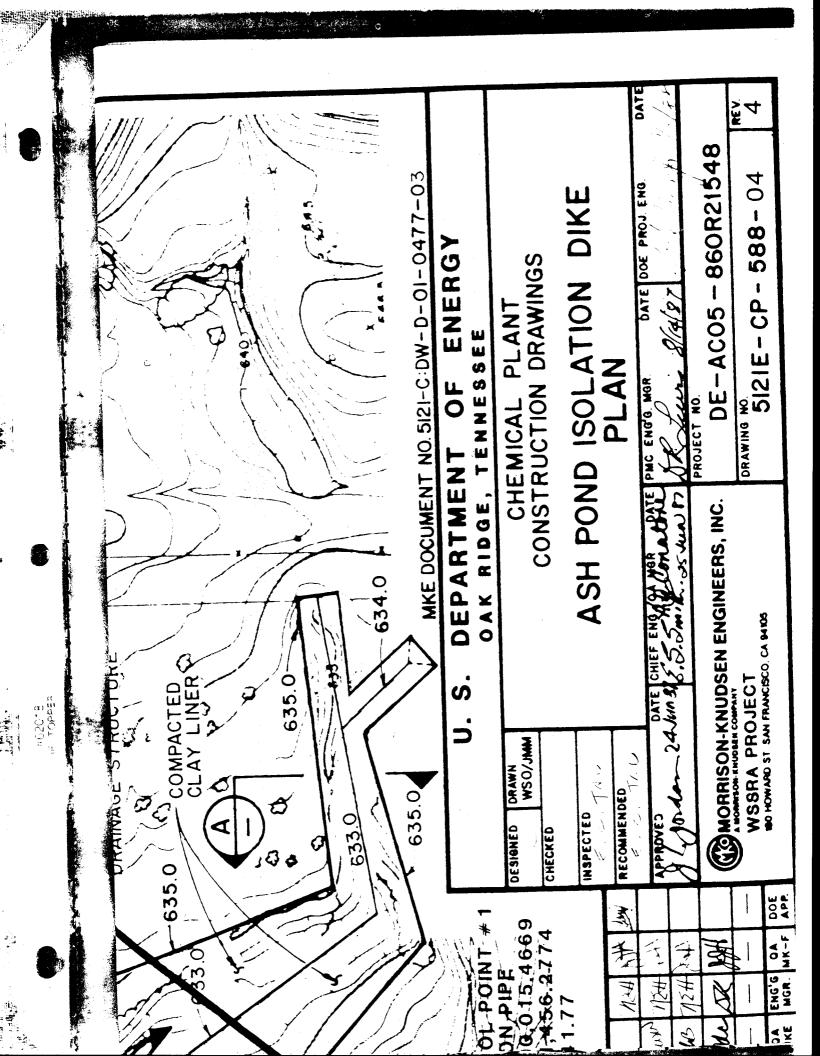
G. SOURCES

Limit or Guideline Source Basic Dose Limits Dosimetry model and dose limits International Commission on Radiological Protection (1977, 1978) Generic Guidelines for Residual Radioactivity Residual concentrations of radium 40 CFR Part 192 and thorium in soil Airborne radon decay products 40 CFR Part 192 40 CFR Part 192 External gamma radiation Surface contamination Adapted from U.S. Nuclear Regulatory Commission (1982) Control of Radioactive Wastes and Residues DOE Order 5480.1A and subsequent Interim storage guidance DOE Order 5480.1A and subsequent Long-term management guidance; 40 CFR Part 192;

DOE Order 5820.2

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CHEMICAL SOIL DATA REPORT TO SUPPORT INTERIM RESPONSE ACTIONS, CONSTRUCTION STAGING AREA, AND ADMINISTRATION BUILDING

For The:

Weldon Spring Site Remedial Action Project Weldon Spring, Missouri

Prepared By MK-Ferguson Company **FEBRUARY**, 1989

REV. 0



U.S. Department Of EnergyOak Ridge Operations OfficeWeldon Spring Site Remedial Action Project

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CHEMICAL SOIL DATA REPORT TO SUPPORT INTERIM RESPONSE ACTIONS, CONSTRUCTION STAGING AREA AND ADMINISTRATION BUILDING

FEBRUARY 1989

Prepared for:
U.S. DEPARTMENT OF ENERGY
Oak Ridge Operations Office
Under Contract No. DE-AC05-860R21548

bу

MK-FERGUSON COMPANY
Project Management Contractor
and

JACOBS ENGINEERING GROUP INC.
Route 2, Highway 94 South
St. Charles, Missouri 63303

ABSTRACT

Five activities are planned to improve environmental conditions or to improve facilities at the Weldon Spring Site. Each activity must be evaluated for potential environmental impacts. Chemical soil contamination was potentially present in each affected area. A sampling program was designed and implemented to evaluate chemical soil conditions. Samples were analyzed for nitroaromatic compounds, metals, inorganic anions, semi-volatile and volatile organic compounds, pesticides, and PCBs.

This investigation documented low concentrations of semi-volatile organic compounds, pesticides, PCBs and nitroaromatics. Higher concentrations of nitrate, sulfate and some metals were also detected.

The contaminants detected are consistent with past operations at the WSS. The concentrations of contaminants do not significantly impact the proposed activities. Data from this investigation has been incorporated into the planning and documentation activities for each activity.

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1 INTRODUCTION

This report addresses three areas proposed for interim remedial action (IRA) and two areas where construction is planned prior to completion of chemical soil characterization. The three IRA areas are the Ash Pond Isolation Dike (AID), the Southeast Drainage Isolation Dike (SID), and the Material Staging Area (MSA). The two construction areas are the locations of the Administration Building (AB) and the construction staging area (CSA). These areas are shown in Figure 1.

The soils in these five areas were sampled in support of the design of the IRAs, to validate previous sampling results, and to evaluate the environmental impact of the IRAs. This report summarizes the analytical data from these samples.

This soil characterization effort was required before the overall chemical soil characterization could be performed. The overall chemical soil characterization program is described fully in the chemical soil characterization sampling plan (MK-F, 1988a). The overall soil sampling rationale is presented in that sampling plan and should be reviewed before attempting to further interpret the analytical data presented in this report.

The three IRA areas were identified during previous investigations as areas which could benefit from small actions not biasing an overall Record of Decision (ROD) on the disposition of the majority of the wastes on site. These IRAs support the overall Weldon Spring Site Remedial Action Program (WSSRAP) and will maintain exposure as low as reasonably achievable. These actions consist of diverting and isolating surface drainage in two areas and constructing a contaminated materials storage area.

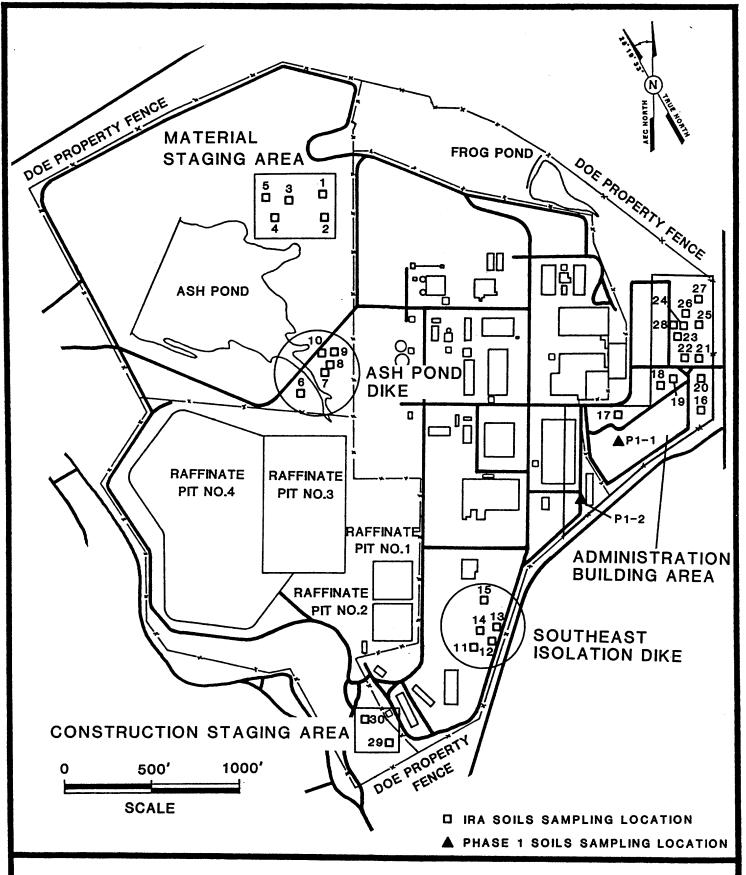


FIGURE 1

WELDON SPRING IRA SOIL SAMPLING LOCATIONS

1.1 PURPOSE

The purpose of the soil sampling in each of the areas was to provide data for the design of the IRA and construction activities. Sampling analyses provided information on chemical soil contamination in the areas affected by the isolation and drainage dikes and on the storage area for holding contaminated materials. The results also delineate conditions in the vicinity of the proposed Administration Building and the Construction Staging Area.

1.2 SCOPE

This program was designed to detect chemical soil contamination from the Weldon Spring Ordnance Works (WSOW) and the Weldon Spring Uranium Feed Materials Plant (WSUFMP) in the five areas. The WSOW produced explosives for use in World War II from 1941 to 1945. The WSUFMP processed uranium ore and produced uranium metal from 1956 to 1966. These two facilities comprise the known potential sources for chemical soil contamination.

Radiological contamination in each area is not discussed in this report. Radiological characterization data is presented in Radiological Characterization Reports for each IRA.

This sampling effort consisted of collecting 150 samples from 30 sample locations. Five borings were located in the Material Staging Area, five in the Ash Pond Isolation Dike area, and five in the Southeast Isolation Dike area. Thirteen borings were located around several proposed Administration Building sites, and two borings were located in the Construction Staging Area. Table 1 lists all boring locations, west-north coordinates and boring depths. The location numbers (1-30) correspond to the locations shown in Figure 1. Boring depths were determined by evaluating the depths that will be affected

TABLE 1
BORING LOCATIONS

LOCATION	IRA	COORDINATES	BORING DEPTH
NO.	AREA	WEST, NORTH	(FEET)
1	Material Staging Area	51150, 101207	10
2	Material Staging Area	51137, 101068	12
3	Material Staging Area	51360, 101175	10
4	Material Staging Area	51445, 101065	12
5	Material Staging Area	51500, 101190	12
6	Ash Pond Dike	51308, 100085	10
7	Ash Pond Dike	51150, 100220	10
8	Ash Pond Dike	51125, 100260	10
9	Ash Pond Dike	51100, 100335	10
10	Ash Pond Dike	51180, 100335	7
11	Southeast Isolation Dike	50290, 98700	8
12	Southeast Isolation Dike	50160, 98735	12
13	Southeast Isolation Dike	50140, 98820	8
14	Southeast Isolation Dike	50252, 98800	8
15	Southeast Isolation Dike	50230, 98991	8
16	Administration Building	49000, 99985	8
17	Administration Building	49475, 99985	12
18	Administration Building	49250, 100140	10
19	Administration Building	49172, 100180	8
20	Administration Building	49000, 100180	8
21	Administration Building	49000, 100295	8
22	Administration Building	49080, 100295	10
23	Administration Building	49132, 100440	16
24	Administration Building	49101, 100500	16
25	Administration Building	49000, 100500	12
26	Administration Building	49082, 100570	16
27	Administration Building	49000, 100665	12
28	Administration Building	49160, 100500	12
29	Construction Staging Area	50800, 98150	15
30	Construction Staging Area	50950, 98300	15
P1-1	Phase I - Admin. Building	49500, 99800	6
P1-2	Phase I - Admin. Building	49700, 99500	6

Pl - Phase I Chemical Soil Investigation Location.

by the specific construction activity and the depth of fill in each specific area.

Analytical parameters were selected on the basis of the results of Phase I chemical soil investigation (MK-F, 1988b) and Phase I water quality assessment (MK-F, 1987) which detected elevated concentrations of inorganic anions, metals, and nitroaromatics in several areas of the Weldon Spring Site (WSS). Certain locations were analyzed for Hazardous Substance List (HSL) volatiles, semi-volatiles, pesticides, and PCBs to provide additional information of the affected areas.

A brief description and the previous characterization data for each area is presented in the following subsections. The sampling and analysis methods are described in Section 2. Analytical data and interpretations are presented in Section 3.

1.3 MATERIAL STAGING AREA

The Material Staging Area (MSA) consists of approximately three acres located about 1,100 feet north of Raffinate Pit 3. This area subtends 500 feet by 250 feet and will be used to store contaminated materials from other IRAs, such as Debris Consolidation and Army Vicinity Property cleanup.

Previous investigations (MK-F, 1988b) in the MSA included adequate radiological soils characterization, but used only one borehole for chemical analyses. No chemical contamination was detected in this single borehole. Therefore, additional data were required to more fully characterize any chemical contamination which could be present in this area.

The additional chemical characterization for the MSA consisted of five boreholes drilled to depths of 10 ft to 12 ft with continuous sample collection. These depths exceed the depth of soil disturbance expected from this IRA. Samples were

composited over two-foot intervals in each borehole. All samples were analyzed for metals, inorganic anions (nitrate, sulfate, chloride, and fluoride), and nitroaromatic compounds. Certain locations were analyzed for pesticides, PCBs and semi-volatiles. After sampling, the boreholes were sealed by grouting with a cement-bentonite grout.

1.4 ASH POND ISOLATION DIKE

The Ash Pond Isolation Dike (AID) is proposed to divert surface runoff around the contaminated areas of the South Dump and Ash Pond by means of an earth embankment and drainage channel.

Previously collected information within the affected area consisted of adequate radiological soils characterization data, but only one borehole and three samples were analyzed for chemical parameters (MK-F, 1988b). These analyses indicated slightly elevated nitrate and sulfate levels in the soils. Chemical characterization data was required to evaluate the effects of ponding water on soils in the area affected by this IRA. These data will be used to further define the environmental impacts of the proposed IRA.

Additional characterization activities to support the AID IRA included drilling five boreholes ten feet deep with continuous sample collection. Samples were composited over two-foot intervals in each borehole and analyzed for metals, inorganic anions, and nitroaromatics. Also, certain locations were analyzed for pesticides, PCBs, and semi-volatiles. The boreholes were sealed by grouting with a cement-bentonite grout. The boreholes were located in potential borrow areas and at the former locations of Weldon Spring Ordnance Works (WSOW) buildings and wastewater lines.

1.5 SOUTHEAST ISOLATION DIKE

The third IRA requiring additional characterization is the Southeast Drainage Isolation Dike (SID). The scope of this IRA is similar to the AID IRA. No known structures or process lines from the WSOW were in this area.

Characterization requirements and activities for this IRA were also very similar, with five boreholes drilled. Samples were collected continuously and were analyzed for the same chemical parameters as other IRA locations.

1.6 ADMINISTRATION BUILDING AREA

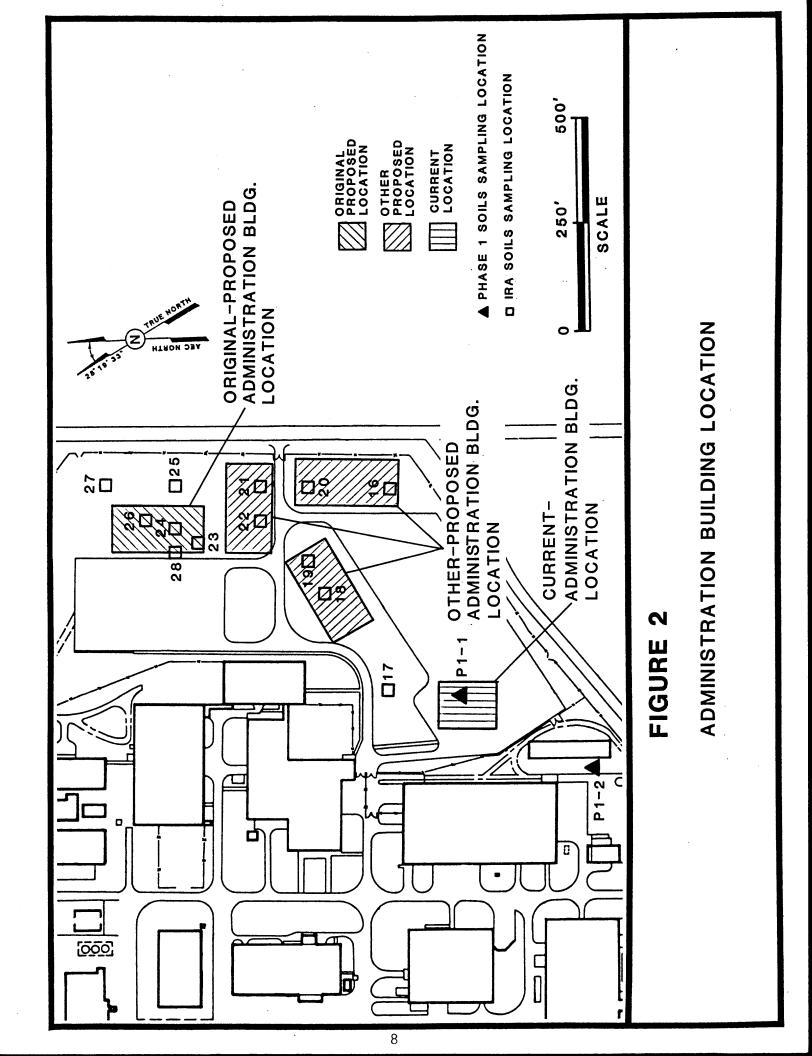
Four proposed sites for the Administration Building (ABA) were investigated to ensure the selection of an uncontaminated area for this building (Figure 2).

The chemical characterization activity for this area consisted of drilling 13 boreholes with continuous sample collection. The boreholes varied in depth from 10 ft to 16 feet, which exceeds the excavation depth for building construction. At each borehole location, the samples collected were composited over two-foot intervals. These samples were analyzed for the same parameters as the other IRAs.

One of the boreholes was located at the site of a WSOW Toluene Storage Area. Samples from this borehole were analyzed for volatile and semi-volatile compounds in addition to the other parameters.

1.7 CONSTRUCTION STAGING AREA

The Construction Staging Area (CSA) covers approximately one acre at the southwest corner of the WSCP/WSRP. This area was a candidate site for construction support facilities including



decontamination facilities for vehicles and personnel, a subcontractor trailer area, a guard shelter, and a control point for access to the controlled area.

The chemical characterization activities for this area consisted of drilling two boreholes to a depth of 15 feet. Two composite samples from each location were collected. The samples from the 0-ft to 7-ft interval were analyzed for metals, inorganic anions (nitrate, sulfate, chloride, and fluoride), nitroaromatic compounds, semi-volatiles, volatiles, PCB's and pesticides. The 8-ft to 15-ft interval samples were analyzed for inorganic anions only.

2 SAMPLING AND ANALYSES

2.1 SAMPLE COLLECTION

A total of 150 samples were collected from 30 sample locations using a truck-mounted Central Mine Equipment 55 (CME) drill rig employing a 6 5/8 inch 0.D. hollow stem auger for drilling and the CME continuous sampler system. All samples, except those located in the Construction Staging Area, were composited over two-foot intervals to optimize analytical costs and achieve representative samples.

2.2 EQUIPMENT DECONTAMINATION

Soil sampling equipment was cleaned using a decontamination procedure designed to protect against cross contamination by nitroaromatic compounds and other chemical species. All augers, drill rods, and continuous samplers were washed using a high-pressure hot water washer. Augers and drill rods were cleaned between boreholes while continuous samplers were decontaminated between samples.

After washing, the continuous samplers were allowed to air dry. Then they were rinsed with toluene, followed by rinsing with acetone and hexane. The toluene rinse was used to dissolve any nitroaromatic residues. Acetone and hexane rinses were employed to remove toluene and other contaminants not removed by the hot water wash. The continuous samplers were allowed to air dry again prior to being reassembled. All rinsing solvents were collected. Stainless steel spatulas and pans were washed with distilled water, then rinsed with the same solvent sequence as used on the continuous samplers. This procedure was performed between every sample.

Field personnel wore new disposable vinyl gloves when collecting soil samples. Gloves were changed after decontaminating sampling equipment.

2.3 SAMPLE HANDLING AND PRESERVATION

The filled continuous samplers were opened in a shaded area to prevent photolysis of nitroaromatic compounds. Samples were collected from the continuous core using decontaminated stainless steel spatulas and pans. No chemical preservation was required during sample collection. The collected samples were placed in a cooler with blue ice. All samples were chilled immediately following sample collection and kept chilled throughout sample collection and shipment. All field samples were sent to the analytical laboratory in accordance with WSSRAP chain-of-custody standard operating procedures.

2.4 SAMPLE ANALYSES

Sample analyses were performed according to applicable EPA CLP protocols for metals, organics, pesticides, and PCBs. EPA method 300.0 was used for nitrate, sulfate, chloride, and fluoride analyses. EPA method 106.1 was used for pH analyses. Nitroaromatic compound analyses were performed following USATHAMA - HPLC methodology. Analytical parameters were selected on the basis of known or suspected contaminants from WSOW and/or WSUFMP processes. Samples were analyzed by metaTRACE, Inc. of Earth City, Missouri.

2.5 SAMPLE DESCRIPTION

A soil description for each sample was recorded in the field during sample collection. The soil samples from the Material Staging Area consisted primarily of gray-brown mottled clay. The Ash Pond Isolation Dike area soil was mostly rusty-red cherty clay. The Southeast Isolation Dike soil was

more varied with gray brown mottled clay, orange-gray mottled clay with sandy layers, buff dense gray silty clay, and orange-gray mottled clay with chert. The soil in the Administration Building Area consisted mostly of brown-gray mottled clay with some red-gray mottled clay. The soil in the Construction Staging Area was mostly brown-gray mottled clay with small amounts of red-gray mottled clay.

The soil sample descriptions in Appendix A address each sample interval in detail. These soils are typical of WSCP/WSRP soils which consist of the following units: Ferrelview Formation, clay till and basal till. A more detailed description of WSCP/WSRP soils is provided in the chemical soil investigation sampling plan (MK-F, 1988a).

3 DATA SUMMARY

This section presents a summary of the results of the chemical soil analyses. The detailed results of the inorganic and metals analyses are presented in Appendix B. Only those volatile, semi-volatile, PCB, and pesticide results which were above the detection limits are discussed in the following subsections. The detection limits achieved during these analyses are presented in Appendix C. These detection limits are in agreement with those required in the EPA Contract Laboratory Program (CLP).

As part of the Phase I chemical soils investigation (MK-F, 1988b), background metal concentrations across the WSS were analyzed. The results of this analysis are reproduced in Table 2. These background ranges should be considered when interpreting the concentration levels in samples taken from the five IRA and construction areas discussed in this report.

Aluminum, calcium, and magnesium are common in the clay soils present at the WSCP/WSRP. Clay particles are composed primarily of aluminum, calcium, and magnesium silicates with iron, potassium, and sodium ions readily interchangeable into the particle structure. In addition, elevated manganese levels are attributable to the presence of pyrolusite (MnO₂) in the soil.

Completion of this soil characterization program provided a large data set of inorganic anion results. The inorganic anion concentrations from this investigation and from the Phase I chemical soil characterization (MK-F, 1988b) were evaluated statistically to determine average background concentrations and ranges. Frequency histograms were plotted for each inorganic anion. These histograms were evaluated to detect concentrations outside the normal background distribution of inorganic anions. Elevated concentrations were removed from the data set prior to

TABLE 2

Statistical Data for Background Metal
Concentrations in Soils at the MSCP/MSRP

Compound	Sample	Arithmetic	Geometric	Arithmetic Standard		ackground nges	
=======================================	Size	Mean mg/Kg	Mean mg/Kg	Deviation mg/Kg	Low mg/Kg	High mg/Kg	
A1	142	12,536	11,350	4,902	1250	27,700	
Sb	98	29	25	8	2	40	
As	114	6	6	4	2	15	
Ва	140	161	145	70	25	390	
Ве	129	1	1	1	< DL	6	
Cd	125	3	3	1	< DL	7	
Ca	114	3,495	3,044	1,839	190	9,300	
Cr	144	24	23	, 6	2	42	
Со	144	16	14	7	6	40	
Cu	143	15	14	6	3	34	
Fe	139	18,636	17,914	5,306	8,500	35,400	
Pb	127	29	25	16	7	84	
Li	92	10	9	3	< DL	17	
Mg	133	2,437	2,256	956	417	5,900	
Mn	127	495	370	334	49	1,400	
Hg	Background less	than the detect	ion limit of	0.1 mg/Kg			
Ni	138	19	18	7	7	43	
K	145	757	698	311	255	1,701	
Se	Background less	than the detect	ion limit of	0.5 mg/Kg			
Ag	96	3	2	2	1	13	
Na	136	486	437	202	4 9	982	
Tl	Background less	than the detect	ion limit of	1.0 mg/Kg			
V	141	35	34	7	6	54	
Zn	141	4 5	39	29	6	220	

< DL - Less than detection limit

Source: MK-F, 1988b

calculating the arithmetic mean, arithmetic standard deviation, and the geometric mean. This statistical information is presented in Table 3. These background ranges were used to evaluate the data from this investigation.

3.1 MATERIAL STAGING AREA

Analyses of the samples from the Material Staging Area detected no significant concentrations of nitrate, fluoride, sulfate, chloride, or nitroaromatic compounds. However, several organic compounds were detected. Appendix B presents the results of analyses for metals and inorganic anions in the Material Staging Area samples. Table 4 summarizes the significant organic data.

Twenty-eight samples were taken from five boreholes. Phthalates were identified in 12 samples. Phthalates are usually a result of laboratory contamination. The most common source is from the leaching of sample containers and laboratory tubing. However, dimethyl phthalate and di-n-butylphthalate are constituents of explosive propellants used in fuel matrices of double base rocket propellants, and they have also been used as insecticide propellant.

Phthalates were identified in samples taken from all five areas and were not detected in any of the field blanks. Therefore, they could be widespread in the area soils at a level of about one part per million. The final determination of the effects of phthalates in WSCP/WSRP soils will be made in the overall soil characterization program.

Aldrin, a pesticide, was found in two samples from the MSA. In sample SO-51137, 101068, Aldrin was found in the 0-ft to 2-ft, 2-ft to 4-ft, 4-ft to 6-ft, and 10-ft to 12-ft samples in concentrations ranging from 20 ug/kg to 1,600 ug/kg. In sample

TABLE 3

STATISTICAL DATA FOR
BACKGROUND INORGANIC ANION CONCENTRATIONS
IN SOILS AT THE WSCP/WSRP

				ARITHMETIC	ONSITE BA	CKGROUND GES
COMPOUND	SAMPLE SIZE	ARITHMETIC MEAN mg/Kg	GEOMETRIC MEAN mg/Kg	STANDARD DEVIATION mg/Kg	LOW mg/Kg	HIGH mg/Kg
Nitrate	250	2.5	1.09	2.0	0.5	10.0
Sulfate	247	33.0	23.00	27.0	1.0	110.0
Chloride	228	4.4	3.60	2.6	0.5	14.0
Fluoride	250	7.7	6.30	4.3	1.0	18.0

TABLE 4
MATERIAL STAGING AREA DATA SUMMARY

Organics Phthalates found in 12 samples

Chemical	Concentration	Interval	Sample Location	NO.
Aldrin 1	240 ug/kg 70 ug/kg ,600 ug/kg 20 ug/kg 18 ug/kg	(0-2') (2-4') (4-6') (10-12') (8-10')	SO-51137, 101068 SO-51137, 101068 SO-51137, 101068 SO-51137, 101068 SO-51500, 101190	2 2 2 2 2 2

SO-51500, 101190, Aldrin was found in the 8-ft to 10-ft sample at 18 ug/kg.

Aldrin is a chlorinated organic contact/fumigant insecticide. It was used to control soil insects in the 1950s and 1960s. Typically, aldrin undergoes biodegradation by oxidation to dieldrin with 75% to 100% disappearance from soils in one to six years. Dieldrin also degrades with 75% to 100% disappearance from soils in three to 25 years.

Given this biodegradation, the low concentrations detected during this investigation should not prohibit these IRA activities. The final effect of low aldrin concentrations will be evaluated in the Phase II chemical soil characterization program.

In summary, no chemical soil contamination was detected in the MSA which would effect performance of this IRA.

3.2 ASH POND ISOLATION DIKE

The Ash Pond Isolation Dike (AID) data summary (Table 5) presents the results of the analysis for detected organics, PCBs, pesticides, and nitroaromatic compounds. No elevated metal or inorganic anion concentrations were observed in AID soils.

Twenty-three samples were taken from five boreholes. The organic test results indicated phthalates in ten samples. Aroclor, a PCB, was identified in two samples at 270 ug/kg. One sample contained 1.04 ug/g of 2,4 DNT. These concentrations are well below cleanup criteria established at similar sites for the same compounds.

Most of the remaining organic compounds in Table 5 are associated with coal tar, gasoline, motor oil, and wood

TABLE 5

ASH POND DIKE DATA SUMMARY

Organics

Phthalates found in 10 samples.

2,4 DNT - 1.04 ug/g

Aroclor - 270 ug/kg

The following organics were identified in sample SO-51100, $100335\ (8-10')$:

Concentration

^{*} Below U.S. EPA-CLP contract required detection limits

preservatives and have been previously identified in the Ash Pond area (MK-F, 1988b). The compounds 2,4,6 trichlorophenol and 2,4,5 trichlorophenol are used widely in pesticides, fungicides, and bactericides. Hexachlorobutadiene is used as a solvent for synthetic rubber, heat transfer fluids, and washing fluids for removing hydrocarbons.

All of these compounds, with the exception of diethylphthalate, were detected below the U.S. EPA Contract Laboratory Program contract-required detection limits. These contract-required detection limits are established to detect concentrations of environmental concern. Detected concentrations below these limits should not be of concern from an environmental regulation standpoint. Appendix B presents metals and inorganic anion data for the AID samples.

The proposed IRA will not be affected by the chemical contaminants present in this area. No increase in chemical concentrations via surface discharge is expected as a result of impounding or diverting water around Ash Pond.

3.3 SOUTHEAST ISOLATION DIKE

Data for the Southeast Isolation Dike (SID) area are summarized in Table 6. These data represent 22 samples taken from five boring locations. No elevated concentrations of metals, inorganic anions, or nitroaromatic compounds were detected in the SID area. Organic analyses indicated phthalates present in six samples. Aroclor 1248, a PCB, was detected in one sample (SO-50160, 98735) in the 0-ft to 2-ft interval at 468 ug/kg. In the same sample, fluoranthene and pyrene were detected in the 2-ft to 4-ft interval, pyrene in the 4 ft to 6 ft interval, and phenol and 2-chlorophenol in the 8-ft to 10-ft interval. These compounds are associated with coal-tar by-products. Appendix B presents metal and inorganic anion data for the Southeast Isolation Dike samples.

TABLE 6 SOUTHEAST ISOLATION DIKE DATA SUMMARY

Organics

Phthalates found in six samples.

Concentration

Aroclor 1248 - 1.04 ug/g
Fluoranthene - 270 ug/kg *
Phenol - 14 ug/kg *
2-Chlorophenol - 11 ug/kg *
Pyrene - 110 ug/kg *
56 ug/kg *

^{*} Below U.S. EPA CLP contract required detection limit.

TABLE 7 ADMINISTRATION BUILDING AREA DATA SUMMARY

<u>Organics</u>

Phthalates were found in 24 samples.

CONCENTRATION	SAMPLE LOCATION	DEPTH
- 53 ug/kg* 59 ug/kg* 47 ug/kg* 17 ug/kg 10 ug/kg 18 ug/kg	49101, 100500 49101, 100500 49101, 100500	4-6' 4-6' 4-6'
12 ug/kg 1.4 ug/g		
	- 53 ug/kg* 59 ug/kg* 47 ug/kg* 17 ug/kg 10 ug/kg 18 ug/kg 12 ug/kg	- 53 ug/kg* 49101, 100500 59 ug/kg* 49101, 100500 47 ug/kg* 49101, 100500 17 ug/kg 10 ug/kg 18 ug/kg 12 ug/kg

<u>Nitrates</u>

High nitrate levels were found in sample SO-49101,100500.

CONCENTRATION	<u>DEPTH</u>
141 ug/g 1,285 ug/g 1,354 ug/g 1,297 ug/g 1,202 ug/g	(2-4') (4-6') (6-8') (8-10') (10-12')
1,108 ug/g	(12-14')

Sulfate - SO-49250 - 100140

CONCENTRATION	DEPTH
1,548 ug/gl	(2-4')

No chemical contamination which would impact the performance of this IRA was detected.

3.4 ADMINISTRATION BUILDING AREA

The Administration Building Area (ABA) data are summarized in Table 7. This summary represents 74 samples from 13 boring locations collected during this investigation. Samples were collected from four general locations in this area during the Phase I chemical soil sampling program (MK-F, 1988a). Several boreholes were located to confirm the past findings of elevated nitrate levels at depth in the original proposed Administration Building location. These findings were made during Phase I chemical soils characterization sampling in early 1987 (MK-F, 1988b). It was determined that the drainage from one of the major process buildings flowed under this location before the area was regraded to its current topography. Additional sample locations were also selected to evaluate other potential building locations. All ABA sampling locations and proposed building locations are presented in Figure 2.

Location SO-49101, 100500, was sampled to confirm previous detection of nitrates. These data support the previous findings of elevated nitrate levels. Elevated nitrate concentrations were detected in all intervals from 2-ft to 14-ft. The source of this contamination is a drainage ditch from WSOW Building 1-T-9 (trinitration house) which was revealed during aerial photography analysis and interpretation.

N-nitrosodiphenylamine, fluoranthene, and pyrene were also identified in the 4-ft to 6-ft sample. Chloroethane was detected in one sample at 12 ug/kg and 2,4,6 TNT was detected in one sample at 1.4 ug/g. Sulfate was detected in one sample (SO-49250 -100140) at an elevated concentration of 1,548 ug/g. Methylene chloride, a probable laboratory contaminant, was

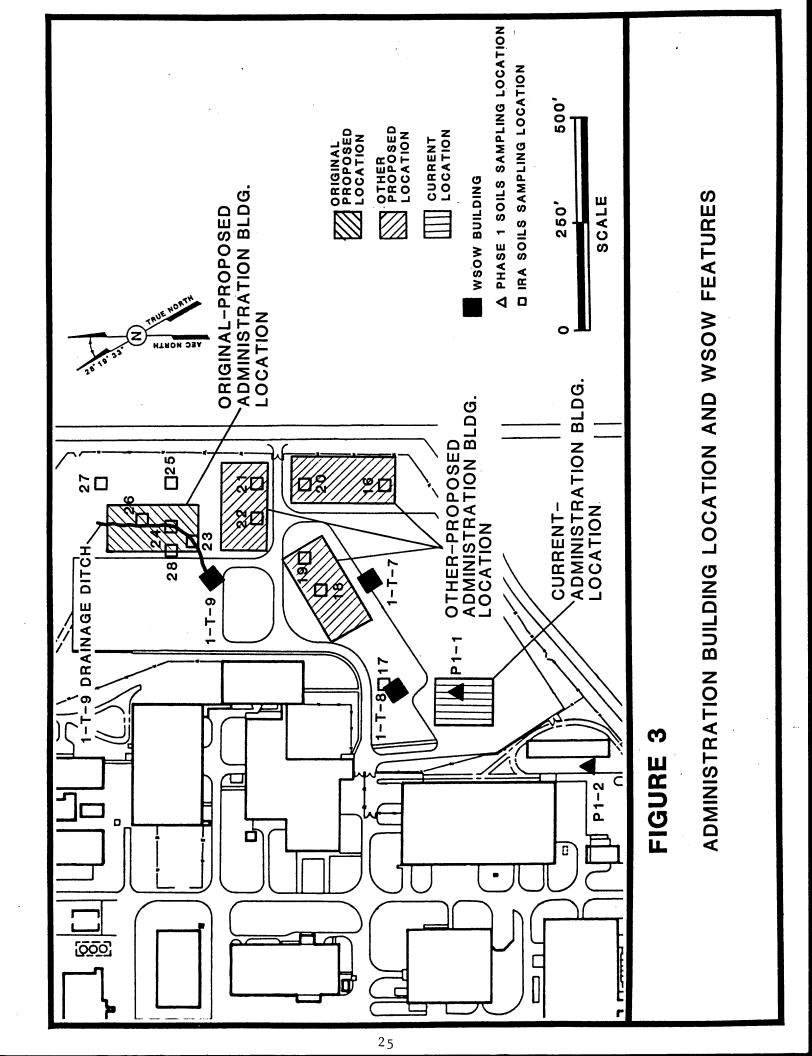
detected in three samples. Appendix B presents data on metal and inorganic anions for the Administration Building samples.

All other IRA activities discussed in this report were designed in support of remedial action and were temporary in nature. The Administration Building was a potentially permanent structure which was to be constructed in an uncontaminated area unaffected by remedial actions. For this reason, additional data were collected and presented, and further interpretations were made.

Previous investigations of WSOW contamination indicated that several process and support areas were most likely contaminated with nitroaromatic compounds. These areas include the wash house, settling tanks, wooden wastewater lines, burn areas, and wastewater lagoons. Chemical contamination from other sources was possible, but was probably less severe with respect to size and concentration than the areas mentioned above.

The siting of the ABA was evaluated using this information. None of the buildings or areas involved in the final production phases or purification process of TNT are located in the vicinity of the ABA. The closest area of concern is more than 700 ft north of the building site and is in a different drainage basin from the ABA site. All WSOW features, proposed building locations, and sampling locations are shown in Figure 3.

The closest two WSOW buildings to the ABA were Buildings 1-T-8 (Acid Recovery) and 1-T-7 (Mono-Nitration). No wastewater was generated in these process buildings. Aerial photography analysis and interpretation of 1945 imagery indicated that there were no drainage features from these process buildings through the ABA. This confirmed that wastewater was not generated.



The extensive decontamination efforts performed in 1954, prior to the transfer of WSOW land to the AEC, are documented (MK-F, 1988a) and appear to have been thorough. This interpretation was supported by the absence of elevated concentrations of nitroaromatic compounds in potential source areas as documented in this investigation and the Phase I chemical soil data report (MK-F, 1988b). It may be concluded that the contamination, if present, was removed in 1954.

The ABA area was used for personal vehicle parking for WSUFMP personnel. All contaminants from the process and support facilities drained away from this area. Therefore, no contamination from WSUFMP sources was expected. In addition, any chemical contamination from WSUFMP processes would likely be accompanied by elevated radioactivity levels. Field surveys and soil sample analysis have documented that radioactivity levels are not elevated in this area.

The available data indicate that chemical soil contamination is not present in the ABA and that a permanent facility could be sited at the proposed location.

3.5 CONSTRUCTION STAGING AREA

The Construction Staging Area (CSA) data indicated no chemical soil contamination from four samples taken at the two boring locations. The concentrations of all detected metals were within the background ranges for the Weldon Spring Site. The organic analysis indicated phthalates and methylene chloride present in two samples taken from the 0-ft to 7-ft interval. Both of these organic compounds are probable laboratory contaminants. No significant concentrations of nitroaromatic compounds, nitrate, sulfate, chloride, or fluoride were found.

Chemical soil contamination was not found in the area proposed to be used as a Construction Staging Area.

4 DATA QUALITY ANALYSIS

Analytical quality control procedures were performed according to EPA Contract Laboratory Program (CLP) criteria where applicable. The following summary addresses analytical conformance for GC/MS, GC/HPLC, and inorganic measures. Reference should be made to the CLP quality control requirements for specific control limits. Additional QC information on percent recoveries and duplicate analyses is presented in Appendix D.

GC/MS

The GC/MS analysis conformance summary indicated no blank contamination detected in the B/N or A/E fractions. Methylene chloride was detected in the blank VOA fraction at 2.5 ug/l. Surrogate recoveries were within required limits for the VOA fraction. Fifteen samples were not within the acceptable recovery range for the B/N, A/E fractions. All samples were analyzed within the specified holding time.

GC-(EPA/CLP)/HPLC (USATHAMA)

GC/HPLC conformance summary indicated no contaminants were detected in any of the blank samples. All samples were analyzed within specified holding times.

Metals/Inorganics

The metal/inorganic conformance summary indicated no contaminants were detected in any blank samples. All analyses were performed within specified holding times.

In summary, the data presented in this report is valid and of sufficient quality to be used in this and future assessments.

5 CONCLUSIONS

This soil sampling effort for selected interim response actions provides sufficient data of acceptable quality to support the design of the IRAs. The data from samples collected at depth from each of the five areas provide chemical characterization information pertinent to evaluating the environmental impact of the interim response action.

The data and interpretations presented in this report confirm those of previous investigations indicating limited chemical soil contamination on the WSCP/WSRP. No chemical contamination which would significantly affect the five IRAs discussed in this report was discovered.

This investigation also supported previous conclusions regarding the absence of significant nitroaromatic soil contamination.

The data presented in this report will also be used in support of the overall soil characterization as detailed in the soil sampling plan (MK-F, 1988a).

6 REFERENCES

- Karelverschueren Agr University of Wageningen, Netherlands, 1983. Handbook of Environmental Data on Organic Chemicals, 2nd Edition, Van Nostrand Reinhold Co.
- MK-F, 1987. Phase I Water Quality Assessment Report, DOE/OR/21548-003, December.
- MK-F, 1988a. Chemical Soil Investigation Sampling Plan for the Weldon Spring Chemical Plant/Raffinate Pits, DOE/OR/21548-013, May.
- MK-F, 1988b. Phase I Chemical Soil Investigation Data Report for the Weldon Spring Chemical Plant/Raffinate Pits, DOE/OR/21548-016, June.
- U.S. Department of Agriculture, 1982. Soil Survey of St. Charles County, Missouri, Soil Conservation Service, May.

APPENDIX A SOIL SAMPLE DESCRIPTION

SOIL SAMPLE DESCRIPTION

MATERIAL STAGING AREA

Sample Number:	SO-51150,	101207
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Location: Material Staging Area

Analysis Performed: Metals, inorganics, nitroaromatics, pH, %

moisture

Sample Description:

0 to 2'	6 inches of topsoil, 1.5 feet of gray-brown mottled clay
2 to 4'	2 feet gray-brown mottled clay
4 to 6'	2 feet gray-brown mottled clay
6 to 8'	6 inches gray-brown mottled clay, 1.5 feet
	brown clay moist
8 to 10'	2 feet wet brown clay

Sample Number: SO-51137,101068

Location: Material Staging Area

Analysis Performed: Metals, inorganics, nitroaromatics,

semi-volatiles, PCBs, pesticides, pH, %

moisture

Sample Description:

_		_ •	
0	to	2'	8 inches topsoil, 16 inches red-brown clay
2	to	4'	2 feet gray-brown mottled clay
4	to	6 '	2 feet gray-brown mottled clay
6	to	8'	6 inches brown clay with black chunks; 18 inches moist brown clay
8	to	10'	6 inches brown clay; 18 inches gray-brown mottled clay
10	to	12'	2 feet gray-brown mottled clay

Sample Description: SO-51500,101190

Location: Material Staging Area

Analysis Performed: Metals, inorganics, nitroaromatics,

semi-volatiles, PCBs, pesticides, pH, %

moisture

Sample Description:

0 to 2'	l foot top soil, l foot red-brown clay
2 to 4'	2 feet dry gray-brown mottled clay
4 to 6'	2 feet dry gray-brown mottled clay
6 to 8'	1 foot dry gray-brown mottled clay, 1 foot
	same but moister with black oxide
8 to 10'	2 feet buff-brown mottled, moist
10 to 12'	18 inches buff-brown mottled, moist; 6 inches
	brown-black mottled moist

Sample Number: S0-51445,101065

Location: Material Staging Area

Analysis Performed: Metals, inorganics, nitroaromatics, pH, %

moisture

Sample Description:

0 t	o 2'	8 inches topsoil, 16 inches brown-gray clay
2 t	o 4'	18 inches brown-gray clay, 6 inches dense
		gray-brown mottled clay
4 t	o 6'	2 feet gray-brown mottled clay, dry
6 t	o 8'	2 feet gray-brown mottled clay, dry
8 t	o 10'	2 feet gray-brown mottled clay, dry
10 t	o 12'	18 inches gray-brown mottled clay, 6 inches
		brown clay with black specks, possibly oxide

Sample Number: S0-51360,101175

Location: Material Staging Area

Analysis Performed: Metal, inorganics, nitroaromatics, pH, %

moisture.

Sample Description:

0 to 2'	4 inches road rocks, 10 inches brick red
	clay, 10 inches brown-buff clay
2 to 4'	2 feet dry gray-brown clay
4 to 6'	2 feet dry brown-gray clay
6 to 8'	2 feet brown-gray clay
8 to 10'	2 feet brown-gray clay, gummy

ASH POND DIKE

Sample Number: S0-51180,100335
Location: Ash Pond Dike
Analysis Performed: Metals, inorganics, nitroaromatics

Sample Description:

0 to 2'
4 inches of topsoil, 20 inches buff dense but friable clay - glass beads (volcanic)
2 to 4'
20 inches buff clay with more rust color increasing with depth, 4 inches

Sample Number: S0-51125,100260 Location: Ash Pond Dike

Analysis Performed: Metals, inorganics, nitroaromatics

Sample Description:

1 foot topsoil, 1 foot buff to orange loose,
dry clay
2 to 4' 8 inches orange dry clay, 8 inches cherty dry
clay, 8 inches rusty cherty clay
4 to 6' 2 feet red-rusty, cherty clay
6 to 8' 2 feet red-rusty, cherty clay

8 to 10' 1 foot red-rusty, cherty clay, 1 foot

buff-brown, moist clay

Sample Number: SO-51150,100220 Location: Ash Pond Dike

Analysis Performed: Metals, inorganics, nitroaromatics,

semi-volatiles, PCBs, pesticides

Sample Description:

0 to 2'
2 inches topsoil, 16 inches brown-gray clay,
6 inches brown dry topsoil with chert
towards bottom
2 to 4'
2 inches brown topsoil cherts, 22 inches
rusty-red clay with 80% chert
4 to 6'
5 feet rusty red clay 80% chert
6 to 8'
7 to 10'
8 to 10'
9 inches rusty red clay 80% chert
10 inches rusty red clay 80% chert
11 inches
12 inches

Sample Number: SO-51100,100335 Location: Ash Pond Dike

Analysis Performed: Metals, inorganics, nitroaromatics,

semi-volatiles, PCBs, pesticides

Sample Description:

0 to 2'	6 inches topsoil - 18 inches gray-brown clay
2 to 4'	2 feet buff friable clay, rust color increases with depth
4 to 6'	<pre>2 feet of dense, moist, gray-rust mottled clay, friable at top 6 inches with black specks</pre>
6 to 8'	1 foot dense, moist, gray-rust mottled clay, 1 foot chert chunks with clay, chert 80%, clay 20%
8 to 10'	2 feet rusty clay with chert (20%)

Sample Number: SO-51308,100085 Location: Ash Pond Dike

Analysis Performed: Metals, inorganics, nitroaromatics

Sample Description:

0 to 2'	1 foot topsoil, organics layer, more topsoil (6 inches), dense gray clay - 6 inches
2 to 4'	l foot dense gray clay, 6 inches gray clay with Fe oxide stains, 1 inch dry cherty clay with Fe oxide
4 to 6'	<pre>1 foot gray with Fe oxide stain increasing with depth to gray/brown mottled in second foot</pre>
6 to 8' 8 to 10'	2 feet gray-rust moist dense clay 2 feet gray-rust moist dense clay

SOUTHEAST ISOLATION DIKE

SUUTHEAST ISULATION	DIKE
Sample Number: Location: Analysis Performed: Sample Description:	SO-50290,98700 SE Isolation Dike Metals, inorganics, nitroaromatics
0 to 2'	10 inches topsoil, 10 inches gray-brown moist mottled dense clay, 4 inches clay with cherts 50%, friable
2 to 4'	14 inches brown clay with chert, 10 inches brown clay dense and friable
4 to 6' 6 to 8'	2 feet brown-gray mottled clay, moist & dense 1 foot gray-brown mottled clay, 6 inches dark gray dense clay, 6 inches same with chert
Sample Number: Location: Analysis Performed: Sample Description:	SO-50140,98820 SE Isolation Dike Metals, inorganics nitroaromatics
0 to 2'	4 inches topsoil, 6 inches chert, 6 inches buff moist clay with 50% chert, 8 inches clay without chert
2 to 4'	20 inches orange-gray mottled, moist clay, 4 inches same but siltier
4 to 6'	2 feet orange-gray mottled moist clay with minor sandy layers
6 to 8'	2 feet orange-gray mottled clay, moist with black specks toward bottom 1 foot
Sample Number: Location: Analysis Performed: Sample Description:	SO-50230,98991 SE Isolation Dike Metals, inorganics nitroaromatics
0 to 2'	8 inches of topsoil, 16 inches brown-rust mottled clay
2 to 4'	1 foot buff, dense clay, 1 foot buff silty clay, friable
4 to 6' 6 to 8'	<pre>2 feet buff-gray silty clay 8 inches buff-gray silty clay, 16 inches buff-gray mottled dense clay</pre>

Sample Number: SO-50160,98735 Location: SE Isolation Dike

Analysis Performed: Metals, inorganics, nitroaromatics, semi-volatiles, PCBs, pesticides

Sample Description:

0 to 2'
2 to 4'
2 feet dry gray clayey silt
4 to 6'
6 to 8'
8 inches topsoil, 16 inches gray silty clay
2 feet dry gray clayey silt
6 to 8'
8 inches gray clayey silt
9 feet gray silty clay - darker at top
9 inches light gray loose clay, 4 inches
9 dense, moist clay, 12 inches dense orange9 gray mottled clay with chert
10 to 12'
2 feet orange-gray mottled clay

Sample Number: S0-50252,98800 Location: SE Isolation Dike

Analysis Performed: Metals, inorganics, nitroaromatics

Sample Description:

0 to 2'	4 inches topsoil, 14 inches buff, moist, dense clay, 6 inches buff-gray silty clay, friable
2 to 4'	22 inches buff-gray silty clay, 2 inches of light gray silt
4 to 6'	<pre>16 inches brick red-gray mottled clay, inches buff-gray mottled clay</pre>
6 to 8'	<pre>1 foot buff-gray mottled clay, 6 inches buff- gray mottled with black specks, 6 inches without</pre>

ADMINISTRATION BUILDING

Sample Number: SO-49475,99983 Administration Building Location: Analysis Performed: Metals, inorganics, nitroaromatics Sample Description: 0 to 2' 6 inches topsoil, 18 inches brown dry clay 2 to 4' 2 feet dense brown clay with small rocks 4 to 6' 8 inches brown clay, 16 inches moist soft gray clay 2 feet gray moist clay 6 to 8' 2 feet gray-brown mottled clay 8 to 10' 10 to 12' 2 feet gray-brown mottled clay so-49000,100180 Sample Number: Location: Administration Building Analysis Performed: Metals, inorganics, nitroaromatics Sample Description: 0 to 2' 6 inches of rock fill, 18 inches moist brown clay 2 to 4' 2 feet moist brown clay 4 to 6' 2 feet moist brown-gray clay 6 to 8' 2 feet most gray-brown clay Sample Number: SO-49000,99985 Administration Building Location: Metals, inorganics, nitroaromatics, Analysis Performed: volatiles, semi-volatiles Sample Description: 0 to 2' 2 inches topsoil, 6 inches soil with rock fill, 8 inches dry buff clay, 8 inches moist dense buff clay 2 feet gray-brown mottled clay, moist & dense 2 to 4' 4 to 6' 2 feet gray-brown mottled clay 6 to 8' 2 feet gray-brown mottled clay Sample Number: SO-49000, 100295 Location: Administration Building Analysis Performed Metals, inorganics, nitroaromatics Sample Description: 0 to 2' 4 inches topsoil, 8 inches buff, dry, clay, 12 inches buff-brown mottled, hard clay 2 to 4' 2 feet buff-brown mottled clay - top 8 inches hard, lower 16 inches softer 4 to 6' 2 feet brown-gray mottled clay, moist 6 to 8' 2 feet brown-gray mottled clay, moist

Sample Number: SO-49172,100180 Administration Building Location: Metals, inorganics, nitroaromatics Analysis Performed: Sample Description: 6 inches topsoil, 18 inches brown clay-denser 0 to 2' and more red towards bottom 2 to 4' 1 foot brown clay, dense, 1 foot brown-gray mottled clay 2 feet gray-brown clay, with some silty clay 4 to 6' layers 2 feet gray-brown clay, dense 6 to 8' so-49250,100140 Sample Number: Administration Building Location: Metals, inorganics, nitroaromatics Analysis Performed: Sample Description: 6 inches topsoil, 18 inches brown silty clay 0 to 2' 2 feet of brown clay with silty layer (1/2"2 to 4' thick) at 10 inches down 2 feet brown-gray mottled clay 4 to 6' 6 to 8' 2 feet brown-gray mottled clay - dense 2 feet brown-gray mottled dense clay 8 to 10' so-49000,100500 Sample Number: Administration Building Location: Metals, inorganics, nitroaromatics Analysis Performed: Sample Description: 4 inches topsoil, 20 inches buff dry clay 0 to 2' 2 feet brown-buff, dry clay 1 foot brown-buff, dry clay, 1 foot moist 2 to 4' 4 to 6' dense, brown clay 6 to 8' 2 feet sticky moist brown-gray mottled clay 2 feet sticky moist brown-gray mottled clay 8 to 10' 2 feet moist brown-gray clay 10 to 12' SO-49080, 100295 Sample Number: Administration Building Location: Metals, inorganics, nitroaromatics Analysis Performed: Sample Description: 4 inches topsoil, 8 inches soft moist, brown 0 to 2' clay, 12 inches hard, dense, brown clay 1 foot hard brown clay, 1 foot hard brown-2 to 4' gray mottled clay 4 to 6' 2 feet brown-gray mottled clay 2 feet brown-gray mottled clay 6 to 8' 2 feet brown-gray mottled clay - softer at 8 to 10' bottom 1 foot

Sample Number: S0-49082,100570

Location: Administration Building

Analysis Performed: Metals, inorganics, nitroaromatics, semi-volatiles, PCBs, pesticides

Sample Description:

0	to	2'	6	inche	es topsoil, 18 inches dry, buff clay
2	to	4'	2	feet	dry, brown clay
4	to	6'	1	foot	dry brown clay, 1 foot moist brown
			(clay	
6	to	8'			red-brown clay - moist
8	to	10'	2	feet	brown-gray mottled clay - moist
10	to	12'	2	feet	brown-gray mottled clay
12	to	14'	2	feet	gray-brown mottled clay
14	to	16'	2	feet	gray-brown mottled clay

Sample Number: SO-49160,100500

Location: Administration Building

Analysis Performed: Metals, inorganics, nitroaromatics

Sample Description:

0 to 2'	4 inches topsoil, 4 inches rock fill, 1
	foot dry buff clay, 4 inches moist brown
	clay
2 to 4'	2 feet moist brown clay

2 to 4' 2 feet moist brown clay

4 to 6' 1 foot moist brown clay, 1 foot moist brown-gray clay

6 to 8' 2 feet moist brown-gray clay

8 to 10' 1 foot moist brown-gray clay, 1 foot moist brown clay

2 feet moist brown clay

Sample Number: S0-49000,100665

Location: Administration Building

Analysis Performed: Metals, inorganics, nitroaromatics

Sample Description:

10 to 12'

0	to	2'	6 inches topsoil, 8 inches moist clayey
			dirt, 10 inches dry brown clay
2	to	4'	2 feet dry brown clay
4	to	6'	2 feet dry brown clay
6	to	8'	1 foot dry brown clay, 1 foot moist,
			red-gray mottled clay
8	to	10'	2 feet moist red-gray mottled clay
10	to	12'	1 foot moist red-gray mottled clay, 1 foot
			moist brown clay

Sample Number: S0-49101,100500

Location: Administration Building

Analysis Performed: Metals, inorganics, nitroaromatics,

semi-volatiles, PCBs, pesticides

Sample Description:

Λ	to	21	/.	inche	e tone	oil 9	20 inches di	ew buff	claw
2	to	4'	2	feet	dry bi	rown cl	lay, harder	at botto	om
4	to	6'	2	feet	soft,	moist	brown clay		
6	to	8'	2	feet	soft,	moist	brown-gray	mottled	clay
8	to	10'	2	feet	soft,	moist	brown-gray	mottled	clay
10	to	12'	2	feet	soft,	moist	brown-gray	mottled	clay
12	to	14'	2	feet	soft,	moist	gray-brown	mottled	clay
14	to	16'	2	feet	soft,	moist	brown-gray	mottled	clay

Sample Number: SO-

SO-49132,100440

Location:

Administration Building

Analysis Performed: Metals, inorganics, nitroaromatics, semi-volatiles, PCBs, pesticides

Sample Description:

0	to	2'	4	inche	es soi	i1, 20	inche	es brow	wn-	gray	clay,
			(dense							
2	to	4'	2	feet	hard	brown-	gray	clay			
4	to	6'	2	feet	hard	brown.	gray	clay,	mc	oist	
6	to	8 '	2	feet	hard	brown-	-gray	clay			
8	to	10'	2	feet	hard	brown-	gray	clay			
10	to	12'				brown-					
12	to	14'							1	foot	softer
				orown			0 ,	• ,			

CONSTRUCTION STAGING AREA

Sample Number: S0-50800,98150

Location: Construction Staging Area

Analysis Performed: Metals, inorganics, nitroaromatics,

volatiles, semi-volatiles, PCBs,

pesticides, pH, % moisture

Sample Description:

0 to 7' 12 inches topsoil, gray-brown mottled clay

8 to 15' gray-brown mottled clay

Sample Number: S0-50950,98300

Location: Construction Staging Area

Analysis Performed: Metals, inorganics, nitroaromatics,

volatiles, semi-volatiles, PCBs,

pesticides, pH, % moisture

Sample Description:

0 to 7' 6 inches topsoil, gray-brown mottled clay

8 to 15' gray-brown mottled clay

APPENDIX B
ANALYTICAL DATA - METALS, INORGANIC ANIONS

TABLE B-1

ADMINISTRATION BUILDING AREA IRA SOILS RESULTS - ANIONS

\$0-49000, \$100180-0, 2-1187	LOGRATION	DATE	CONCENTRATION (UG/G)					
\$0-49000, 100180-2, 4-1187	LOCATION		CHLORIDE	FLOURIDE	NITRATE	SULFATI		
\$0-49000, 100180-4, 6-1187	SO-49000,100180-0,2-1187	11/11/87	1.07	12.44	2.68	91.62		
\$0-49000,100180-4,6-1187	SO-49000,100180-2,4-1187	11/11/87	4.40	21.13	1.87	58.11		
\$\text{SO-49000,100295-0,2-1187}\$ \$11/11/87\$ \$12.79 \$9.76 \$1.73\$ \$163.41		11/11/87	1.53	17.60	2.84	28.30		
\$\text{SO-49000,100295-0,2-1187}\$ \$11/11/87\$ \$12.79\$ \$9.76\$ \$1.73\$ \$163.41 \text{SO-49000,100295-4,6-1187}\$ \$11/11/87\$ \$11/11/87\$ \$12.79\$ \$9.76\$ \$1.73\$ \$163.41 \text{SO-49000,100295-6,8-1187}\$ \$11/11/87\$ \$11.09\$ \$12.10\$ \$14.88\$ \$5.51\$ \$31.93\$ \$SO-49000,100500-0,2-1187\$ \$11/12/87\$ \$1.60\$ \$9.72\$ \$2.63\$ \$39.45\$ \$SO-49000,100500-10,12-1187\$ \$11/12/87\$ \$1.60\$ \$9.72\$ \$2.63\$ \$39.45\$ \$SO-49000,100500-10,12-1187\$ \$11/12/87\$ \$1.08\$ \$7.20\$ \$3.48\$ \$8.16\$ \$SO-49000,100500-4,6-1187\$ \$11/12/87\$ \$0.46\$ \$10.04\$ \$1.15\$ \$60.38\$ \$SO-49000,100500-4,6-1187\$ \$11/12/87\$ \$1.01\$ \$7.92\$ \$2.89\$ \$15.95\$ \$SO-49000,100500-6,8-1187\$ \$11/12/87\$ \$1.01\$ \$7.92\$ \$2.89\$ \$15.95\$ \$SO-49000,100500-8,10-1187\$ \$11/12/87\$ \$1.00\$ \$7.92\$ \$2.89\$ \$15.95\$ \$SO-49000,100500-8,10-1187\$ \$11/12/87\$ \$2.00\$ \$1.88\$ \$0.94\$ \$41.20\$ \$SO-49000,100665-0,2-1187\$ \$11/12/87\$ \$2.00\$ \$1.88\$ \$0.94\$ \$41.20\$ \$SO-49000,100665-10,12-1187\$ \$11/12/87\$ \$2.03\$ \$13.75\$ \$2.04\$ \$8.95\$ \$SO-49000,100665-4,4-1187\$ \$11/12/87\$ \$4.84\$ \$12.96\$ \$3.27\$ \$127.35\$ \$SO-49000,100665-6,8-1187\$ \$11/12/87\$ \$9.87\$ \$9.74\$ \$3.01\$ \$46.35\$ \$SO-49000,100665-6,8-1187\$ \$11/12/87\$ \$9.58\$ \$12.43\$ \$2.72\$ \$49.85\$ \$SO-49000,99985-0,2-1187\$ \$11/11/87\$ \$2.94\$ \$9.28\$ \$ND \$74.90\$ \$SO-49000,99985-4,6-1187\$ \$11/11/87\$ \$2.94\$ \$9.28\$ \$ND \$74.90\$ \$SO-49000,99985-2,4-1187\$ \$11/11/87\$ \$2.94\$ \$9.28\$ \$ND \$74.90\$ \$SO-49000,100295-2,4-1187\$ \$11/11/87\$ \$2.94\$ \$9.28\$ \$ND \$74.90\$ \$SO-49000,100295-2,4-1187\$ \$11/12/87\$ \$1.01\$ \$1.287\$ \$1.34\$ \$0.94\$ \$0.94\$ \$0.95\$ \$0.9988,100295-2,4-1187\$ \$11/12/87\$ \$1.26\$ \$1.81\$ \$2.90\$ \$2.40\$ \$1.11\$ \$1.11\$ \$1.15\$ \$2.40\$ \$3.55\$ \$3.59\$ \$3.59\$ \$3.59\$ \$3.59\$ \$3.59\$ \$3.59\$ \$3.59\$ \$3.59\$ \$3.59\$ \$3.59\$ \$3.59\$ \$3.60\$ \$3.60\$ \$3.60\$ \$3.60\$ \$3.60\$ \$3.60\$ \$3.60\$ \$3.60\$ \$3.60\$ \$3.60\$ \$3.60\$ \$3.60\$ \$3.60\$ \$3.60\$ \$3.60\$ \$3.60\$ \$3.60\$ \$3.60\$ \$3.60\$ \$3.60\$ \$3.60\$ \$3.60\$ \$3.60\$ \$3.60\$ \$3.60\$ \$3.60\$ \$3.60\$ \$3.60\$ \$3.60\$ \$3.60\$ \$3.60\$ \$3.60	· · · · · · · · · · · · · · · · · · ·	11/11/87	9.61	19.64	3.59	19.15		
\$0-49000,100295-2,4-1187		11/11/87	1.93	9.66	4.52	33.57		
\$0-49000,100295-4,6-1187		11/11/87	12.79	9.76	1.73	163.41		
\$\text{SO-49000,} 100295-6, 8-1187 \text{11/11/87} \text{13.09} \text{14.88} \text{5.51} \text{31.91} \\ \$\text{SO-49000,} 100500-0, 2-1187 \text{11/12/87} \text{1.60} \text{9.72} \text{2.63} \text{39.45} \\ \$\text{SO-49000,} 100500-2, 4-1187 \text{11/12/87} \text{1.60} \text{9.72} \text{3.48} \text{8.16} \\ \$\text{SO-49000,} 100500-2, 4-1187 \text{11/12/87} \text{0.61} \text{5.99} \text{0.98} \text{37.33} \\ \$\text{SO-49000,} 100500-6, 8-1187 \text{11/12/87} \text{0.61} \text{5.99} \text{0.98} \text{3.33} \\ \$\text{SO-49000,} 100500-8, 10-1187 \text{11/12/87} \text{1.09} \text{7.88} \text{1.32} \\ \$\text{SO-49000,} 100665-0, 2-1187 \text{11/12/87} \text{2.00} \text{1.88} \text{0.94} \text{4.12} \\ \$\text{SO-49000,} 100665-0, 2-1187 \text{11/12/87} \text{5.35} \text{3.27} \text{2.33} \\ \$\text{SO-49000,} 100665-4, 6-1187 \text{11/12/87} \text{5.63} \text{5.65} \text{4.34} \\ \$\text{SO-49000,} 100665-6, 8-1187 \text{11/12/87} \qu		11/11/87	19.39	12.10	1.40	27.73		
\$\text{SO-49000,} 100500-0, 2-1187 \text{11/12/87} \text{1.60} \text{9.72} \text{2.63} \text{3.48} \text{8.16} \\ \$\text{SO-49000,} 100500-10,12-1187 \text{11/12/87} \text{1.08} \text{7.20} \text{3.48} \text{8.16} \\ \$\text{SO-49000,} 100500-2, 4-1187 \text{11/12/87} \text{0.61} \text{5.99} \text{0.98} \text{3.73} \\ \$\text{SO-49000,} 100500-4, 6-1187 \text{11/12/87} \text{0.61} \text{5.99} \text{0.98} \text{3.73} \\ \$\text{SO-49000,} 100500-8, 10-1187 \text{11/12/87} \text{1.00} \text{7.92} \text{2.89} \text{1.21} \\ \$\text{SO-49000,} 100665-0, 2-1187 \text{11/12/87} \text{2.00} \text{1.88} \text{4.22} \\ \$\text{SO-49000,} 100665-0, 2-1187 \text{11/12/87} \text{5.35} \			13.09			31.91		
\$\text{SO-49000,} 100500-10,12-1187 \text{11/12/87} \text{1.08} \text{7.20} \text{3.48} \text{8.16} \\ \$\text{SO-49000,} 100500-2,4-1187 \text{11/12/87} \text{0.46} \text{10.04} \text{1.15} \text{60.38} \\ \$\text{SO-49000,} 100500-4,6-1187 \text{11/12/87} \text{0.61} \text{5.99} \text{0.98} \text{37.38} \\ \$\text{SO-49000,} 100500-8, 10-1187 \text{11/12/87} \text{1.00} \text{7.88} \text{1.33} \\ \$\text{SO-49000,} 100665-0,2-1187 \text{11/12/87} \text{2.00} \text{1.88} \text{0.94} \text{41.20} \\ \$\text{SO-49000,} 100665-0,2-1187 \text{11/12/87} \text{5.35} \text{3.75} \text{2.96} \text{2.72} \			1.60	9.72		39.47		
\$\text{SO-49000,100500-2,4-1187}\$ \$11/12/87\$ \$0.46\$ \$10.04\$ \$1.15\$ \$60.38\$ \$\text{SO-49000,100500-4,6-1187}\$ \$11/12/87\$ \$1.01\$ \$7.92\$ \$2.89\$ \$15.97\$ \$\text{SO-49000,100500-8,10-1187}\$ \$11/12/87\$ \$1.01\$ \$7.92\$ \$2.89\$ \$15.97\$ \$\text{SO-49000,100500-8,10-1187}\$ \$11/12/87\$ \$1.09\$ \$7.88\$ \$1.33\$ \$12.13\$ \$\text{SO-49000,100665-0,2-1187}\$ \$11/12/87\$ \$2.00\$ \$1.88\$ \$0.94\$ \$41.20\$ \$\text{SO-49000,100665-10,12-1187}\$ \$11/12/87\$ \$2.30\$ \$13.75\$ \$2.04\$ \$2.00\$ \$1.88\$ \$0.94\$ \$41.20\$ \$2.00\$ \$1.88\$ \$0.94\$ \$41.20\$ \$2.00\$ \$1.88\$ \$0.94\$ \$41.20\$ \$2.00\$ \$1.88\$ \$0.94\$ \$41.20\$ \$2.00\$ \$1.89\$ \$0.4900,100665-2,4-1187\$ \$11/12/87\$ \$2.53\$ \$13.75\$ \$2.04\$ \$3.27\$ \$2.00\$ \$1.89\$ \$3.27\$ \$2.127.33\$ \$3.20-49000,100665-2,4-1187\$ \$11/12/87\$ \$2.63\$ \$15.65\$ \$1.43\$ \$13.62\$ \$3.26\$ \$3.27\$ \$49.85\$ \$3.24900,100665-4,6-1187\$ \$11/12/87\$ \$9.87\$ \$9.74\$ \$3.01\$ \$46.33\$ \$50-49000,100665-8,10-1187\$ \$11/12/87\$ \$9.87\$ \$9.74\$ \$3.01\$ \$46.33\$ \$50-49000,99985-0,2-1187\$ \$11/11/87\$ \$2.94\$ \$9.28\$ \$10.72\$ \$2.143\$ \$2.72\$ \$49.85\$ \$50-49000,99985-4,6-1187\$ \$11/11/87\$ \$2.574\$ \$8.80\$ \$7.72\$ \$2.274.49 \$8.50-49000,99985-4,6-1187\$ \$11/11/87\$ \$2.574\$ \$8.80\$ \$7.72\$ \$2.274.49 \$8.50-49080,100295-0,2-1187\$ \$11/12/87\$ \$3.34\$ \$6.08\$ \$2.62\$ \$26.22\$ \$50-49080,100295-0,2-1187\$ \$11/12/87\$ \$3.34\$ \$6.08\$ \$2.62\$ \$26.22\$ \$50-49080,100295-4,6-1187\$ \$11/12/87\$ \$2.32\$ \$12.30\$ \$1.16\$ \$76.88\$ \$50-49080,100295-4,6-1187\$ \$11/12/87\$ \$1.20\$ \$21.40\$ \$1.67\$ \$4.30\$ \$50-49080,100295-6,8-1187\$ \$11/12/87\$ \$1.20\$ \$21.40\$ \$1.67\$ \$4.30\$ \$50-49080,100295-8,10-1187\$ \$11/12/87\$ \$1.20\$ \$21.40\$ \$1.67\$ \$4.30\$ \$50-49080,100295-6,8-1187\$ \$11/12/87\$ \$1.20\$ \$21.40\$ \$1.67\$ \$4.30\$ \$50-49080,100295-6,8-1187\$ \$11/12/87\$ \$1.20\$ \$21.40\$ \$1.67\$ \$4.30\$ \$50-49080,100295-6,8-1187\$ \$11/12/87\$ \$1.20\$ \$21.40\$ \$1.67\$ \$4.30\$ \$50-49080,100295-6,8-1187\$ \$11/12/87\$ \$1.59\$ \$7.48\$ \$4.31\$ \$32.22\$ \$50-49080,100570-0,2-1187\$ \$11/12/87\$ \$1.99\$ \$0.49082,100570-0,2-1187\$ \$11/12/87\$ \$3.99\$ \$6.11\$ \$1.26\$ \$1.73\$ \$3.58\$ \$2.99\$ \$6.2 \$2.10.50570-6,8-1187\$ \$11/12/87\$ \$3.99\$ \$6.11\$ \$1.29\$ \$1.26\$ \$1.40\$ \$1.67\$ \$2.20\$ \$2.60\$ \$2.20\$ \$2.60\$ \$2.20\$ \$2.60\$ \$2.20\$ \$						8.16		
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S0-49101,100500-2,4-1187 11/12/87 0.93 ND 141.32 88.3 S0-49101,100500-4,6-1187 11/12/87 1.11 1.11 1285.56 7.0	•							
SO-49101,100500-4,6-1187 11/12/87 1.11 1.11 1285.56 7.0								
- COLADOO COMBOO & U 1107 1107 1107 1 LU 1 71 1764 64 64	S0-49101,100500-4,6-1187 S0-49101,100500-6,8-1187	11/12/87	1.11		1354.64	6.68		

TABLE B-1 (continued)

ADMINISTRATION BUILDING ARRA IRA SOILS RESULTS - ANIONS

LOGAMION	DAME		CONCENTRATION (UG/G)					
LOCATION	DATE SAMPLED	CHLORIDE	FLOURIDE	NITRATE	SULFATE			
SO-49101,100500-8,10-1187	11/12/87	1.49	1.49	1297.08	6.81			
SO-49132,100440-0,2-1187	11/12/87	1.40	4.55	3.27	77.56			
SO-49132,100440-10,12-1187	11/12/87	0.85	8.41	2.68	15. 4 8			
SO-49132,100440-12,14-1187	11/12/87	0.83	5.13	2.62	13.23			
SO-49132,100440-14,16-1187	11/12/87	54.93	2.52	3.49	18.75			
SO-49132,100440-2,4-1187	11/12/87	0.83	5.01	1.07	129.93			
SO-49132,100440-4,6-1187	11/12/87	0.95	11.19	1.9	16.79			
SO-49132,100440-6,8-1187	11/12/87	0.7	12.39	3.39	13.32			
SO-49132,100440-8,10-1187	11/12/87	1.06	9.78	8.72	11.19			
SO-49160,100500-0,2-1187	11/12/87	3.54	10.06	1.49	46.31			
SO-49160,100500-10,12-1187	11/12/87	4.86	10.94	4.13	6.69			
SO-49160,100500-2,4-1187	11/12/87	3.87	10.89	2.42	35.93			
SO-49160,100500-4,6-1187	11/12/87	5.97	15.45	2.11	5.97			
SO-49160,100500-6,8-1187	11/12/87	4.87	14.71	3.68	5.46			
\$0-49160,100500-8,10-1187	11/12/87	4.40	13.56	2.62	5.35			
SO-49172,100180-0,2-1187	11/11/87	9.77	13.49	2.76	170.77			
SO-49172,100180-2,4-1187	11/11/87	25,59	12.19	2.66	237.63			
SO-49172,100180-4,6-1187	11/11/87	30.12	11.81	1.09	14.94			
SO-49172,100180-6,8-1187	11/11/87	26.00	14.26	7.48	16.30			
SO-49250,100140-0,2-1187	11/10/87	2.36	10.87	0.82	136.90			
SO-49250,100140-2,4-1187	11/10/87	7.88	5.93	1.08	1548.14			
SO-49250,100140-4,6-1187	11/10/87	22.58	11.72	4.00	17.94			
SO-49250,100140-6,8-1187	11/10/87	1.60	11.66	1.14	30.86			
\$0-49250,100140-8,10-1187	11/10/87	0.86	12.92	1.80	14.34			
SO-49475,99985-0,2-1187	11/10/87	1.82	8.53	12.97	89.22			
SO-49475,99985-10,12-1187	11/10/87	4.07	19.39	3.78	15.42			
SO-49475,99985-2,4-1187	11/10/87	1.81	5.69	9.86	116.34			
SO-49475,99985-4,6-1187	11/10/87	4.17	8.57	2.25	43.04			
SO-49475,99985-6,8-1187	11/10/87	5.55	10.75	12.45	25.25			
\$0-49475,99985-8,10-1187	11/10/87	4.88	13.25	2.85	11.95			

TABLE B-1

ADMINISTRATION BUILDING AREA IRA SOILS RESULTS - ANIONS

. OG IMYON	DAME	CONCENTRATION (UG/G)					
LOCATION	DATE SAMPLED	CHLORIDE	FLOURIDE	NITRATE	SULFATE		
SO-49000,100180-0,2-1187	11/11/87	1.07	12.44	2.68	91.62		
SO-49000,100180-2,4-1187	11/11/87	4.40	21.13	1.87	58.11		
SO-49000,100180-4,6-1187	11/11/87	1.53	17.60	2.84	28.30		
SO-49000,100180-6,8-1187	11/11/87	9.61	19.64	3.59	19.15		
SO-49000,100295-0,2-1187	11/11/87	1.93	9.66	4.52	33.57		
SO-49000,100295-2,4-1187	11/11/87	12.79	9.76	1.73	163.41		
SO-49000,100295-4,6-1187	11/11/87	19.39	12.10	1.40	27.73		
SO-49000,100295-6,8-1187	11/11/87	13.09	14.88	5.51	31.91		
SO-49000,100500-0,2-1187	11/12/87	1.60	9.72	2.63	39.47		
SO-49000,100500-10,12-1187	11/12/87	1.08	7.20	3.48	8.16		
SO-49000,100500-2,4-1187	11/12/87	0.46	10.04	1.15	60.38		
SO-49000,100500-4,6-1187	11/12/87	0.61	5.99	0.98	37.38		
SO-49000,100500-6,8-1187	11/12/87	1.01	7.92	2.89	15.97		
SO-49000,100500-8,10-1187	11/12/87	1.09	7.88	1.33	12.13		
SO-49000,100665-0,2-1187	11/12/87	2.00	1.88	0.94	41.20		
SO-49000,100665-10,12-1187	11/12/87	5.35	13.75	2.04	8.91		
SO-49000,100665-2,4-1187	11/12/87	4.84	12.96	3.27	127.31		
SO-49000,100665-4,6-1187	11/12/87	2.63	15.65	1.43	136.20		
SO-49000,100665-6,8-1187	11/12/87	9.87	9.74	3.01	46.32		
\$0-49000,100665-8,10-1187	11/12/87	9.58	12.43	2.72	49.83		
SO-49000,99985-0,2-1187	11/11/87	2.94	9.28	ND	74.90		
SO-49000,99985-2,4-1187	11/11/87	25.74	8.80	7.72	327.44		
SO-49000, 99985-4, 6-1187	11/11/87	48.71	12.54	2.85	59.93		
SO-49000, 99985-6, 8-1187	11/11/87	5.45	16.25	1.81	20.97		
SO-49080,100295-0,2-1187	11/12/87	3.34	6.08	2.62	26.25		
SO-49080,100295-2,4-1187	11/12/87	2.32	12.30	1.16	76.84		
\$0-49080,100295-4,6-1187	11/12/87	1.07	13.48	0.54	3.53		
SO-49080,100295-6,8-1187	11/12/87	1.20	21.40	1.67	4.30		
SO-49080,100295-8,10-1187	11/12/87	0.84	12.86	2.88	8.05		
SO-49082,100570-0,2-1187	11/12/87	1.59	7.48	4.31	32.29		
SO-49082,100570-10,12-1187	11/12/87	45.3	2.98	6.2	21.34		
SO-49082,100570-12,14-1187	11/12/87	1.17	11.15	5.4	17.36		
SO-49082,100570-14,16-1187	11/12/87	0.94	6.95	2.47	13.66		
SO-49082,100570-2,4-1187	11/12/87	3.09	9.15	1.26	140.7		
SO-49082,100570-4,6-1187	11/12/87	4.93	6.11	1.29	106.91		
SO-49082,100570-6,8-1187	11/12/87	5.77	8.85	8.2	25.64		
SO-49082,100570-8,10-1187	11/12/87	5.17	7.87	2.95	32.61		
SO-49101,100500-0,2-1187	11/12/87	1.3	9.53	3.58	34.24		
\$0-49101,100500-10,12-1187	11/12/87	1.43	1.55	1202.49	8.61		
\$0-49101,100500-12,14-1187	11/12/87	1.58	1.58	1108.81	12.92		
SO-49101,100500-14,16-1187	11/12/87	1.32	1.44	53.9	11.5		
SO-49101,100500-2,4-1187	11/12/87	0.93	ND	141.32	88.38		
SO-49101,100500-4,6-1187	11/12/87	1.11	1.11	1285.56	7.02		
SO-49101,100500-6,8-1187	11/12/87	1.58	1.21	1354.64	6.68		

TABLE B-1 (continued)

ADMINISTRATION BUILDING AREA IRA SOILS RESULTS - ANIONS

LOGARION	Dimp		CONCENTRA	ATION (UG/	G)
LOCATION	DATE SAMPLED	CHLORIDE	FLOURIDE	NITRATE	SULFATE
SO-49101,100500-8,10-1187	11/12/87	1.49	1.49	1297.08	6.81
SO-49132,100440-0,2-1187	11/12/87	1.40	4.55	3.27	77.56
SO-49132,100440-10,12-1187	11/12/87	0.85	8.41	2.68	15.48
SO-49132,100440-12,14-1187	11/12/87	0.83	5.13	2.62	13.23
SO-49132,100440-14,16-1187	11/12/87	54.93	2.52	3.49	18.75
SO-49132,100440-2,4-1187	11/12/87	0.83	5.01	1.07	129.93
SO-49132,100440-4,6-1187	11/12/87	0.95	11.19	1.9	16.79
SO-49132,100440-6,8-1187	11/12/87	0.7	12.39	3.39	13.32
SO-49132,100440-8,10-1187	11/12/87	1.06	9.78	8.72	11.19
SO-49160,100500-0,2-1187	11/12/87	3.54	10.06	1.49	46.31
SO-49160,100500-10,12-1187	11/12/87	4.86	10.94	4.13	6.69
SO-49160,100500-2,4-1187	11/12/87	3.87	10.89	2.42	35.93
SO-49160,100500-4,6-1187	11/12/87	5.97	15.45	2.11	5.97
SO-49160,100500-6,8-1187	11/12/87	4.87	14.71	3.68	5.46
SO-49160,100500-8,10-1187	11/12/87	4.40	13.56	2.62	5.35
SO-49172,100180-0,2-1187	11/11/87	9.77	13.49	2.76	170.77
SO-49172,100180-2,4-1187	11/11/87	25.59	12.19	2.66	237.63
SO-49172,100180-4,6-1187	11/11/87	30.12	11.81	1.09	14.94
SO-49172,100180-6,8-1187	11/11/87	26.00	14.26	7.48	16.30
SO-49250,100140-0,2-1187	11/10/87	2.36	10.87	0.82	136.90
SO-49250,100140-2,4-1187	11/10/87	7.88	5.93	1.08	1548.14
SO-49250,100140-4,6-1187	11/10/87	22.58	11.72	4.00	17.94
SO-49250,100140-6,8-1187	11/10/87	1.60	11.66	1.14	30.86
SO-49250,100140-8,10-1187	11/10/87	0.86	12.92	1.80	14.34
\$0-49475,99985-0,2-1187	11/10/87	1.82	8,53	12.97	89.22
\$0-49475,99985-10,12-1187	11/10/87	4.07	19.39	3.78	15.42
SO-49475,99985-2,4-1187	11/10/87	1.81	5.69	9.86	116.34
SO-49475,99985-4,6-1187	11/10/87	4.17	8.57	2.25	43.04
\$0-49475,99985-6,8-1187	11/10/87	5.55	10.75	12.45	25.25
SO-49475,99985-8,10-1187	11/10/87	4.88	13.25	2.85	11.95

TABLE B-2

AMINISTRATION BUILDING AREA IRA SOILS RESULTS - METALS

	Zn	23.2	33.8	18.5	21.4	28.3	19.2	14.2	21.4	49.0	16.6	27.0	35.2	22.4	22.2	42.7	14.1	28.3	22	19.6	17.3	28.1	20.9	16.0	15.2	27.4	22.9	28.5	20.6	20.0	41.1	12	14.4
	۸	47.2	49.8	33.8	37.4	40.5	41.7	32.7	44.4	43.5	42.2	33.9	64.4	42.2	31.8	38.7	35.5	33	36	28.9	39.3	38.8	46.2	25.8	32.1	38.9	27.0	31.0	17.2	27.3	27.6	24.2	8.12
	I	R	2	R	오	2	Q	2	웆	2	2	2	욷	욷	문	문	물	욷	물	문	욷	문	2	문	R		욷	Q	2	2	2 :	2 9	2
	Na	욷	R	문	足	旲	욷	물	2	R	R	2	2	2	足	웆	웆	2	욷	웆	681	웆	707	욷	2	웆	딡	문	욷	R	2 !	2 !	₹
	Ag	R	R	물	욷	ę	R	욷	2	1.5	R	 	2.0	1.2	1.0	1.3	2	웆	2	2	욷	2	2	2	2	1.4	웆	물	2	2	오 !	2 !	3
	Se	2	2	욷	웆	Z	2	문	R	욷	문	旲	문	문	R	R	2	2	R	2	문	2	2	욷	2	2	2	문	문	2	2	₽!	2
	×	2	욷	2	웆	旲	2	2	足	952	물	욷	999		R	717	R	R	旲	2	2	2	2	2	2	2	2	682	욷	욷	오!	2 !	2
	Ni	15.6	14.3	10.5	15.7	15.0	11.7	9.2	15.4	27.2	12.0	12.5	18.7	10.9	13.5	21.1	15.3	18.6	9.7	14.7	7.9	13.2	13.0	8.5	9.5	13.6	11.5	15.2	12.3	16.5	17.7	14.4	11.3
	Hg	읒	문	욷	웆	읒	읒	물	R	0.1	웆	물	夂	ᄝ		R	2	욷	ᄝ	旲	2	웆	2	웆	읒	웆	문	운	R	R	2	2	2
	Æ	1440	86.4	74.4	104	1514	341	38.7	93.2	705	46	802	1747	9.99 9	30.9	534	46.1	610	685	437	50.7	227	404	66.5	62	8	78.8	81.4	55.1	238	459	73	70
	M	2857	3494	2166	2075	2086	2297	1715	2022	3615	1704	2101	3054	23.2	2031	2690	1690	2005	1691	1991	1935	2667	2527	2141	1772	2601	2234	2515	1961	1959	7680	1583	1842
	Li	6.5	11.2	2	Q	6.7	R	Đ	2	15.2	R	8.9	13.6	2	R	8.9	2	8.7	9.9	9.7	2	7.5	R	R	딡	7.0	2	2	R	R	6.2	2	2
(DC/C)	Pb I																														6.3		
	ът 9																							_			_		_		17188	_	
Soncentration	3	9.5	10.8	7.3	8.4	8.0	8.2	7.5	10.7	20.8	10.4	13.2	14.2	13.5	11.6	23	8.2	7.5	6.7	5.9	11.6	7.0	7.7	7.0	7.3	13.4	9.5	10.6	7.7	8.7	15.9	6.2	6.3
Conc	පි	7.6	물	R	R	5.5	17.3	2	욷	11.4	6.7	11.3	27.5	R	R	11.2	Ę	6.01	11.1	6.7	문	10.0	8.9	9.5	욷	문	문	8.7	2	23.8	∞	2	2
	អូ																														17		
	Ça	13627	4584	3667	3249	3602	3594	3115	3365	3472	3455	2113	3151	3809	3201	3396	3877	1611	1846	2563	4090	4670	4293	4006	3361	3770	2925	2815	3013	3371	2878	3195	4065
	g	R	2	£	R	0.7	2	R	물	1.5	2	6.0	1.2	8.0	9.0	2	2	웆	2	R	2	2	R	2	足	8.0	R	0.5	9.0	R	2	2	욷
٠	Be	1.2	-	0.9	0.9	0.9	0.9	0.8	1.2	_	8 1.0	0.7	1.2	0.8	0.7	1:1	8.0	0.8	0.7	9.0	0.8	0.9	1.0	0.7	6.0	0.8	8.0	1.2	1.2	1.4	0.8	0.8	1.2
	Ba	248	88	95.2	8	234	127	223	59.0	253	57.88	195	299	88	59.1	142	78	229	164	154	83	157	261	441	150	169	112	113	110	163	220	88	40.9
	As	16.6	34	16.7	8.9	13.0	22.7	20.7	31.7	10.2	7.3	7.9	2.3	δ. 8.	10.0	29.4	10.8	9.5	11.1	10.9	13.8	9.7	27.6	14.5	13.2	7.6	7.1	3.9	4.4	8.1	16.5	16	8.1
	S	R	R	2	R	2	R	2	2	0.9	R	R	2		Q	Q	R	2	R	旲	욷	0.9	문	읒	문	R	2	R	R	7.6	오	문	R
	A1	14481	30008	13658	16070	11272	13665	11027	15873	14758	12046	10638	25260	16432	16139	14044	12147	10528	10717	13234	16192	11870	13528	11343	10260	16547	15103	18771	14060	14231	8681	12361	11897
1	Date Sampled	11/11/87	11/11/87	11/11/87	11/11/87	11/11/87	11/11/87	11/11/87	11/11/87	11/12/87	7 11/12/87	11/12/87	11/12/87	11/12/87	11/12/87	11/12/87	7 11/12/87	11/12/87	11/12/87	11/12/87	11/12/87	11/11/87	11/11/87	11/11/87	11/11/87	11/12/87	11/12/87	11/12/87	11/12/87	11/12/87	_	7 11/12/87	7 11/12/87
	Location	50-49000,100180-0,2-1187	50-49000,100180-2,4-1187	50-49000, 100180-4, 6-1187	50-49000, 100180-6, 8-1187	\$0-49000,100295-0,2-1187	50-49000, 100295-2, 4-1187	50-49000, 100295-4, 6-1187	50-49000,100295-6,8-1187	50-49000,100500-0,2-1187	S0-49000, 100500-10, 12-1187	50-49000,100500-2,4-1187	50-49000, 100500-4, 6-1187	50-49000, 100500-6, 8-1187	50-49000, 100500-8, 10-1187	50-49000,100665-0,2-1187	50-49000,100665-10,12-1187	50-49000,100665-2,4-1187	50-49000, 100665-4, 6-1187	50-49000,100665-6,8-1187	50-49000,100665-8,10-1187	50-49000, 99985-0, 2-1187	50-49000, 99985-2, 4-1187	S0-49000, 99985-4, 6-1187	50-49000, 99985-6, 8-1187	50-49080, 100295-0, 2-1187	SO-49080, 100295-2, 4-1187	50-49080, 100295-4, 6-1187	50-49080, 100295-6, 8-1187	50-49080, 100295-8, 10-1187	50-49082,100570-0,2-1187	50-49082,100570-10,12-1187	SO-49082, 100570-12, 14-1187

TABLE B-2 (continued)

ADMINISTRATION BUILDING ARRA IRA SOILS RESULTS - METALS

	Zn	19.4 25.4 17.7 18.7 18.3 18.3 19.6 19.6 19.6 19.6 19.6 19.6 19.6 19.6
	۸	19.3 34.2 34.2 34.2 31.8 31.8 31.8 31.8 32.9 32.6 4.2 32.6 32.6 32.6 4.2 32.6 32.6 32.6 32.6 32.6 32.6 32.6 32
	Ţ	
	Na	
	Ag	1.3 A B B B B B B B B B B B B B B B B B B
	Se	
	K	
	Ni	23.5 14.2 35.5 11.2 15.6 19.8 13.3 13.3 13.3 10.0 10.0 10.4 10.3 10.4 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8
	Bg	8 8 8 8 8 8 8 8 8 8 8 8 8 8 8 8 8 8 8
	Æ	64 611 246 49.1 32.6 49.1 32.6 65.5 814 43.3 48.8 173.5 805 173 805 173 805 173 805 173 173 173 173 173 173 173 173 173 173
	₩ Wg	1895 1830 2074 2025 1846 2167 1978 1994 1994 11994 1104 1104 1104 1104 110
	Li	86.9 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8 10.8
	TP	4.2 12.7 17.7 17.7 17.7 17.7 17.7 17.3 17.3 17
	He H	11065 113910 15543 15895 20020 13738 11563 8130 18407 11353 14507 11306 20505 14722 25155 6217 11370 11370 11370 11419 9511 11328 20970 113285 20970 113285 20970 113285 20970 113285 20970 113285 20970 113285 113285 20970 113285 20970 113285 20970 113285 20970 113285 20970 113285 20970 113285 20970 113285 20970 113285 20970 113285 20970 113285 20970 113285 20970 113285 20970 113285 20970 113285 20970 113285 20970 113285 20970 113285 20970 113285 20970 113285 20970 113285 20970 113285 20970 113285 20970 113285 20970 113285 20970 113285 20970 113285 20970 113285 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 11338 20970 113385 20970 113385 20970 113385 20970 113385 20970 11338 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 113385 20970 20970 20970 20970 20970 20970 20970 20970 2097
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(9/90)	පි	14.2 9.3 9.3 9.3 9.3 9.3 9.3 9.3 9.3 9.9 8.9 8.9 8.9 8.9 8.9 8.9 8.9 8.9 8.9
ation	ಟ	34 664 664 222.4 222.4 117.6 117.6 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 117.5 1
Concentration	Ca	3932 1865 2532 3820 3814 2958 3407 3931 344 3522 344 3522 344 3523 3046 3015 3010 3010 3012 3016 3012 3016 3016 3016 3017 3016 3017 3017 3017 3017 3018 3018 3018 3018 3018 3018 3018 3018
	g	
	Be	11.3 11.2 11.2 11.2 11.2 11.3 11.3 11.3
	Ba	39.8 1175 128 166.4 145 43.4 37.8 36 38.1 36 38.1 36 38.1 178 56.3 178 67 67 67 67 67 178 82 67 67 178 82 67 178 82 67 178 82 67 178 82 67 178 82 67 178 82 67 178 82 82 67 178 83 178 83 178 83 178 83 178 83 178 83 178 83 178 83 178 83 178 83 178 83 178 83 178 83 178 83 178 83 178 83 178 83 178 83 178 83 178 83 178 83 178 83 178 83 178 83 178 83 178 83 178 83 178 178 178 178 178 178 178 178 178 178
	As	10.6 13.6 13.6 13.6 13.6 13.6 14.7 14.9 14.9 14.4 14.9 14.9 16.2 16.2 16.2 16.2 16.2 16.2 16.2 16.2
ľ	eS S	
	A1	13055 7401 13687 13101 13101 13101 13101 13101 13101 13101 13101 13101 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 13100 1
	Date Sampled	11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1 11/12/87 1
	Location S	\$0-49082, 100570-14, 16-1187 1 \$0-49082, 100570-2, 4-1187 1 \$0-49082, 100570-4, 6-1187 1 \$0-49082, 100570-6, 8-1187 1 \$0-49082, 100570-8, 10-1187 1 \$0-49082, 100500-0, 2-1187 1 \$0-49101, 100500-12, 14-1187 1 \$0-49101, 100500-14, 16-1187 1 \$0-49101, 100500-14, 16-1187 1 \$0-49101, 100500-14, 16-1187 1 \$0-49101, 100500-14, 16-1187 1 \$0-49132, 100440-12, 14-1187 1 \$0-49132, 100440-14, 16-1187 1 \$0-49132, 100440-14, 16-1187 1 \$0-49132, 100440-14, 16-1187 1 \$0-49132, 100440-6, 8-1187 1 \$0-49132, 100440-6, 8-1187 1 \$0-49132, 100500-0, 2-1187 1 \$0-4916, 100500-0, 2-1187 1 \$0-4916, 100500-1, 1-1187 1 \$0-4916, 100500-1, 1-1187 1 \$0-4916, 100500-1, 1-1187 1 \$0-4916, 100500-1, 1-1187 1 \$0-49172, 100180-0, 2-1187 1 \$0-49172, 100180-2, 4-1187 1 \$0-49172, 100180-6, 8-1187 1 \$0-49172, 100180-6, 8-1187 1 \$0-49250, 100140-2, 4-1187 1 \$0-49250, 100140-2, 4-1187 1 \$0-49250, 100140-2, 4-1187 1 \$0-49250, 100140-2, 4-1187 1

TABLE B-2 (continued)

ADMINISTRATION BUILDING ARRA IRA SOILS RESULTS - METALS

	Date									ت	oncentr	Concentration (UG/G)	(9/90)												
Location	Sampled	N1	SS	Às	Ва	Be	8	Ca	ភូ	9	r Cr	년 9	Q.	ij	5g W	Ž	fg	Ni	×	જુ	Åg	Na I	T1 V	.7	Zn
S0-49250.100140-6.8-1187	11/10/87	12953	Ę	8 2	123	6	. 5	2916	15.6	Ę	α 4	ı	7.7	Ę	1613	, 5		1	Į į	Ę	l	1 5	3,4		-
50-49250, 100140-8, 10-1187	11/10/87	10344	2	14.1	136	1.3	2	2862	14.8	9 0	9.9	17504	26.5	2	1429	233	2 2	10.1	2	2 2	2 2	2	38.5		12.3
SO-49475, 99985-0, 2-1187	11/10/87	14329	2	23.4	184		8.0	11523	21.8	12.5	17.9		17	8.8	4364	771	•	•	%	呈			ND 42.		2
SO-49475, 99985-10, 12-1187	11/10/87	12009	2	6	64		2	3462	14.4	오	7.8		4.9	2	1442	57.2			2	2					3
S0-49475, 99985-2, 4-1187	11/10/87	13701	2	16.6	246		9.0	18408	21.4	16.2	15.4		26.5	6.9	2405]	176			2	2					ī.
S0-49475, 99985-4, 6-1187	11/10/87	11682	2	16.8	22.3		0.7	15788	20.3	10.1	22.1		76	8.9	2499	304			2	R					5
50-49475, 99985-6, 8-1187	11/10/87	8271	R	7.1	230	_	R	2945	20.1	R	11.4		5.8	Q	1545	63			2	2					٣.
SO-49475, 99985-8, 10-1187	11/10/87	12095	욷	8.2	92.4	6.0	R	3333	14.8	7.1	5.5		8.8		1435	88			R						7.5

TABLE B-3
Ash Pond Dike IRA Soils Results - Anions

LOGIMION	DAME		CONC	ENTRATION	(UG/G)
LOCATION	DATE SAMPLED	CHLORIDE	FLOURIDE	NITRATE	SULFATE
SO-51100,100335-0,2-1187	11/09/87	3,71	5.95	6.76	26.76
SO-51100,100335-2,4-1187	11/09/87	4.09	1.75	10.67	79.55
SO-51100,100335-4,6-1187	11/09/87	3.93	9.27	2.29	26.67
SO-51100,100335-6,8-1187	11/09/87	4.17	7.71	1.87	23.81
SO-51100,100335-8,10-1187	11/09/87	3.37	9.22	4.42	11.5
SO-51125,100260-0,2-1187	11/09/87	4.67	1.29	1.57	21.64
SO-51125,100260-2,4-1187	11/09/87	0.99	1.5	1.35	19.47
SO-51125,100260-4,6-1187	11/09/87	4.01	5.35	1.76	17.97
SO-51125,100260-6,8-1187	11/09/87	4.97	5.52	96.47	12.29
SO-51125,100260-8,10-1187	11/09/87	4.14	10.38	5.3	7.48
SO-51150,100220-0,2-1187	11/09/87	5.71	4.82	2.20	42.09
SO-51150,100220-2, 4 -1187	11/09/87	2.74	4.85	5.53	26.81
SO-51150,100220-4,6-1187	11/09/87	4.16	7.29	3.99	25.66
SO-51150,100220-6,8-1187	11/09/87	4.36	8.98	2.13	25.55
SO-51150,100220-8,10-1187	11/09/87	4.44	7.62	8.86	26.05
SO-51180,100335-0,2-1187	11/09/87	2.39	8.05	1.66	123.5
SO-51180,100335-2,4-1187	11/09/87	3.51	9.08	2.89	150.9
SO-51308,100085-0,2-1187	11/09/87	5.58	4.39	4.46	17.98
\$0-51308,100085-2,4-1187	11/09/87	1.55	ND	0.89	58.63
\$0-51308,100085-4,6-1187	11/09/87	3.53	1.87	1.74	37.97
\$0-51308,100085-6,8-1187	11/09/87	1.53	ND	3.38	97.77
\$0-51308,100085-8,10-1187	11/09/87	1.48	ND	18.23	62.20

TABLE B-4

Ash Pond Dike IRA Soils Results - Metals

DOMING MAPICAD MAPIC		E C									Ŝ	CONCENTRATION (UG/G)	NOI (UG	(9/								1
11/09/87 1234 ND 21.2 284 1 1.5 3945 20.6 15.6 14.3 20464 19.6 7.5 2290 1546 ND 17.3 805 ND 1.27 ND ND 11/09/87 12194 ND 5 162 1.2 0.9 2280 13.4 1.3 11.9 18181 1.3 2090 860 ND 15.8 600 ND ND ND ND ND 11/09/87 12164 ND 5 162 1.2 0.9 2381 15.7 ND 8.3 16991 12.7 ND 1860 110 ND 8.1 ND ND ND ND ND ND 11/09/87 12164 ND 21.1 175 0.9 0.8 2561 21.2 17.4 20.6 32122 16.4 18.1 2829 393 933 945 945 948 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 949 9	NO	SAMPLED	Al	S.	As	Ba	Be	g	Ca	r S	පි		Fe	<u> </u>	þ,				! !		>	ı
11/09/87 12294 ND 7:6 192 0.7 2260 18.4 12.3 11.9 18181 18.1 7.3 2090 860 ND 15.8 600 ND	100335-0,2-1187	11/09/87	14142	2	21.2	784		1.5								l		ŀ		i		1
11/09/87 2184 W 5 162 1.2 0.9 3281 24.5 36 9.8 20037 18.2 9.2 2473 865 M 14.3 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0 M 0	100335-2,4-1187	11/09/87	12294	R	7.6	192	0.7	0.7														
11/09/87 12164 ND 8 105 0.9 0.8 3311 15.7 ND 8.3 16991 12.7 ND 1860 110 ND 8.1 ND ND ND ND ND ND ND N	100335-4,6-1187	11/09/87	21184	R	5	162	1.2	0.9														
11/09/87 12644 ND 0.6 2.06 3 0.8 5261 21.2 17.4 20.6 32122 16.4 18.1 2829 933 ND 67.2 ND ND ND ND ND ND ND N	100335-6,8-1187	11/09/87	12164	足	80	105	6.0	8.0														
11/09/87 12164 ND 21.1 175 0.9 0.8 1655 178 18.2 11.2 19474 20.4 6.9 1860 1345 ND 11.5 ND ND ND ND ND ND ND N	,100335-8,10-1187	11/09/87	15849	웆	10.6	206	က	. 8.0														
11/09/87 11004 ND 18.7 120 0.8 ND 1579 18.6 6.9 8.9 16868 6.3 ND 1454 497 ND 11.5 ND ND ND ND ND ND ND N	,100260-0,2-1187	11/09/87	12164	문	21.1	175	6.0	8.0														
11/09/87 125244 ND 11.8 103 1.1 0.8 5322 25 7.8 15.4 30548 20.9 ND 2769 161 ND 35.9 ND ND ND ND ND ND ND ND ND 11/09/87 12591 ND 22.2 108 2.8 0.7 3321 19 8.3 19.2 2013 7.9 ND 2061 352 ND 41.9 ND ND ND ND ND ND ND ND ND 11/09/87 12591 ND 22.2 108 2.8 0.7 3321 19 8.3 19.2 2013 7.9 ND 2061 352 ND 41.9 ND ND ND ND ND ND ND ND 11/09/87 12019 ND 10.8 208 1 ND 2501 17.1 9.5 12.4 18219 12.2 ND 2175 633 ND 14.2 ND 11/09/87 13236 ND 25.5 81 0.9 ND 3356 31 ND 8.5 1862 17.4 18219 12.2 ND 1588 53.5 ND 14.2 ND ND ND ND ND ND ND ND ND 11/09/87 13236 ND 25.5 8 1 0.9 ND 2996 15. ND 12.4 17502 11.4 ND 1687 78.1 ND ND ND ND ND ND ND ND ND 11/09/87 13859 ND 7.2 188 1 0.9 ND 12.4 17502 11.4 ND 1687 78.1 ND ND ND ND ND ND ND ND 11/09/87 13859 ND 4.6 2363 19.5 10 12.3 ND 12.4 ND 14.2 ND 14.2 ND ND ND ND ND ND ND 11/09/87 13859 ND 4.6 2363 19.5 10 12.3 ND 12.4 ND 14.8 ND ND ND ND ND ND ND ND 11/09/87 1105/87 1105/87 1105/87 1105/87 1105/87 1105/87 1105/87 1105/87 1105/87 1105/87 1105/87 1105/87 1105/87 1105/87 1105/87 1105/87 1105/87 1105/87 1105/87 1105/87 1105/87 1105/87 1105/87 1105/87 1105/87 1105/87 1105/87 1105/87 1105/87 1105/87 1105/87 1105/87 1105/87 1105/87 1105/87 1105/87 1105/87 ND 14.2 ND	,100260-2,4-1187	11/09/87	11004	웆	18.7	120	8.0	2														
11/09/87 1976 ND 20.9 1.6 0.8 5222 2.5 7.8 15.4 30548 20.9 ND 2769 161 ND 365.9 ND 365.0 1.0 3.321 1.9 1.9 1.0 1.2 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1.0 1	,100260-4,6-1187	11/09/87	25244	Q	11.8	103	1.1	8.0														
11/09/87 12591 ND 22.2 108 2.8 0.7 3321 19 8.3 19.2 20113 7.9 ND 2061 352 ND 41.9 ND ND ND ND ND ND 11/09/87 12019 ND 10.8 260 1 ND 2501 17.1 9.5 12.4 18219 12.2 ND 2175 633 ND 14.2 ND ND ND ND ND ND 11/09/87 12019 ND 5.6 85 0.7 ND 3663 23.7 ND 6.5 1922 4 ND 1588 53.5 ND 14.2 ND ND ND ND ND ND 11/09/87 13236 ND 25.5 81 0.9 ND 2970 16.5 ND 8.7 1780 6.66 1623 37 ND 17 ND ND ND ND ND ND 11/09/87 13859 ND 7.2 188 1 0.9 ND 2996 15 ND 12.4 1750 6.66 1623 37 ND 18.7 ND 19.5 ND ND ND ND ND ND 11/09/87 13859 ND 7.2 188 1 0.9 ND 296 15 ND 12.4 1750 8.5 2437 440 ND 14.2 ND ND ND ND ND ND ND 11/09/87 11957 ND 5.8 31 0.7 ND 1073 9.7 ND 12.7 16776 8.5 2437 440 ND 14.2 ND ND ND ND ND ND 11/09/87 13246 ND 5.8 31 0.7 ND 1073 9.7 ND 9.1 11/09/87 13261 ND 5.8 31 0.7 ND 1073 9.7 ND 9.1 11/09/87 13261 ND 13.4 48 1.3 0.7 ND 13.4 12.4 5.9 17159 8.6 ND 11/09/87 13261 ND 13.4 48 1.3 0.7 ND 13.4 12.4 5.9 17159 8.6 ND 11/09/87 13261 ND 13.4 ND 13.4 12.4 5.9 17159 8.6 ND 11/109/87 13261 ND 13.4 ND 13.4 12.4 5.9 17159 8.6 ND 11/109/87 13261 ND 13.4 ND 13.4 ND 13.4 12.4 5.9 17159 8.6 ND 11/109/87 13261 ND 13.4 ND 13.4 12.4 5.9 17.3 ND 13.4 12.4 ND 13.1 220 ND 5.8 ND 0.1 ND	,100260-6,8-1187	11/09/87	19761	2	20.9	1.6	8.0	5222														
11/09/87 12678 ND 5.6 85 0.7 ND 2501 17.1 9.5 12.4 18219 12.2 ND 2175 633 ND 14.2 ND ND ND ND ND ND ND 11/09/87 12678 ND 5.6 85 0.7 ND 2356 31 ND 8.5 12.6 ND 8.5 12.6 ND 15.8 85 13.5 ND 9.6 ND	,100260-8,10-1187	11/09/87	12591	문	22.2	108	2.8	0.7									-				-	
11/09/87 12678 ND 5.6 85 0.7 ND 3063 23.7 ND 6.5 19623 4 ND 1588 53.5 ND 9.6 ND ND ND ND ND 44.6 11/09/87 13236 ND 25.7 66 ND ND 2970 16.5 ND 8.7 17780 6.66 1623 37 ND 178 ND	,100220-0,2-1187	11/09/87	12019	웆	10.8	208	-	Q													-	
11/09/87 19028 ND 25.5 81 0.9 ND 3356 31 ND 8.5 22677 4.7 2059 48 ND 15 ND ND ND ND ND ND ND 71.4 11/09/87 13236 ND 25.7 66 ND ND 2970 16.5 ND 8.7 17780 6.66 1623 37 ND 17 ND	,100220-2,4-1187	11/09/87	12678	문	5.6	88	0.7	2	-							_					-	
11/09/87 13236 ND 25.7 66 ND ND 2996 15 ND 12.4 17502 11.4 ND 1687 78.1 ND 17.8 ND ND ND ND ND ND ND ND 17.4 11/09/87 11947 ND 19.8 69 0.9 ND 2996 15 ND 12.4 17502 11.4 ND 1687 78.1 ND 19.5 ND ND ND ND ND ND 11/09/87 11957 ND 4.6 239 1 0.6 2442 20 10.8 13.4 18111 8.2 11.7 2627 342 ND 14.2 ND ND ND ND ND ND 11/09/87 11957 ND 4.6 239 1 0.6 2363 19.5 10 12.7 16776 8.5 8.5 2437 440 ND 14.2 ND ND ND ND ND ND ND 11/09/87 11767 ND 8.6 131 0.7 ND 1073 9.7 ND 9.1 1518 8.7 ND 615 138 ND 5.6 ND	,100220-4,6-1187	11/09/87	19028	ᄝ	25.5	81	6.0	Q												•	-	
11/09/87 11947 ND 19.8 69 0.9 ND 2996 15 ND 12.4 17502 11.4 ND 1687 78.1 ND 19.5 ND ND ND ND ND ND ND ND 11/09/87 13859 ND 7.2 188 1 0.6 2642 20 10.8 13.4 18111 8.2 11.7 2627 342 ND 19.8 711 ND ND ND ND 11/09/87 11957 ND 4.6 239 1 0.6 2363 19.5 10 12.7 16776 8.5 8.5 2437 440 ND 14.2 ND ND ND ND ND ND ND 11/09/87 11767 ND 8.65 131 0.7 ND 1073 9.7 ND 9.8 13542 14 ND 1317 340 ND 9.2 ND	,100220-6,8-1187	11/09/87	13236	읒	25.7	99	R	R												•		
11/09/87 13859 ND 7.2 188 1 0.6 2642 20 10.8 13.4 18111 8.2 11.7 2627 342 ND 19.8 711 ND ND ND ND 11/09/87 11957 ND 4.6 239 1 0.6 2363 19.5 10 12.7 16776 8.5 8.5 2437 440 ND 14.2 ND ND ND ND ND ND 11/09/87 11767 ND 8.65 131 0.7 0.6 3350 15.9 ND 9.8 13542 14 ND 1317 340 ND 9.2 ND ND ND ND ND ND 11/09/87 11246 ND 9.01 84 1.2 0.6 2844 15.9 12.3 8.9 9690 9.7 ND 1448 579 ND 9.7 ND	,100220-8,10-1187	11/09/87	11947	문	19.8	69	6.0	R														
11/09/87 11957 ND 4.6 239 1 0.6 2363 19.5 10 12.7 16776 8.5 8.5 2437 440 ND 14.2 ND ND ND ND ND ND 11/09/87 11767 ND 8.65 131 0.7 0.6 3350 15.9 ND 9.8 13542 14 ND 1317 340 ND 9.2 ND ND ND ND ND 11/09/87 11246 ND 9.01 84 1.2 0.6 2844 15.9 12.3 8.9 9690 9.7 ND 1448 579 ND 9.7 ND	,100335-0,2-1187	11/09/87	13859	R	7.2	188	-	9.0														
11/09/87 11767 ND 8.65 131 0.7 0.6 3350 15.9 ND 9.8 13542 14 ND 1317 340 ND 9.2 ND ND ND ND ND ND ND 11/09/87 1343 ND 5.8 31 0.7 ND 1073 9.7 ND 9.1 5198 8.7 ND 615 138 ND 5.6 ND ND ND ND ND ND 11/09/87 11246 ND 9.01 84 1.2 0.6 2844 15.9 12.3 8.9 9690 9.7 ND 1448 579 ND 9.7 ND	,100335-2,4-1187	11/09/87	11957	£	4.6	239	_	9.0				•										
11/09/87 8343 ND 5.8 31 0.7 ND 1073 9.7 ND 9.1 5198 8.7 ND 615 138 ND 5.6 ND ND ND ND ND ND 11/09/87 11246 ND 9.01 84 1.2 0.6 2844 15.9 12.3 8.9 9690 9.7 ND 1448 579 ND 9.7 ND	,100085-0,2-1187	11/09/87	11767	윷	8.65	131	0.7	9.0														
11/09/87 11246 ND 9.01 84 1.2 0.6 2844 15.9 12.3 8.9 9690 9.7 ND 1448 579 ND 9.7 ND ND ND ND ND ND ND 11/09/87 13261 ND 13.4 48 1.3 0.6 1266 17.8 13 9.3 22249 11.7 ND 1015 283 ND 8.4 ND	,100085-2,4-1187	11/09/87	8343	욧	5.8	31	0.7	包														
11/09/87 13261 ND 13.4 48 1.3 0.6 1266 17.8 13 9.3 22249 11.7 ND 1015 283 ND 8.4 ND	,100085-4,6-1187	11/09/87	11246	R	9.01	84	1.2	9.0														
11/09/87 11840 ND 11.53 39 0.7 ND ND 13.4 12.4 5.9 17159 8.6 ND 711 220 ND 5.8 ND ND ND ND ND ND	,100085-6,8-1187	11/09/87	13261	R	13.4	48	1.3	9.0				•									-	
	,100085-8,10-1187	11/09/87	11840	물	11.53	33	0.7	R				•										

TABLE B-5

Material Staging Area IRA Soils Results - Anions

	.		Concentra	ation (UG/	G)
Location	Date Sampled	Chloride	Flouride	Nitrate	Sulfate
50-51137,101068-0,2-1187	11/06/87	ND	2.79	0.85	15.31
80-51137,101068-10,12-1187	11/06/87	ND	9.49	2.43	17.89
50-51137,101068-2,4-1187	11/06/87	ND	7.92	0.79	82
50-51137,101068-4,6-1187	11/06/87	ND	6.34	1.27	46.92
50-51137,101068-6,8-1187	11/06/87	ND	8.11	1.33	38.49
50-51137,101068-8,10-1187	11/06/87	3.42	7.94	1.83	29.33
50-51150,101207-0,2-1187	11/06/87	ND	2.1	1.46	15.72
50-51150,101207-2,4-1187	11/06/87	ND	6.69	0.89	148.32
50-51150,101207-4,6-1187	11/06/87	5.13	5.13	0.89	90.26
80-51150,101207-6,8-1187	11/06/87	ND	7.97	1.57	24.83
50-51150,101207-8,10-1187	11/06/87	ND	8.83	1.39	17.65
50-51360,101175-0,2-1187	11/06/87	ND	3.42	0.94	97.85
50-51360,101175-2,4-1187	11/06/87	ND	8.56	0.79	78.29
50-51360,101175-4,6-1187	11/06/87	ND	4.96	0.69	79.35
50-51360,101175-6,8-1187	11/06/87	5.43	4.72	1.18	52.31
50-51360,101175-8,10-1187	11/06/87	ND	5.45	1.95	43.51
80-51445,101065-0,2-1187	11/06/87	ND	4.68	2.22	18.5
50-51445,101065-10,12-1187	11/06/87	1.25	4.53	ND	8.39
80-51445,101065-2,4-1187	11/06/87	5.04	12.65	1.68	124.26
50-51445,101065-4,6-1187	11/06/87	5.7	9.72	1.45	144.05
50-51445,101065-6,8-1187	11/06/87	5.51	6.72	1.32	52.88
50-51445,101065-8,10-1187	11/06/87	4.21	6.72	1.82	9.34
80-51500,101190-0,2-1187	11/06/87	2.04	ND	42.18	1.32
50-51500,101190-10,12-1187	11/06/87	1.17	5.16	ND	11.37
80-51500,101190-12,14-1187	11/06/87	1.27	5.54	ND	22.17
50-51500,101190-2,4-1187	11/06/87	4.36	6.94	1.79	53.59
50-51500,101190-4,6-1187	11/06/87	7.40	5.55	1.63	164.24
50-51500,101190-6,8-1187	11/06/87	5.59	3.46	1.56	33.96
50-51500,101190-8,10-1187	11/06/87	1.43	2.51	ND	26.31

Material Staging Area IRA Soils Results - Metals

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TABLE B-7

SOUTH EAST ISOLATION DIKE IRA SOILS RESULTS - ANIONS

LOCATION	DATE		CONCENTRA	TION (UC	(G)
LOCATION	DATE Sampled	CHLORIDE	FLOURIDE	NITRATE	SULFATE
SO-50140,98820-0,2-1187	11/10/87	4.64	13.98	1.68	61.51
SO-50140,98820-2,4-1187	11/10/87	4.85	17.75	2.05	51.2
SO-50140,98820-4,6-1187	11/10/87	4.55	18.91	1.86	22.39
SO-50140,98820-6,8-1187	11/10/87	5.69	14.82	1.8	20.3
SO-50160,98735-0,2-1187	11/10/87	2.86	4.47	1.74	93.3
SO-50160,98735-10,12-1187	11/10/87	4.36	1.33	1.44	20.9
SO-50160,98735-2,4-1187	11/10/87	3.52	3.34	1.44	23.1
SO-50160,98735-4,6-1187	11/10/87	1.33	ND	24.5	ND
SO-50160,98735-6,8-1187	11/10/87	1.19	1.29	0.63	32.5
SO-50160,98735-8,10-1187	11/10/87	0.945	1.84	0.73	26.1
SO-50230,98991-0,2-1187	11/10/87	5.36	5.14	3.15	13.82
SO-50230,98991-2,4-1187	11/10/87	3.33	13.42	1.56	113.97
SO-50230,98991-4,6-1187	11/10/87	3.57	12.67	1.89	26.74
SO-50230,98991-6,8-1187	11/10/87	3	12.24	2.4	20.56
SO-50252,98800-0,2-1187	11/10/87	3.74	9.51	3.03	26.89
SO-50252,98800-2,4-1187	11/10/87	4.27	1.51	2.71	68.49
SO-50252,98800-4,6-1187	11/10/87	2.62	1.67	1.4	93.04
SO-50252,98800-6,8-1187	11/10/87	5.92	4.59	2.21	15.45
SO-50290,98700-0,2-1187	11/09/87	4.77	5.85	1.99	87.23
SO-50290,98700-2,4-1187	11/09/87	4.6	6.67	2.25	514.76
SO-50290,98700-4,6-1187	11/09/87	4.82	8.38	1.65	263.77
SO-50290,98700-6,8-1187	11/09/87	5.19	7.81	6.71	11297

TABLE B-8

SOUTH EAST ISOLATION DIKE IRA SOILS RESULTS - METALS

DATE SAMPLED																	10100100					
	Al	gs Sp	As	Ba	Be	po	Ca	Cr.	8	õ	Fe	£.	Li	b y	Æ	ЭÉ	Ni	×	sz	Åg	Na	I
78/01/11	0330	9	7 9	H	,	-	46097	1	7 3	10 01	10384	ļ ģ	l	11575	304	Ę	10.2	Ę	E	2.1	Ę	€
11/10/87	14568	2 5	, rc	× 86			3816		С	17.2	23129	11.7		2622	107	2	25.8	2	2	2	足	2
11/10/87	8168	2	, 80 .3	314	1.7	6.0	3179	12.3	23.6	14.9	12496	13.2	2	1897	1218	2	39.3	R	2	2	문	2
11/10/87	10762	2	4.3	139	2.9	0.8	3383		7.8	23	20415	6.4		2187	455	2	36.9	2	욷	윤	R	R
11/10/87	10702	2	5.8	142	6.0	1.1	26698		12	97	17113	24.4		3251	8590	2	15.8	R	R	1.3	욷	2
50-50160, 98735-10, 12-1187 11/10/87	10590	R	6.7	8	0.9	Q	3153		6.4	13.1	16781	13.9		1425	163	2	12.3	욷	웆	2	문	2
11/10/87	6765	7.4	6.2	186		6.0	3788		18.9	8.9	23558	27.1		1950	3126	2	13.7	2	足	1.4	욷	웆
11/10/87	9052	R	7.5	129	8.0	8.0	1734		23.3	17	15067	22.8		1786	2026	2	16.5	旲	R	R	R	R
11/10/87	6951	R	3.6	108	0.7	R	823		7.8	21	10602	11.5		770	295	2	10.1	2	R	足	R	R
11/10/87	8105	7.8	7.2	366	-	6.0	1300		25.7	7.5	22596	25.1		3251	R	13	R	2	1.6	R	욷	62.4
11/10/87	20315	2	9.6	168	1.2	1	2156		9.9	20.5	24543	9.3		2075	292	2	19.7	912	R	R		2
11/10/87	13120	R	4.2	262	6.0	0.7	2140		8.2	35.1	17841	8.6		2650	555	욷	19.1	619	R	딮	욷	웆
11/10/87	10245	R	5.9	171.2	8.0	2	1900		5.9	19.6	12902	8.1		1872	259	2	10.6	S	2	2	R	욷
11/10/87	12409	Q	4.2	202	6.0	9.0	2727		10.2	97	15179	22.3		1884	470	2	8.6		2	욷	R	물
11/10/87	9191	2	6.7	159	6.0	R	6513		13.9	11.3	16542	21.1		1973	834	욷	16.2	2	R	2	욷	R
11/10/87	7166	R	4.5	121	8.0	2	1342		10.4	5.9	12996	18.6		305	979	2	6.7	딡	R	R	물	R
11/10/87	8037	R	6.4	101	6.0	9.0	2423			10.6	19895	12		1492	117	R	8.6		R	오	ę	2
11/10/87	16496	R	7.6	88	8.0	ᄝ	3210		21	11.5	16085	20		15.4	396	R	15.6	물	R	R	R	문
11/09/87	12552	Ð	5.4	139	6.0	딡	6425		10.8	10.2	14305	16.6		2132	533	웆	12.4	R	Q	웆	R	R
11/09/87	9293	B	6.3	183.6	6.0	6.0	18518		9.6	14.3	15893	16.7		5114	455	욷	16.4	9	웊	1.1	욷	웆
11/09/87	9988	문	7.2	98.9	1.2	9.0	4158		6	14	18989	20.5		2088	426		22	물	R	R	웆	물
11/09/87	10355	R	8.96	96.7	_	R	5508		9.3	10.9	14272	8.2		2124	240	2	18.2	R	R	2	R	R

TABLE B-9
CONSTRUCTION STAGING AREA IRA SOILS RESULTS - ANIONS

			CONCENTRATIO	N (UG/G)	
LOCATION	DATE	CHLORIDE	FLOURIDE	NITRATE	SULFATE
50-50800,98150-0,7-0288	02/09/88	15	10	1.5	75
80-50800,98150-8,15-0288	02/09/88	10	15	10	10
80-50950,98300-0,7-0288	02/09/88	30	10	5	75
80-50950,98300-8,15-0288	02/09/88	10	10	2.0	10

TABLE B-10 CONSTRUCTION STAGING ARRA IRA SOILS RESULTS - METALS

	DATE										8	CONCENTRATION (uq/L)	TION (nd/F)										
LOCATION	SAMPLED	A1	Al Sb As Ba	As	Ba		3	Ca	çr	ප	Çn	Fe	Pb	Li	Mg	Mn 1	Hg 1	Ni	se Cd Ca Cr Co Cu Fe Pb Li Mig Min Hig Ni K Se Ang Na Tl V	Ag	Na	T.	>	Zn
80-50800,98150-0,7-0288	02/09/88 8715 ND 14.0 125	8715	물	14.0	125	0.80	~ ₽	3750	12.9	10.5	8.45 1	2880 1	4.3	N 15	181	20	ND 13.	.2 42	ND 3750 12.9 10.5 8.45 12880 14.3 ND 1981 720 ND 13.2 427 2.93 ND 121 ND 24.6 26.6	2	121	NO 2	34.6	9.97
80-50950,98300-0,7-0288	02/09/88 10010	10010		12.71	ND 12.71 141 0.79		0.65	2210	15.6	5.97	11.8 1	0.65 2210 15.6 5.97 11.8 16160 11.1 ND 2040	1.1)Z	740 2	19 1	MD 12	o. 88	219 ND 12.0 385 1.09 ND 335 ND 29.2 28.1	2	335	ND 2	9.5	28.1

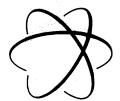
APPENDIX C ANALYTICAL DETECTION LIMITS

	DETECTION			
ANALYTICAL PARAMETER	UNITS	LIMITS	CATEGORY	
Alaminum	ug/g	20	Metals	
Aluminum	ug/g ug/g	6	Metals	
Antimony	ug/g ug/g	i	Metals	
Arsenic Barium	ug/g	20	Metals	
	ug/g	0.5	Metals	
Beryllium	ug/g	0.5	Metals	
Cadmium Calcium	ug/g	500	Metals	
Chromium	ug/g	1	Metals	
Cobalt	ug/g	5	Metals	
	ug/g	2.5	Metals	
Copper Iron	ug/g	10	Metals	
Lead	ug/g	0.5	Metals	
Lead Lithium	ug/g	5	Metals	
	ug/g	500	Metals	
Magnesium	ug/g	1.5	Metals	
Manganese	ug/g	0.1	Metals	
Mercury Nickel	ug/g	4	Metals	
Potassium	ug/g	500	Metals	
	ug/g	0.5	Metals	
Selenium	ug/g	1	Metals	
Silver	ug/g ug/g	500	Metals	
Sodium	ug/g ug/g	1	Metals	
Thallium	ug/g ug/g	5	Metals	
Vanadium Zinc	ug/g ug/g	2	Metals	
ZINC	46/6		_	
Nitrate	ug/g	0.5	Ions	
Sulfate	ug/g	1	Ions	
Chloride	ug/g	1.25	Ions	
Fluoride	ug/g	1.25	Ions	
2,4,6-TNT	ug/g	1.2	Nitroaromatics	
2,4 DNT	ug/g	0.75	Nitroaromatics	
2,6 DNT	ug/g	1.41	Nitroaromatics	
Nitrobenzene	ug/g	1.44	Nitroaromatics	
1,3,5-Trinitrobenzene	ug/g	0.57	Nitroaromatics	
1,3-Dinitrobenzene	ug/g	0.9	Nitroaromatics	
Percent Moisture	prent	_	Misc.	
pH	units	-	Misc.	
Pheno1	ug/kg	330	Semi-volatiles	
bis(2-Chlorethyl) ether	ug/kg	330	Semi-volatiles	
2-Chlorophenol	ug/kg	330	Semi-volatiles	
1,3-Dichlorobenzene	ug/kg	330	Semi-volatiles	
1,4-Dichlorobenzene	ug/kg	330	Semi-volatiles	
Benzyl Alcohol	ug/kg	330	Semi-volatiles	
1,2-Dichlorobenzene	ug/kg	330	Semi-volatiles	
	ug/kg ug/kg	330	Semi-volatiles	
2-Methylphenol bis(2-Chloroisopropyl) ether		330	Semi-volatiles	
	ug/kg	330	Semi-volatiles	
4-Methylphenol	~6/ * 6			

ANALYTICAL PARAMETER	UNITS	DETECTION LIMITS	CATEGORY
		222	
N-Nitro-Dipropylamine	ug/kg	330	Semi-volatiles
Hexachloroethane	ug/kg	330	Semi-volatiles
Nitrobenzene	ug/kg	330	Semi-volatiles
Isophorone	ug/kg	330	Semi-volatiles
2-Nitrophenol	ug/kg	1600	Semi-volatiles
2,4-Dimethyphenol	ug/kg	330	Semi-volatiles
Benzoic Acid	ug/kg	1600	Semi-volatiles
bis(2-Chloroethoxy) methane	ug/kg	330	Semi-volatiles
2,4-Dichlorophenol	ug/kg	330	Semi-volatiles
1,2,4-trichlorobenzene	ug/kg	330	Semi-volatiles
Naphthalene	ug/kg	330	Semi-volatiles
4-Chloroaniline	ug/kg	330	Semi-volatiles
Hexachlorobutadiene	ug/kg	330	Semi-volatiles
4-Chloro-3-methylphenol	ug/kg	330	Semi-volatiles
2-Methylnaphthalene	ug/kg	330	Semi-volatiles
Hexachlorocyclopentadiene	ug/kg	330	Semi-volatiles
2,4,6-Trichlorophenol	ug/kg	330	Semi-volatiles
2,4,5-Trichlorophenol	ug/kg	1600	Semi-volatiles
2-Chloronaphthalene	ug/kg	330	Semi-volatiles
2-Nitroaniline	ug/kg	1600	Semi-volatiles
Dimethyl Phthalate	ug/kg	330	Semi-volatiles
Acenaphthylene	ug/kg	330	Semi-volatiles
2,6-Dinitrotoluene	ug/kg	330	Semi-volatiles
3-Nitroaniline	ug/kg	1600	Semi-volatiles
Acenaphthene	ug/kg	330	Semi-volatiles
2,4-Dinitrophenol	ug/kg	1600	Semi-volatiles
4-Nitrophenol	ug/kg	1600	Semi-volatiles
Dibenzofuran	ug/kg	330	Semi-volatiles
2,4-Dinitrotoluene	ug/kg	330	Semi-volatiles
Diethylphthalate	ug/kg	330	Semi-volatiles
4-Chlorophenyl Phenyl Ether	ug/kg	330	Semi-volatiles
Fluorene	ug/kg	330	Semi-volatiles
4-Nitroaniline	ug/kg	1600	Semi-volatiles
4,6-Dinitro-2-methylphenol	ug/kg	1600	Semi-volatiles
N-nitrosodiphenylamine	ug/kg	330	Semi-volatiles
4-Bromophenyl Phenyl Ether	ug/kg	330	Semi-volatiles
Hexachlorobenzene	ug/kg	330	Semi-volatiles
Pentachlorophenol	ug/kg	1600	Semi-volatiles
Phenanthrene	ug/kg	330	Semi-volatiles
Anthracene	ug/kg	330	Semi-volatiles
Di-n-butylphthalate	ug/kg	330	Semi-volatiles
Fluoranthene	ug/kg	330	Semi-volatiles
Pyrene	ug/kg	330	Semi-volatiles
Butyl Benzyl Phthalate	ug/kg	330	Semi-volatiles
3,3'-Dichlorobenzidine	ug/kg	660	Semi-volatiles
Benzo(a)anthracene	ug/kg	330	Semi-volatiles
Chrysene	ug/kg	330	Semi-volatiles
bis(2-ethylhexyl)phthalate	ug/kg	330	Semi-volatiles
Di-n-octyl Phthalate	ug/kg	330	Semi-volatiles
Benzo(b)fluoranthene	ug/kg	330	Semi-volatiles

	DETECTION		
ANALYTICAL PARAMETER	UNITS	LIMITS	CATEGORY
Benzo(k)fluoranthene	ug/kg	330	Semi-volatiles
Benzo(a)pyrene	ug/kg	330	Semi-volatiles
Indeno(1,2,3-cd)pyrene	ug/kg	330	Semi-volatiles
Dibenzo(a,h)anthracene	ug/kg	330	Semi-volatiles
Benzo(g,h,i)perylene	ug/kg	330	Semi-volatiles
alpha-BHC	ug/kg	8	Pesticide/PCBs
beta-BHC	ug/kg	8	${\tt Pesticide/PCBs}$
delta-BHC	ug/kg	8	Pesticide/PCBs
gamma-BHC (Lindance)	ug/kg	8	${ t Pesticide/PCBs}$
Heptachlor	ug/kg	8	${ t Pesticide/PCBs}$
Aldrin	ug/kg	8	Pesticide/PCBs
Heptachlor Epoxide	ug/kg	8	${ t Pesticide/PCBs}$
Endosulfan I	ug/kg	8	Pesticide/PCBs
Dieldrin	ug/kg	16	Pesticide/PCBs
4,4'-DOE	ug/kg	16	Pesticide/PCBs
Endrin	ug/kg	16	${ t Pesticide/PCBs}$
Endosulfan II	ug/kg	16	Pesticide/PCBs
4,4'-DOD	ug/kg	16	${ t Pesticide/PCBs}$
Endosulfan Sulfate	ug/kg	16	Pesticide/PCBs
4,4'-DOT	ug/kg	16	Pesticide/PCBs
Endrin Ketone	ug/kg	16	Pesticide/PCBs
Methxychlor	ug/kg	80	Pesticide/PCBs
alpha-chlordane	ug/kg	80	Pesticide/PCBs
gamma-chlordane	ug/kg	80	Pesticide/PCBs
Toxaphene	ug/kg	160	Pesticide/PCBs
Aroclor-1016	ug/kg	80	Pesticide/PCBs
Aroclor-1221	ug/kg	80	Pesticide/PCBs
Aroclor-1232	ug/kg	80	Pesticide/PCBs
Aroclor-1242	ug/kg	80	Pesticide/PCBs
Aroclor-1248	ug/kg	80	Pesticide/PCBs
Aroclor-1254	ug/kg	160	Pesticide/PCBs
Aroclor-1260	ug/kg	160	Pesticide/PCBs

APPENDIX D
ANALYTICAL QUALITY CONTROL DATA SUMMARY



WELDON SPRING SITE REMEMDIAL ACTION PROJECT

Quality Control Report

CLIE	ENT: MK Ferguson	
PROJ SAMP	ECT #'s: 100-02 And 100-03 OLE #'s: AA05276 - AA05649 (All Soil Sumples)	
	GC/MS ANALYSIS CONFORMANCE SUMMARY	
1)	GC/MS TUNE SPECIFICATIONS	Ø
	a) BFB PASSED b) DFTPP PASSED	
2)	GC/MS TUNING FREQUENCY - PERFORMED PER METHOD EPA CLP	
3)	GC/MS CALIBRATION - INITIAL CALIBRATION CURVE OR CALIBRATION CHECK STANDARD RUN PER METHOD FOR - CLP	\bowtie
4)	GC/MS CALIBRATION REQUIREMENTS MET	\boxtimes
	a) CALIBRATION CHECK COMPOUNDSb) SYSTEM PERFORMANCE CHECK COMPOUNDS	
5)	a) VOA FRACTION metylene chloride 2.5 ug/l b) B/N FRACTION no contaminates c) A/E FRACTION no contaminates	
6)	SURROGATE RECOVERIES MEET CRITERIA (IF NOT MET, REFER TO INDIVIDUAL SURROGATE RECOVERY FORMS FOR ACTUAL RECOVERIES)	\bowtie
	a) VOA FRACTION All Surrogates within limits b) B/N FRACTION AA05276(2), AA5307(2), AA05553(2), AA056 c) A/E FRACTION AA05276(2), AA5307(2), AA05553(1), AA056	46(2) 46(2)
7)	SPIKED BLANK WITHIN CONTROL LIMITS Not Applicable	
8)	All Lines were met	X
9)	MINIMUM DETECTION LIMITS ON ALL FRACTIONS AT OR BELOW METHOD SPECIFICATIONS. (IF NOT CHECKED REFER TO INDIVIDUAL ANALYSIS REPORTS FOR THE ACTUAL MDL'S)	M
10	OTHERWISE DENOTED BELOW.	
ADI	DITIONAL COMMENTS:	-
	Richard Manny PROJECT MANAGER	- - -

CLIENT: MK Ferguson PROJECT #15: 100-02 and 100-03 (All soil samples)	
PROJECT #'s: 100-02 and 100-03 (All soil Samples) SAMPLE #'s: AAO5271-AAO5649	
GC/HPLC ANALYSIS CONFORMANCE SUMMARY	
1) GC/HPLC CALIBRATION - INITIAL CALIBRATION CURVE OR/CALIBRATION CHECK STANDARD RUN PER METHOD GC - EPA CLP / HPLC - USAT HAM	₹
2) BLANK CONTAMINATION - COMPOUNDS LISTED	
a) GC <u>no conteminates</u>	
b) HPLC no contaminates	
3) SPIKED BLANK WITHIN CONTROL LIMITS	
4) SAMPLE HOLDING TIMES MET All holding times met	Z.
5) MINIMUM DETECTION LIMITS ON GC/HPLC METHODS AT OR BELOW METHOD SPECIFICATIONS	ZĮ.
6) ALL SAMPLES CONFORM TO FPA - CLP QA/QC CRITERIA UNLESS () OTHERWISE DENOTED BELOW	Ø
ADDITIONAL COMMENTS:	
PROJECT MANAGER	

CLIEN PROJE SAMPI	NT: MK Ferouson ECT #'s: 100-02 100-03 LE #'s: AA 05271 - AA 0 5649 (Soil) METALS/INORGANIC ANALYSIS CONFORMANCE SUMMARY INITIAL CALIBRATION CURVE OR CALIBRATION CHECK STANDARD RUN	1
1)	PER METHOD FPA - CLP	
2)	BLANK CONTAMINATION - COMPOUNDS LISTED a) METALS No Contaminates	
	a) METALS No Concernates	
	a) METALS No Contaminates b) INORGANIC No Contaminates	Z.
3)	spiked blank (Laboratory Control Sample) within Control Limits (Sample Holding Times MET All holding times met	A
4)	SAMPLE HOLDING TITE	Z
5)	MINIMUM DETECTION LIMITS ON METALS/INORGANICS AT OR BELOW METHOD SPECIFICATIONS FOR CLP QA/QC CRITERIA UNLESS ALL SAMPLES CONFORM TO FOR DENOTED BELOW	Z
6)	ALL SAMPLES CONFORM TO OTHERWISE DENOTED BELOW	
IA	DDITIONAL COMMENTS:	
	PROJECT MANAGER	

CLI	ENT:	MK	Ferguson	
PRO SAM	JECT #' PLE #'s	's: :-	100-02 100-03 AAO 6554 - 7053 Soils 4-238	
			RADIOCHEMICAL ANALYSIS CONFORMANCE SUMMARY	
1)	GAS P	ROPC	ORTIONAL COUNTER	
		a)	BACKGROUND ACCEPTABLE ALPHA	
		b)	BACKGROUND ACCEPTABLE BETA	
		c)	PERFORMANCE CHECK ACCEPTABLE ALPHA	
		d)	PERFORMANCE CHECK ACCEPTABLE BETA	
2)	ALPHA	SPE	ECTROMETER	
		a)	BACKGROUND ACCEPTABLE	\square
		b)	CALIBRATION (KeV/CHANNEL) VERIFICATION	\boxtimes
3)	ALPHA	SC.	INTILLATION COUNTER	
		a)	BACKGROUND ACCEPTABLE	
		b)	PERFORMANCE CHECK ACCEPTABLE	
4)	METHO	D SI	PECIFIC PARAMETERS	
		a)	BLANK IN CONTROL	\boxtimes
		b)	SPIKED BLANK IN CONTROL	M
		c)	RPD FOR DUPLICATES IN CONTROL	区
ADE	OITIONA	L C	OMMENTS:	_
<u></u>			$\mathbb{Q} \cdot \mathbb{Q} \setminus \mathbb{Q}$	
			PROJECT MANAGER	_

(314) 298-8566

WSSRAP SOIL SPIKES MATRIX: SOIL UNITS: UG/6

SAMPLE NO: AA05283

SITE ID: S0-51360-101175-2,4-1187

	SAMPLE CONC	ADDED AMOUNT	SPIKE	PERCENT RECOVERY
NITRATE	0.7	2.14	1.5	84
CHLORIDE	(1.25	2,07	180	87
FLUORIDE	7.6	3.52	9.88	93
SULFATE	69.5	15.9	85.4	103

SAMPLE NO: AA05292

SITE ID: S0-51445-101065-10,12-1187

	SAMPLE CONC	ADDED AMOUNT	SPIKE	PERCENT RECOVERY
NITRATE	<0.5	2.5	2.36	94
CHLORIDE	1.1	2.25	3.32	102
FLUORIDE	4	2.75	6.8	98
SULFATE	7.4	3.5	10.89	115

SAMPLE NO: AA05637

SITE ID: S0-49160-100500-0,2-1187

	SAMPLE CONC	ADDED AMOUNT	SPIKE	PERCENT RECOVERY
NITRATE	1.3	2	3.21	91
CHLORIDE	3.1	2	5.04	84
FLUORIDE	8.8	3.75	12.59	103
SULFATE	40.5	10	50.5	100

■ 13715 Rider Trail North

SAMPLE NO: AA05335

SITE ID: S0-50230-98991-4,6-1187 (MS)

	SAMPLE CONC	ADDED AMOUNT	SPIKE	PERCENT RECOVERY
NITRATE	11.5	10	21.6	101
CHLORIDE	13.4	10	23.5	101
FLUORIDE	22.4	20	43.6	106
SULFATE	28.5	20	48.1	98

SAMPLE NO: AA05301

SITE ID: S0-51308-100085-2,4-1187

	SAMPLE CONC	ADDED AMOUNT	SPIKE	PERCENT RECOVERY
NITRATE	0.75	10	10.15	94
CHLORIDE	1.31	10	9.41	81
FLUORIDE	<1.25	10	9.4	94
SULFATE	49.4	25	74.4	100

WSSRAP SOIL SPIKES MATRIX: SOIL UNITS: UG/6

SAMPLE NO: AA05610

SITE ID: S0-49080-100295-0,2-1187

	SAMPLE CONC	ADDED AMOUNT	SPIKE	PERCENT RECOVERY
NITOATE	2 2	2.5	4.99	115
NITRATE CHLORIDE	2.2 2.8	4.55	7.35	117
FLUORIDE	5.1	3	8.2	104
SULFATE	22	8	31.9	95

SAMPLE NO: AA05300

SITE ID: S0-51308-100085-0,2-1187

	SAMPLE CONC	ADDED AMOUNT	SPIKE	PERCENT RECOVERY
NITRATE	3.82	10	12.3	89
CHLORIDE	4.78	15	9.3	92
FLUORIDE	3.76	10	13.1	9 5
SULFATE	15.4	10	23.4	88

SAMPLE NO: AA05328

SITE ID: S0-50140-98820-2,4-1187 (MS)

	SAMPLE CONC	ADDED AMOUNT	SPIKE	PERCENT RECOVERY
NITRATE	10.8	10	20.18	93.8
CHLORIDE	11.9	10	20.1	82
FLUORIDE	21.6	20	39.9	91.5
SULFATE	31.6	25	52.6	84

WSSRAP PROJECT NO: 100-03 SOIL DUPLICATES MATRIX: SOIL UNITS: UG/G

SAMPLE NO: AA05280

SITE ID: S0-51137-101068-8,10-1187

	NITRATE	FLUORIDE	CHLORIDE	SULFATE
RESULT #1	1.83	7.94	3.42	29.33
RESULT #2	1.4	6.5	2.5	26.0
SAMPLE NO: AAO SITE ID: SO-51	15294 500-101190-2,4-1	187		
	NITRATE	FLUORIDE	CHLORIDE	SULFATE
RESULT #1	1.79	6.94	4.36	53.59
RESULT #2	2.3	5.2	5.9	59.2
SAMPLE NO: AAO SITE ID: SO-49	95613 1080-100295-6,8-1	187		
	NITRATE	FLUORIDE	CHLGRIDE	SULFATE
RESULT #1	1.67	21.40	1.20	4.30
RESULT #2	1.4	18.7	0.7	3.7
SAMPLE NO: AAC SITE ID: SO-49	05623 2101-100500-0,2-1	187		
	NITRATE	FLUORIDE	CHLORIDE	SULFATE
RESULT #1	3.58	9.53	1.3	34.24

RESULT #2 3.5 8.8 1.2 34.1

SAMPLE NO: AA05633

SITE ID: S0-49000-100665-4,6-1187

	NITRATE	FLUORIDE	CHLORIDE	SULFATE
RESULT #1	1.43	15.65	2.63	136.20
RESULT #2	1.2	13.2	2.2	114
SAMPLE NO: AAC	95650 9082-100570-14,16	-1187		

	NITRATE	FLUORIDE	CHLORIDE	SULFATE
RESULT #1	2.47	6.95	0.94	13.66
RESULT #2	1.8	8.7	0.9	12.6

SAMPLE NO: AA05309

SITE ID: S0-51150-100220-8,10-1187

	NITRATE	FLUORIDE	CHLORIDE	SULFATE
RESULT #1	8.86	7.62	4.44	26.05
RESULT #2	6.29	6.66	0.94	18.9

SAMPLE NO: AA05556

SITE ID: S0-49000-100295-4,6-1187

	NITRATE	FLUORIDE	CHLORIDE	SULFATE
RESULT #1	1.40	12.10	19.39	27.73
RESULT #2	0.61	10.6	14.6	22.4

WSSRAP SOIL SPIKES MATRIX: SOIL UNITS: UG/6

SAMPLE NO: AA05278

SITE ID: S0-51137-101068-4,6-1187

	SAMPLE CONC	SPIKE CONC	SAMPLE+SPIKE	PERCENT RECOVERY
ALUMINUM		NA		
ANTIMONY	⟨6.0	10	⟨6.0	NC
ARSENIC	6.15	4	11.3	129
BARIUM	140	200	334	97
BERYLLIUM	0.71	5	5.95	104.8
CADMIUM	<0.5	5	4.96	99.2
CALCIUM		NA		
CHROMIUM	19	20	32.8	69
COBALT	7.82	50	56.6	97.6
COPPER	5.26	25	28.9	94.6
IRON		NA		
LEAD	⟨3.6	50	49.9	99.8
MAGNESIUM		NA		
MANGANESE	615	50	680	NC
MERCURY	<0.1	1	1.23	123
NICKEL	11.1	50	58.3	94.4
POTASSIUM		NA		***
SELENIUM	<0.5	1	<0.5	0
SILVER	<0.30	5	4.49	89.8
SODIUM		NA		
THALLIUM	<0.5	5	4.3	86
VANADIUM	28.8	50	74.9	92.2
ZINC	20.4	50	65	89.2
LITHIUM		NA		

SAMPLE NO: AA05327

SITE ID: S0-50140-98820-2,4-1187

	SAMPLE CONC	SPIKE CONC	SAMPLE+SPIKE	PERCENT RECOVERY
ALUMINUM		NA		
ANTIMONY	< 6. 0	10	16.3	163
ARSENIC	6.56	4	10.9	87
BARIUM	65555.3	200	467	201
BERYLLIUM	1.09	5	6.92	117
CADMIUM	0.6	5	6.24	113
CALCIUM		NA		
CHROMIUM	17	20	46.3	147
COBALT	6.2	50	76.6	141
COPPER	12.6	25	45.7	132
IRON		NA		
LEAD	8.61	50	73.4	130
MAGNESIUM		NA		
MANGANESE	77.8	50	257	358
MERCURY	<0.1	1	0.93	93
NICKEL	18.9	50	86.6	135
POTASSIUM		NA		
SELENIUM	<0.5	1	<0.5	0
SILVER	0.76	5	6.22	109
SODIUM		NA		
THALLIUM	<0.5	5	4.4	88
VANADIUM	28.5	50	111	165
ZINC	28.3	50	85.2	114
LITHIUM		NA		

SAMPLE NO: AAO5334

SITE ID: S0-50230-98991-4,6-1187

	SAMPLE CONC	SPIKE CONC	SAMPLE+SPIKE	PERCENT RECOVERY
ALUMINUM		NA		
ANTIKONY	<6	10	14	140
ARSENIC	5	4	9.02	101
BARIUM	146	200	355	104.5
BERYLLIUM	0.71	5	5.98	105.4
CADMIUM	<0.5	5	5.58	111.6
CALCIUM		NA		
CHROMIUM	14	20	36.B	114
COBALT	₹5	50	54.9	109.8
COPPER	16.7	25	29.1	49.6
IRON		NA		~~~
LEAD	6.93	50	53.2	92.5
MAGNESIUM		NA		
MANGANESE	221	50	397	NC
MERCURY	<0.1	1	0.76	76
NICKEL	9	50	55.8	93.4
POTASSIUM		NA		
SELENIUM	⟨0.5	1	<0.5	0
SILVER	0.37	5	4.91	98.2
SODIUM		NA		
THALLIUM	<0.5	5	4.2	84
MUIDANAV	24.B	50	80.9	112
ZINC	24.9	50	66.1	82.4
LITHIUM		NA		

SAMPLE NO: AA05566

SITE ID: S0-49475-99985-0,2-1187

	SAMPLE CONC	SPIKE CONC	SAMPLE+SPIKE	PERCENT RECOVERY
ALUMINUM		NA		
ANTIMONY	<6.0	10	15.12	151
ARSENIC	14.13	4	17.6	87
BARIUM	152	200	340	94
BERYLLIUM	0.98	5	6.23	105
CADMIUM	0.69	5	5.46	95.4
CALCIUM		NA		er er er
CHROMIUM	18	20	36.4	92
COBALT	10.3	50	56.2	91.8
COPPER	14.8	25	39.2	97.6
IRON		NA		
LEAD	14	50	65.5	103
MAGNESIUM		NA		
MANGANESE	636	50	638	NC
MERCURY	<0.2	1	1.12	112
NICKEL	15.6	50	60.7	90.2
POTASSIUM		NA		~ ~ ~ ~
SELENIUM	<0.5	1	<0.5	0
SILVER	<1.0	5	4.61	92.2
SODIUM		NA		
THALLIUM	0.5	5	5.4	108
VANADIUM	35.1	50	82.3	94.4
ZINC	42.5	50	87.9	90.8
LITHIUM		NA		

SAMPLE NO: AA05604

SITE ID: S0-49000-100500-0,2-1187

	SAMPLE CONC	SPIKE CONC	SAMPLE+SPIKE	PERCENT RECOVERY
ALUMINUM		NA		
ANTIMONY	<6	10	7.8	78
ARSENIC	8.9	4	12.6	93
BARIUM	221	200	402	90.5
BERYLLIUM	0.81	5	5.8	100
CADMIUM	1.34	5	6.18	96.8
CALCIUM		NA		
CHROMIUM	22.1	20	41.7	98
COBALT	10	50	57.4	94.8
COPPER	18.2	25	41.2	92
IRON		NA		
LEAD	10	50	59.1	98.2
MAGNESIUM	** ** **	NA		
MANGANESE	616	50	625	NC
MERCURY	<0.1	1	1.02	102
NICKEL	23.8	50	8.86	90
POTASSIUM		NA		
SELENIUM	<0.5	1	<0.5	0
SILVER	1.31	5	5.54	84.6
SODIUM		NA		
THALLIUM	<1.0	5	4.8	96
VANADIUM	37.5	50	80.4	85.8
ZINC	42.8	50	84.7	83.8
LITHIUM		NA		

SAMPLE NO: AA05620

SITE ID: S0-49132-100440-10,12,1187

	SAMPLE CONC	SPIKE CONC	SAMPLE+SPIKE	PERCENT RECOVERY
ALUMINUM		NA		
ANTIMONY	⟨6.0	50	52	104
ARSENIC	2.35	4	7.75	135
BARIUM	46.6	200	243	98
BERYLLIUM	0.86	5	4.75	78
CADMIUM	<0.50	5	4.2	84
CALCIUM		Nā		
CHROMIUM	11.6	20	32.7	106
COBALT	<5.0	50	47	94
COPPER	4.51	. 25	25.6	84
IRON		NA		
LEAD	<0.5	50	55	110
MAGNESIUM		NA		
MANGANESE	60.3	50	117	113
MERCURY	(0.1	1	1.01	101
NICKEL	7.42	50	48.7	82
POTASSIUM		₩A		
SELENIUM	<0.5	i	<0.5	0
SILVER	(0.3	5	3.76	<i>7</i> 5
SODIUM		NA		
THALLIUM	<1.0	5	5.5	110
VANADIUM	7.92	50	60.2	105
ZINC	16.1	50	65.2	98
LITHIUM		NA		

SAMPLE NO: AA05632

SITE ID: S0-49000-100665-2,4-1187

	SAMPLE CONC	SPIKE CONC	SAMPLE+SPIKE	PERCENT RECOVERY
ALUMINUM		NA		
ANTIMONY	<6	10	20.1	201
ARSENIC	8.15	4	11.19	76
BARIUM	121	200	347	113
BERYLLIUM	0.68	5	5.66	99.6
CADMIUM	<0.5	5	5.33	106.6
CALCIUM		NA		
CHROMIUM	19.9	20	35.5	78
COBALT	7.65	50	53.5	91.7
COPPER	7.3	25	29.2	87.6
IRON		NA		
LEAD	8.3	50	53.7	90.8
MAGNESIUM		NA		
MANGANESE	511	50	722	NC
MERCURY	(0.1	1	0.93	93
NICKEL	14.9	50	60.8	91.8
POTASSIUM		NA		
SELENIUM	<0.5	1	<0.5	Q.
SILVER	<1.0	5	4.79	95.8
SODIUM		NA		
THALLIUM	<0.5	5	4.7	94
VANADIUM	28.6	50	76.9	96.6
ZINC	26.9	50	67.5	81.2
LITHIUM		NA		

SAMPLE NO: AA05643

SITE ID: S0-49082-100570-0,2-1187

	SAMPLE CONC	SPIKE CONC	SAMPLE+SPIKE	PERCENT RECOVERY
ALUMINUM		NA		
ANTIMONY	<6.0	10	16.4	164
ARSENIC	14.6	4	16.7	NC
BARIUM	194	200	364	85
BERYLLIUM	0.7	5	6.02	106.4
CADMIUM	<0.5	5	5.46	109.2
CALCIUM		NA		
CHROMIUM	15	20	47.9	165
COBALT	7.11	50	54.9	95
COPPER	14	25	38.5	0.98
IRON		NA		
LEAD	5.57	50	56.1	101
MAGNESIUM		NA		
MANGANESE	405	50	390	NC
MERCURY	<0.1	1	0.75	75
NICKEL	15.6	50	67.5	103.8
POTASSIUM		NA		
SELENIUM	<0.5	1	<0.5	0
SILVER	<1.0	5	4.61	92.2
MUIDOS		NA		
THALLIUM	<0.5	5	3.6	72
VANADIUM	24.4	50	77.7	106.6
ZINC	36.3	50	86.9	100.6
LITHIUM		NA		

WSSRAP SOIL DUPLICATES MATRIX: SOIL UNITS: UG/KG

SAMPLE NO: AA05327

SITE ID: S0-50140-98820-2,4-1187

SAMPLE NO: AA05334

SITE ID: S0-50230-98991-4,6-1187

	RESULT #1	RESULT #2		RESULT #1	RESULT #2
ALUMINUM	14568	13350	ALUMINUH	10245	9430
ANTIMONY	ND	ND	ANT I MONY	ND	ND
ARSENIC	5	8.18	ARSENIC	5.9	4.29
BARIUH	88.7	80	BARIUM	171.2	216
BERYLLIUM	1.5	1.2	BERYLLIUM	0.8	0.71
CADMIUM	0.8	0.55	CADMIUM	ND	0.62
CALCIUM	3816	2725	CALCIUM	1900	1622
CHROMIUM	23.2	19.6	CHROMIUM	16.4	15.4
COBALT	8.5	7	COBALT	5.9	7.5
COPPER	17.2	13.4	COPPER	19.6	16.9
IRON	23129	17950	IRON	12902	12020
LEAD	11.7	8.8	LEAD	8.1	11.1
LITHIUM	D	ND	LITHIUM	7.6	7.14
MAGNESIUM	2622	2150	MAGNESIUM	1872	1600
MANGANESE	107	74.5	MANGANESE	259	586
MERCURY	ND	ND	MERCURY	ND	ND
NICKEL	25.8	20.1	NICKEL	10.6	15.1
POTASSIUM	ND	590	POTASSIUM	ND	ND
SELENIUM	ND	ND	SELENIUM	ND	ND
SILVER	ND	ND	SILVER	ND	ND
SODIUM	ND	ND	SODIUM	ND	ND
THALLIUM	ND	ND	THALLIUM	ND	ND
VANADIUM	38.9	33.5	VANADIUM	29.1	27.7
ZINC	38.6	38.1	ZINC	29.2	25.8

WSSRAP SOIL DUPLICATES MATRIX: SOIL UNITS: UG/KG

SAMPLE NO: AA05563

SITE ID: S0-49000-100180-2,4-1187

	RESULT #1	RESULT #2
ALUMINUM	30009	15030
ANTIMONY	ND	ND
ARSENIC	34	20.7
BARIUM	98	83
BERYLLIUM	1	0.69
CADMIUM	EN	ND
CALCIUM	4584	3400
CHROMIUM	31.8	17.1
COBALT	ND	ND
COPPER	10.8	5.8
IRON	24651	15560
LEAD	4.5	ND
LITHIUM	11.2	5.4
MAGNESIUM	3494	2160
MANGANESE	86.4	90.8
MERCURY	ND	ND
NICKEL	14.3	9.37
POTASSIUM	ND	ND
SELENIUM	D	ND
SILVER	ND	ND
SODIUM	ND	ND
THALLIUM	ND	ND
VANADIUM	49.8	35.4
ZINC	33.8	17.7

WSSRAP SOIL DUPLICATES MATRIX: SOIL UNITS: UG/KG

SAMPLE NO: AA05605

SITE ID: S0-49000-100500-2,4-1187

SAMPLE NO: AA05285

SITE ID: S0-51360-101175-6,8-1187

	RESULT #1	RESULT #2		RESULT #1	RESULT #2
ALUMINUM	10638	13740	ALUHINUM	15031	12400
ANTIMONY	ND	ND	ANTIMONY	ND	ND
ARSENIC	7.9	7.17	ARSENIC	6.7	4.89
BARIUM	195	123	BARIUM	136.9	127
BERYLLIUM	0.7	0.64	BERYLLIUM	0.8	0.84
CADMIUM	0.9	0.61	CADMIUM	0.7	0.51
CALCIUM	2113	1980	CALCIUM	1918	1545
CHROMIUM	17.9	21.8	CHROMIUM	20.8	21.9
COBALT	11.3	5.6	COBALT	19.9	18.1
COPPER	13.2	11.6	COPPER	9.8	8.6
IRON	14916	13820	IRON	15728	15620
LEAD	14.5	9.32	LEAD	14.4	14.7
LITHIUM	8.9	11.3	LITHIUM	12.3	9.1
MAGNESIUM	2101	2210	MAGNESIUM	1877	1550
MANGANESE	802	141	MANGANESE	643	755
MERCURY	ND	ND	MERCURY	ND	ND
NICKEL	12.5	12	NICKEL	10.4	11.2
POTASSIUM	ND	520	POTASSIUM	ND	575
SELENIUM	ND	ND	SELENIUN	ND	ND
SILVER	1.3	ND	SILVER	1.5	1.88
SODIUM	ND	ND	SODIUM	ND	ND
THALLIUM	ND	ND	THALLIUM	ND	ND
VANADIUM	33.9	34.9	VANADIUK	36.5	36.4
ZINC	27	28.4	ZINC	22.3	18.9

SAMPLE NO: AAO5622

SITE ID: S0-49132-100440-14,16,-1187

	RESULT #1	RESULT #2
ALUMINUM	12524	12560
ANTIMONY	ND	ND
ARSENIC	14.9	4.97
BARIUM	59.4	52.7
BERYLLIUM	1.2	0.63
CADMIUK	ND	ND
CALCIUM	3425	3240
CHROMIUM	17.5	15.3
COBALT	ND	ND
COPPER	10.9	10.2
IRON	20505	21700
LEAD	8.3	7.2
LITHIUM	D	DM
MAGNESIUM	1809	1725
MANGANESE	173	132
MERCURY	ND	ND
NICKEL	20.9	ND
POTASSIUM	ND	ND
SELENIUM	ND	ND
SILVER	ND	ND
SODIUM	ND	ND
THALLIUM	ND	ND
VANADIUM	38.3	33.6
ZINC	20.2	15.7

WSSRAP SOIL DUPLICATES MATRIX: SOIL UNITS: UG/KG

SAMPLE NO: AA05632

SITE ID: S0-49000-100665-2,4-1187

SAMPLE NO: AAO5644

SITE ID: S0-49082-100570-2,4-1187

	RESULT #1	RESULT #2		RESULT #1	RESULT #2
ALUMINUM	10528	9377	ALUMINUM	7401	10690
ANTIMONY	DM	ND	ANTIMONY	ND	ND
ARSENIC	9.2	7	ARSENIC	10	11.6
BARIUM	229	121	BARIUM	175	216
BERYLLIUM	0.8	0.68	BERYLLIUM	0.8	0.69
CADMIUM	ND	ND	CADMIUM	ND	0.60
CALCIUM	1611	1445	CALCIUM	1865	1590
CHRONIUM	17.2	19.9	CHROMIUM	16	25.1
COBALT	10.9	7.7	COBALT	7.9	8.78
COPPER	7.5	7.3	COPPER	36.5	15.0
IRON	13813	12780	IRON	13910	12850
LEAD	8.9	8.3	LEAD	7.7	4.9
LITHIUM	8.7	8.2	LITHIUM	מא	9.9
MAGNESIUM	2005	1790	MAGNESIUM	1830	2030
MANGANESE	610	511	MANGANESE	611	329
MERCURY	ND	ND	MERCURY	ND	ND
NICKEL	18.6	14.9	NICKEL	14.2	123
POTASSIUM	ND	ND	POTASSIUM	ND	ND
SELENIUM	ND	ND	SELENIUM	ND	ND
SILVER	ND	ND	SILVER	ND	ND
SODIUM	D	ND	SODIUM	ND	ND
THALLIUM	ND	ND	THALLIUM	ND	ND
VANADIUM	31	28.6	MUIGANAV	30	31
ZINC	28.3	26.9	ZINC	26.4	30.9

NITROAROMATICS - SOILS MS/MSD RESULTS PERCENT RECOVERY

SAMPLE # AA05328

SITE ID: SO-50140,98820

ANALYTE	MATRIX SPIKE	MATRIX SPIKE DUP
1,3,5-TNB	95	97
1,3-DNB	96	96
NITROBENZENE	99	101
2,4,6-TNT	84	84
2,6-DNT	84	87
2,4-DNT	70	72

SAMPLE # AA05335

SITE ID: SO-50230,98991

ANALYTE	MATRIX SPIKE	MATRIX SPIKE DUP
1,3,5-TNB	103	99
1,3-DNB	106	97
NITROBENZENE	118	103
2,4,6-TNT	92	89
2,6-DNT	96	87
2,4-DNT	80	74

SAMPLE # AA05339

SITE ID: S0-50160,98735

ANALYTE	MATRIX SPIKE	MATRIX SPIKE DUP
1,3,5-TNB	96	98
1,3-DNB	96	97
NITROBENZENE	101	99
2,4,6-TNT	86	82
2,6-DNT	86	84
2,4-DNT	72	72

SAMPLE # AA05342

SITE ID: S0-50160,98735

ANALYTE	MATRIX SPIKE	MATRIX SPIKE DUP
1,3,5-TNB	90	104
1,3-DNB	90	101
NITROBENZENE	97	107
2,4,6-TNT	69	89
2,6-DNT	80	90
2,4-DNT	68	77

SAMPLE # AAG5345

SITE ID: S0-50160,98735

ANALYTE	MATRIX SPIKE	MATRIX SPIKE DUP
1,3,5-TNB	98	97
1,3-DNB	95	96
NITROBENZENE	100	102
2,4,6-TNT	85	86
2,6-DNT	87	87
2,4-DNT	74	72

SAMPLE # AAO5348

SITE ID: SO-50160,98735

ANALYTE	MATRIX SPIKE	MATRIX SPIKE DUP
1,3,5-TNB	94	89
1,3-DNB	94	88
NITROBENZENE	101	94
2,4,6-TNT	82	79
2,6-DNT	84	84
2,4-DNT	70	66

SAMPLE # AA05615

SITE ID: S0-49132,100440

ANALYTE	MATRIX SPIKE	MATRIX SPIKE DUP
1,3,5-TNB	71	66
1,3-DNB	110	105
NITROBENZENE	95	97
2,4,6-TNT	51	56
2,6-DNT	87	87
2,4-DNT	116	112

SAMPLE # AA05623

SITE ID: S0-49101,100500

ANALYTE	MATRIX SPIKE	MATRIX SPIKE DUP
1,3,5-TNB	85	85
1,3-DNB	100	101
NITROBENZENE	94	106
2,4,6-TNT	74	76
2,6-DNT	85	86
2,4-DNT	107	111

metatrace inc. 3D 13715 Rider Trail North SPIKE DUPLICATE RECOVERY (314) 298-856

Lab	Name:_	metaTRACE	,	Contract:_	100-03		
Lap	Code:	meta	Case No.:	SAS No.:		SDG No.:	
Mati	rix Spi	ke - EPA	Sample No.:	5338, 39, 40	Level:(lo	w/med) lo	W

COMPOUND	SPIKE ADDED (ug/Kg)	SAMPLE CONCENTRATION (ug/Kg)	MS CONCENTRATION (ug/Kg)	MS % REC #	QC LIMITS REC.
Phenol	6700	0	3600	E /	=====
2-Chlorophenol	6700	0	3300	1 <u>54</u> 1 49	26- 90 25-102
1,4-Dichlorobenzene	3300	0	2200	67	28-104
N-Nitroso-di-n-prop.(1)	3300	0	1700	52	41-126
1,2,4-Trichlorobenzene	3300	0	2100	64	38-107
4-Chloro-3-methylphenol	6700	0	3300	49	26-103
Acenaphthene	3300	0	1000	30*	31-137
4-Nitrophenol	6700	0	2600	39	11-114
2,4-Dinitrotoluene	3300	0	1400	42	28- 89
Pentachlorophenol	6700	0	0	0*	17-109
Pyrene	3300	0	2800	85	35-142

	SPIKE ADDED	MSD CONCENTRATION	MSD %	S -	OC L	IMITS
COMPOUND	(ug/Kg)	(ug/Kg)	REC #	RPD #	RPD	REC.
Phenol 2-Chlorophenol	6700	4800	72	14	=== ===	===== 26- 90
1,4-Dichlorobenzene	<u>6700</u> 3300	3300	<u>49</u> 6*	0 84*	50 27	25-102 28-104
N-Nitroso-di-n-prop.(1)	3300 3300	1900 1600	58	5	38	41-126
4-Chloro-3-methylphenol	6700	6000	90	- 14 29	23 33	38-107 26-103
Acenaphthene	3300 6700	1900	58 72	22* 30	19	31-137
2,4-Dinitrotoluene	3300	2900	88	35	50 47	11-114 28- 89
PentachlorophenolPyrene	6700 3300	2200	3* 1 182*	100*	47 36	17-109
					20	35-142

(1) N-Nitroso-di-n-propylamine

Column to be used to flag recovery and RPD values with an asterisk Values outside of QC limits

RPD:_ pike	4 Reco	out of very:	11 5 ou	outsi t of _	de li	mits outside	limi	ts		
OMMEN										

3D **metgirace** NATRIX715 FIRET MATRIX SPIKE ESTIPLITY CATE RECOVERY (314) 298-8566

Lab Name: metaTRACE	Contract: 100-03
Lab Code:meta Case No.:	SAS No.: SDG No.:
Matrix Spike - EPA Sample No.: <u>5344</u>	, 45, 46 Level: (low/med) low

COMPOUND	SPIKE ADDED (ug/Kg)	SAMPLE CONCENTRATION (ug/Kg)	MS CONCENTRATION (ug/Kg)	MS % REC #	QC LIMITS REC.
Phenol	6700		=======	======	=====
2-Chlorophenol	6700		3900	<u>67</u> 58	26- 90 25-102
1,4-Dichlorobenzene	3300	0	1800	54	28-104
N-Nitroso-di-n-prop.(1)	3300	0	2000	61	41-126
1,2,4-Trichlorobenzene	3300	0	1700	52	38-107
4-Chloro-3-methylphenol	6700	1	4000	60	26-103
Acenaphthene	3300	10	1300	39	31-137
4-Nitrophenol	6700	0	3300	49	11-114
2,4-Dinitrotoluene	3300	01	1700	52	28- 89
Pentachlorophenol	6700	0	3100	46	17-109
Pyrene	3300	56	3500	104	35-142
					i i

Phenol 6700 4000 2-Chlorophenol 6700 3100		RPD #	RPD	IMITS REC.
2 chrotophenol 6700 3100 1,4-Dichlorobenzene 3300 1200 N-Nitroso-di-n-prop.(1) 3300 2300 1,2,4-Trichlorobenzene 3300 900 4-Chloro-3-methylphenol 6700 4000 Acenaphthene 3300 800 4-Nitrophenol 6700 5600 2,4-Dinitrotoluene 3300 1400 Pentachlorophenol 6700 4200 Pyrene 3300 1400	60 46 36 70 27* 60 24* 84 42 63 42		===== 35 50 27 38 23 33 19 50 47 47 36	===== 26- 90 25-102 28-104 41-126 38-107 26-103 31-137 11-114 28- 89 17-109 35-142

(1) N-Nitroso-di-n-propylamine

Column to be used to flag recovery and RPD values with an asterisk Values outside of QC limits

$ ext{RPD:} 3$ out of 1 opike Recovery: 2	<u>l</u> outside limits out of <u>22</u> outside limits
COMMENTS:	

3D **THE MATRIX SPIKE FOUR PLY CAPTED RECOVERY** (314) 298-8566

Lab Name: metaTRACE	Contract: 100-03
Lab Code: meta Case No.:	SAS 'No.: SDG No.:
Matrix Spike - EPA Sample No.: 5350,	51, 52 Level: (low/med) low

!	SPIKE	SAMPLE	MS	MS	QC
	ADDED	CONCENTRATION	CONCENTRATION	*	LIMITS
COMPOUND	(ug/Kg)	(ug/Kg)	(ug/Kg)	REC #	REC.
			=======================================	=====	=====
Phenol	6700	<u>14J</u>	5400I	80	26- 90
2-Chlorophenol	6700	111J	4400	65	25-102
1,4-Dichlorobenzene	3300	10	2100	64	28-104
N-Nitroso-di-n-prop.(1)	3300	10	2000	61	41-126
1,2,4-Trichlorobenzene	3300	0	2500	76	38-107
4-Chloro-3-methylphenol	6700	0	5200	78	26-103
Acenaphthene	3300	0	1600	48	31-137
4-Nitrophenol	6700	0	4800	72	11-114
2,4-Dinitrotoluene	3300	0	2200	67	28- 89
Pentachlorophenol	6700	0	3700	55	17-109
Pyrene	3300	0	3300	100	35-142

1	SPIKE	MSD	MSD		1	
<u> </u>	ADDED	CONCENTRATION	*	Q	QC L	IMITS
COMPOUND	(ug/Kg)	(ug/Kg) [REC #	RPD #	RPD	REC.
Phenol	6700		=====	=====	=====	=====
	6700	3700	55	18	35	26- 90
2-Chlorophenol	6700	2800	42	21	50	25-102
1,4-Dichlorobenzene	3300	1 1300 1	39	24	27	28-104
N-Nitroso-di-n-prop.(1)	3300	1400	42	18	38	41-126
1,2,4-Trichlorobenzene	3300	1500	45	26*	23	38-107
4-Chloro-3-methylphenol	. 6700	3700	55	17	33	26-103
Acenaphthene	3300	1100	33	18	19	31-137
4-Nitrophenol	6700	3400	51	17	50	11-114
2,4-Dinitrotoluene	3300	1600	48	16	47	28- 89
Pentachlorophenol	6700	2800	42	13	47	17-109
Pyrene	3300	2700	82	10	36	35-142
						i

(1) N-Nitroso-di-n-propylamine

Ħ	Column	to be used	to	flag recovery	and	RPD	values	with	an	actorick
*	Values	outside of	OC	limits	۵۵		·azacs	W 1 C11	an	d3CCL13K

RPD: 1 Spike Reco	out of <u>11</u> outside limits overy: <u>0</u> out of <u>22</u> outside limits	
COMMENTS:		

SOIL PESTICIDE MATRIX SPIKE/MATRIX SPIKE DUPLICATE RECOVERY

Lab Name:	<u>metaTRACE</u>	,	Contract: 100-0	3
Lab Code:	meta	Case No.:	SAS No.:	SDG No.:
Matrix Sp	ike - EPA	Sample No.: _	5342 MS Level: 5343 MSD	(low/med)

COMPOUND	SPIKE ADDED (ug/Kg)	SAMPLE CONCENTRATION (ug/Kg)	(ug/Kg)	REC #	QC LIMITS REC.
gamma-BHC (Lindane) Heptachlor Aldrin Dieldrin Endrin 4,4'-DDT	26.7 26.7 26.7 66.7 66.7	0.0 0.0 0.0 0.0 0.0	13.03 9.07 8.89 39.24 39.42 203.18	33.3* 58.8 59.1	46-127 35-130 34-132 31-134 42-139

COMPOUND	SPIKE ADDED (ug/Kg)	MSD CONCENTRATION (ug/Kg)	MSD % REC #	RPD #	-	IMITS REC.
gamma-BHC (Lindane) Heptachlor Aldrin Dieldrin Endrin 4,4'-DDT	26.7 26.7 26.7 66.7 66.7	7.72 7.11 6.17 32.30 27.73 377.73	28.9* 26.6* 23.1* 48.4 41.6* 566.3*	51.2* 24.4 36.2 19.4 34.7 60.1*	50 31 43 38 45 50	46-127 35-130 34-132 31-134 42-139 23-134

- # Column to be used to flag recovery and RPD values with an asterisk
- * Values outside of QC limits

RPD: 2 Spike Recov	out of 6 outside limits very: 8 out of 12 outside limits
COMMENTS:	

SOIL PESTICIDE MATRIX SPIKE/MATRIX SPIKE DUPLICATE RECOVERY

Lab Name:	metaTRACE		Contract:	100-03
Lab Code:	_meta	Case No.:	_ SAS No.:	SDG No.:
Matrix Sp	ike - EPA :		5 MS	Level: (low/med)

COMPOUND	SPIKE ADDED (ug/Kg)	SAMPLE CONCENTRATION (ug/Kg)	(ug/Kg)	REC #	
gamma-BHC (Lindane) Heptachlor Aldrin Dieldrin Endrin 4,4'-DDT	26.7 26.7 26.7 66.7 66.7	0.0 0.0 0.0 0.0 0.0 0.0	9.31 21.59 13.66 20.77 29.90 89.55	34.9* 80.9 51.2 31.1 44.8 134.3*	46-127 35-130 34-132 31-134 42-139 23-134

COMPOUND	SPIKE ADDED (ug/Kg)	MSD CONCENTRATION (ug/Kg)	MSD % REC #	* RPD #		IMITS REC.
gamma-BHC (Lindane) Heptachlor Aldrin Dieldrin Endrin 4,4'-DDT	26.7 26.7 26.7 66.7 66.7	8.92 21.51 14.30 20.60 29.41 84.07	33.4* 80.6 53.6 30.9* 44.1 126.0	4.4 0.4 4.6 0.6 1.6 6.4	50 31 43 38 45 50	46-127 35-130 34-132 31-134 42-139 23-134

- # Column to be used to flag recovery and RPD values with an asterisk
- * Values outside of QC limits

RPD: 0 out of 6 outside limits Spike Recovery: 4 out of 12 outside limits
COMMENTS:

SOIL PESTICIDE MATRIX SPIKE/MATRIX SPIKE DUPLICATE RECOVERY

Lab Name:	metaTRACE	<u> </u>	Contract: 100-0	03
Lab Code:	meta	Case No.: _	SAS No.:	SDG No.: AA05714
Matrix Spi	lke - EPA	Sample No.:	5714 MS Level	:(low/med) low

СОМРОИИД	SPIKE ADDED (ug/Kg)	SAMPLE CONCENTRATION (ug/Kg)	MS CONCENTRATION (ug/Kg)	REC #	
gamma-BHC (Lindane) Heptachlor Aldrin Dieldrin Endrin 4,4'-DDT	26.7 26.7 26.7 66.7 66.7 66.7	0.0 0.0 0.0 0.0 0.0 0.0	20.3 27.6 14.5 46.0 21.8 0.0	103.4 54.3 69.0 32.7	46-127 35-130 34-132 31-134 42-139 23-134

СОМРОИИД	SPIKE ADDED (ug/Kg)	MSD CONCENTRATION (ug/Kg)	REC #	* TRPD #	RPD	IMITS REC.
gamma-BHC (Lindane) Heptachlor Aldrin Dieldrin Endrin 4,4'-DDT	26.7	14.4	53.9	34.0	50	46-127
	26.7	10.7	40.1	88.3*	31	35-130
	26.7	7.44	27.9*	64.2*	43	34-132
	66.7	29.2	43.8	44.7*	38	31-134
	66.7	26.0	39.0*	17.6	45	42-139
	66.7	8.31	12.5*	200*	50	23-134

Column to be used to flag recovery and RPD values with an asterisk

* Values outside of QC limits

RPD: 4 Spike Recov	out of 6 ery: 4 o	_ outside li ut of12	mits outside	limits	
COMMENTS:					

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June 1, 1988

ADDRESSEES

INTERIM RESPONSE ACTION FOR THE ASH POND ISOLATION SYSTEM

Enclosed you will find the revised Engineering Evaluation/Cost Analysis for the referenced Interim Response Action. This document has been revised to reflect comments received from the EPA and the MDNR (there were no comments from the public) and serves as the responsiveness summary for this response action. It is my understanding that responses to comments have, for the most part, been coordinated with your organization. Therefore, subject to your indication to the contrary we assume that this completes the environmental documentation requirements for this action.

We will complete final design within the next three weeks and will then proceed to award a contract for the work.

If you have any questions, please give me a call.

Sincerely,

R. R. Nelson

Project Manager Weldon Spring Site

Remedial Action Project

Enclosure

cc w/enclosure:
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DOCUMENT NUMBER 1-200-203-1,01

ADDRESSEES FOR LETTER DATED JUNE 2, 1988

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PROPOSED INTERIM RESPONSE ACTION: EE/CA

CONSTRUCTION OF ASH POND ISOLATION SYSTEM AT THE WELDON SPRING SITE

May 1988

DOCUM¤

DOCUMENT NUMBER: 1-200-203-1.02

PROPOSED INTERIM RESPONSE ACTION:

CONSTRUCTION OF ASH POND ISOLATION SYSTEM AT THE WELDON SPRING SITE

SITE BACKGROUND

The Weldon Spring site is located in St. Charles County, Missouri, about 48 km (30 mi) west of St. Louis. From 1941 to 1944, the U.S. Department of the Army operated the Weldon Spring Ordnance Works at the site for production of trinitrotoluene and dinitrotoluene. In the mid 1950s, a portion of the property was transferred to the U.S. Atomic Energy Commission (AEC), a predecessor of the U.S. Department of Energy (DOE).

From 1957 to 1966, the AEC operated a uranium processing facility at the Weldon Spring site. Impure uranium ore concentrates and some scrap uranium metals were processed at the chemical plant, and thorium-containing materials were also processed on an intermittent basis. Following closure by the AEC, the Army reacquired the chemical plant in 1967 and began converting the facilities to produce herbicides. Some of the buildings were partially decontaminated and some equipment was dismantled. In 1969, prior to becoming operational, the herbicide project was canceled. Since that time, the plant has remained essentially unused and in caretaker status. The Army returned a portion of the ordnance works property to the AEC in 1971 but retained control of the chemical plant buildings. In 1984, the Army repaired several of these buildings; decontaminated some of the floors, walls, and ceilings; and removed some contaminated equipment to areas outside of the buildings. In 1985, custody of the chemical plant property was transferred to DOE.

The Ash Pond area is located in the far northwest section of the Weldon Spring site and has the lowest surface elevation on the site (Fig. 1). Water is present only intermittently in Ash Pond and is recharged by surface runoff. The watershed of Ash Pond includes the area around the raffinate pits and and the western quarter (about 25 ha [62 acres]) of the chemical plant area (Fig. 2). Discharge from Ash Pond flows northward to Lake 35, an impoundment on Schote Creek in the August A. Busch Memorial Wildlife Area (U.S. Department of Energy 1987a). Based on the results of dye studies conducted at the site by the Missouri Department of Natural Resources in 1983, a hydraulic connection exists between the Ash Pond outflow stream and Burgermeister Spring, which is also located in the Busch Wildlife Area (Dean 1985). Routine environmental monitoring of intermittent surface runoff has identified substantial levels of uranium contamination in the runoff from Ash Pond.

In order to decrease the release of contaminants off-site, it is proposed that an isolation system (e.g., a dike and diversion channels) be constructed upstream of Ash Pond to limit the flow of surface water over the contaminated area. This report documents the proposed Ash Pond construction project as an interim response action.

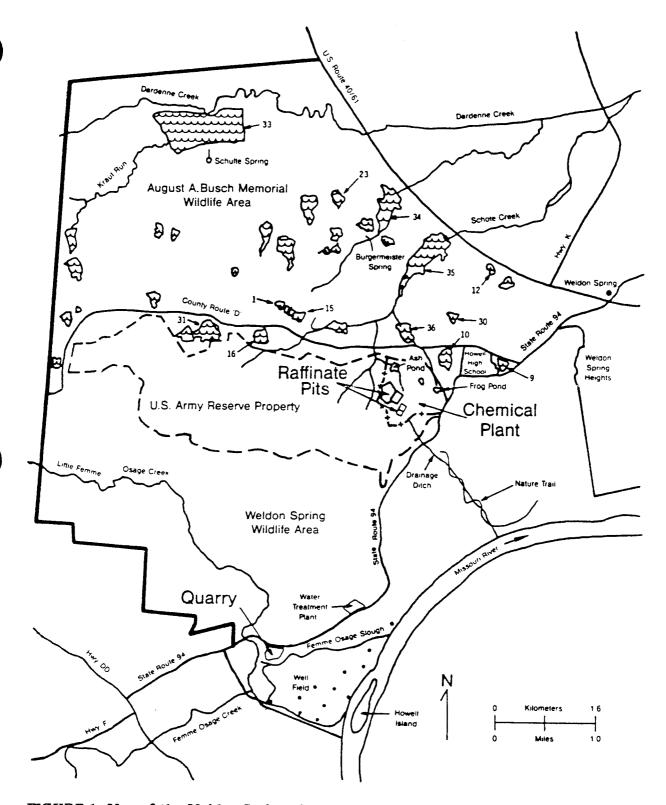


FIGURE 1 Map of the Weldon Spring Site and Vicinity (Source: Modified from U.S. Department of Energy 1987a)

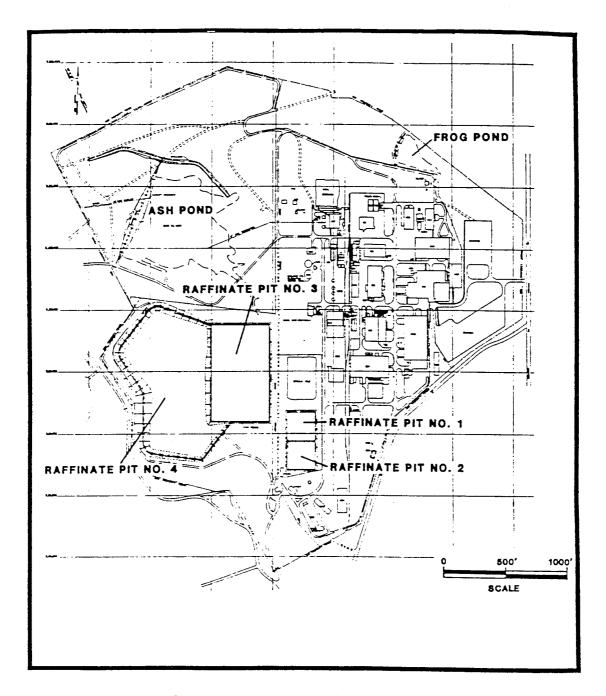


FIGURE 2 Layout of the Weldon Spring Raffinate Pits and Chemical Plant Area

SITE CHARACTERIZATION

Surface Water

A preliminary radiological survey of the Weldon Spring site was performed in 1975. Analyses of water samples from Ash Pond indicated that the concentrations of radium, thorium, and uranium were less than their maximum permissible concentrations (MPCs) as specified in 10 CFR Part 20 (Jacobson 1976; U.S. Department of the Army 1976). (At the time of the survey, 10 CFR Part 20 was the appropriate regulation because the site was under control of the U.S. Department of the Army.) Subsequent radiological sampling identified uranium concentrations in excess of the currently appropriate guideline, i.e., the DOE guideline for uranium-238 in water (600 pCi/L) (U.S. Department of Energy 1986). Levels as high as 4,000 pCi/L were detected in surface runoff from Ash Pond compared with levels up to 400 pCi/L in the watershed upstream from Ash Pond (Kleeschulte and Emmett 1986; MK-Ferguson and Jacobs Engineering 1987).

Recent characterization efforts at the Weldon Spring site have included more extensive sampling for uranium in surface runoff from the Ash Pond watershed. The locations of the sampling points, shown in Fig. 3, were selected because water passing between these points must cross over the known source of radioactive contamination in the watershed — i.e., the South Dump, which was used for disposal of contaminated material during both the uranium-processing period and the Army's decontamination effort at the site.

Results of the runoff sampling program are presented in Table 1. Because rainfall during the months of April, May, June, August, September, October, and November 1987 was insufficient to produce any flow from the watershed, no water samples were collected during those months. The variable results reflect the nature of the sampling method (i.e., grab samples) and the variable flow volumes. To permit the level of contamination to be more accurately determined, procedures and equipment for continuous monitoring and sequential sampling of surface runoff leaving the area were recently put in place; this effort was completed during May 1988.

Geology and Groundwater

During a recent comprehensive characterization of the Weldon Spring site, several boreholes were drilled in and near Ash Pond (see Fig. 4) to define the physical nature of the area. Analysis of these borehole samples indicated that layers of low-permeability clay are present in the area, with thicknesses ranging from 1.5 to 6 m (5 to 20 ft). The thinnest deposits are present in the existing drainage channel, where compacted fill would be placed during the proposed construction project. A cross section of the Ash Pond area is presented in Fig. 5.

Two piezometers placed in the overburden material immediately south of the proposed dike indicate that the local soil is unsaturated. Groundwater in the area occurs

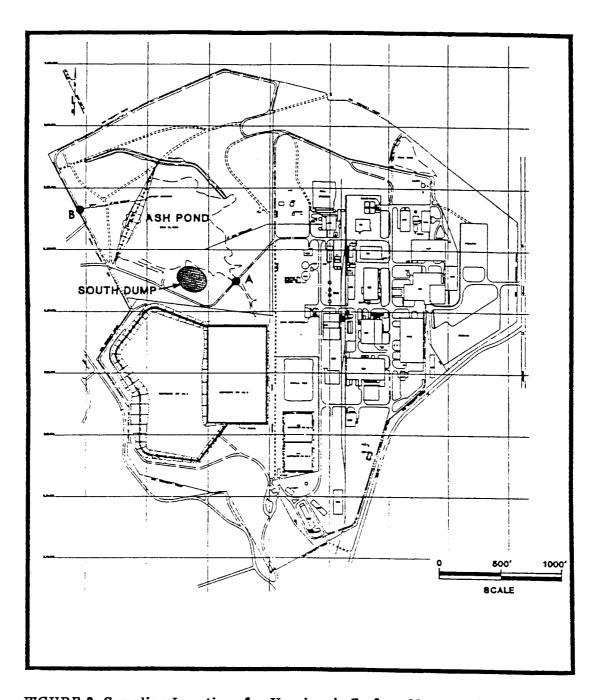


FIGURE 3 Sampling Locations for Uranium in Surface Water at Ash Pond

TABLE 1 Uranium Concentrations in Surface Runoff at the Ash Pond Sampling Points⁸

Month		Natural Uranium (pCi/L)					
	1987		1988				
	Point A	Point B	Point A	Point B			
January	_b	3,500	140 140 45 110 200	2,700 2,800 1,800 1,700 360			
February	-	3,100	100 180	460 900			
March	380	2,100					
July	100	250					
December	- - - -	960 1,500 1,200 1,800					

^aThe locations of Point A (upstream) and Point B (site boundary) are shown in Fig. 3.

in the bedrock, approximately 9 m (30 ft) below the ground surface. Groundwater recharge through this temporary impoundment would be minimal. However, should it occur, the underlying soils would be expected to adsorb contaminants and thus limit migration. (Soils in the area exhibit low hydraulic conductivity and favorable cation exchange properties.) In addition, the proposed upstream isolation dike and diversion channels would significantly reduce the amount of water entering the Ash Pond area, which is believed to be a shallow groundwater recharge area. The resultant decrease in hydraulic head would decrease the rate of infiltration through the contaminated locations in the Ash Pond area (e.g., the South Dump). Based on the thickness and nature of the soils in the affected area, the proposed Ash Pond dike and diversion system would not create a significant groundwater recharge zone. In addition, any water recharging the groundwater from this zone would contain lower levels of uranium than have been

bA dash indicates that no data were collected.

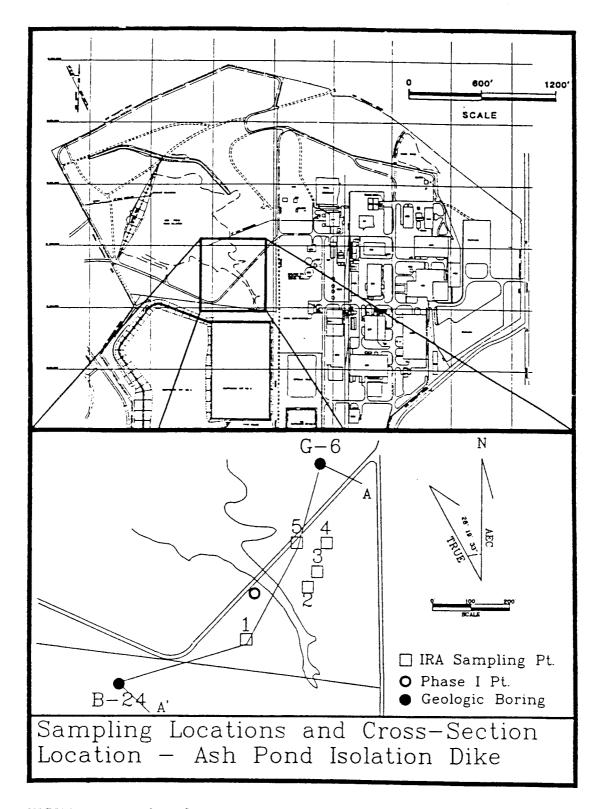


FIGURE 4 Location of Boreholes in the Ash Pond Area

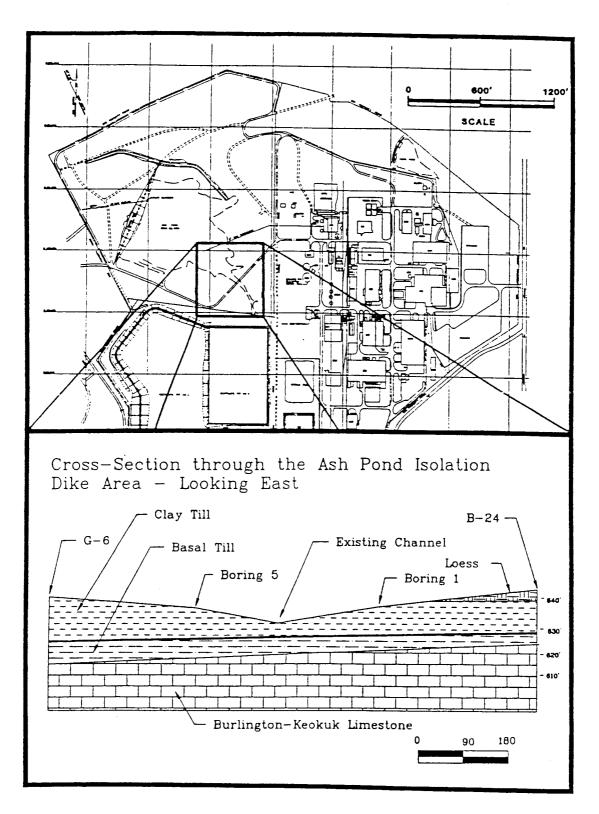


FIGURE 5 Geologic Cross Section of the Ash Pond Area

detected in the losing stream located immediately west of the site. Thus, an improvement in the quality of surface water leaving the Ash Pond area would also improve the quality of the subsurface system (MK-Ferguson and Jacobs Engineering 1988).

Soils

Soils in the Ash Pond area were probably radioactively contaminated as a result of previous processing activities at the Weldon Spring site, migration from the South Dump adjacent to Ash Pond, and past discharges to the pond of decant liquids from the area between raffinate pits 1 and 3 resulting from process line breakage. No known chemical hazards currently exist in the Ash Pond area (MK-Ferguson and Jacobs Engineering 1987).

The Phase I soil investigation program, consisting of a comprehensive radiological and chemical characterization of site soils, was recently completed at the Weldon Spring site (MK-Ferguson and Jacobs Engineering 1988). For the chemical characterization, subsurface soil samples were collected from several boreholes in and around the area proposed for the Ash Pond isolation system (see Fig. 4). These borehole samples Fig. 4) were analyzed for metals, nitroaromatics, inorganic ions (nitrate, sulfate, chloride, and fluoride), and moisture content. Select samples were also analyzed for semivolatile compounds, pesticides, and polychlorinated biphenyls. The results indicated that only nitrate and sulfate levels are slightly elevated, and no chemical hazards exist in the area proposed for the isolation system (MK-Ferguson and Jacobs Engineering 1988).

It is proposed that borrow material for construction of the Ash Pond isolation system be obtained from a spoils pile that is located north of raffinate pit 1 and east of raffinate pit 3 (Fig. 6). This pile probably resulted from the excavation of raffinate pit 4 and typically consists of clayey soils. The spoils pile was chemically characterized during the Phase I soil investigation program. Samples were collected from two locations in the pile and analyzed for metals, nitroaromatics, inorganic ions, and moisture content. No elevated concentrations of chemical contaminants were detected in the samples.

The Ash Pond and spoils pile areas were also surveyed for radiological contamination. The methods employed and values measured during this effort are described in detail in the radiological characterization report for the site (Marutzky et al. 1988). Sampling results for the spoils pile indicate that there is no uranium contamination and that concentrations of radium and thorium are below current DOE guidelines for residual radionuclides in soil (U.S. Department of Energy 1987b), which are provided in Appendix A. (Although DOE has established generic guidelines for radium and thorium in soil, there is no similar guideline for uranium. The guideline for uranium in soil is derived on a site-specific basis.) The pertinent results for the Ash Pond/South Dump area are summarized below.

The analyses of soil samples identified one area south of Ash Pond with a radium-226 concentration above the near-surface (i.e., upper 15 cm [6 in.]) soil guideline,

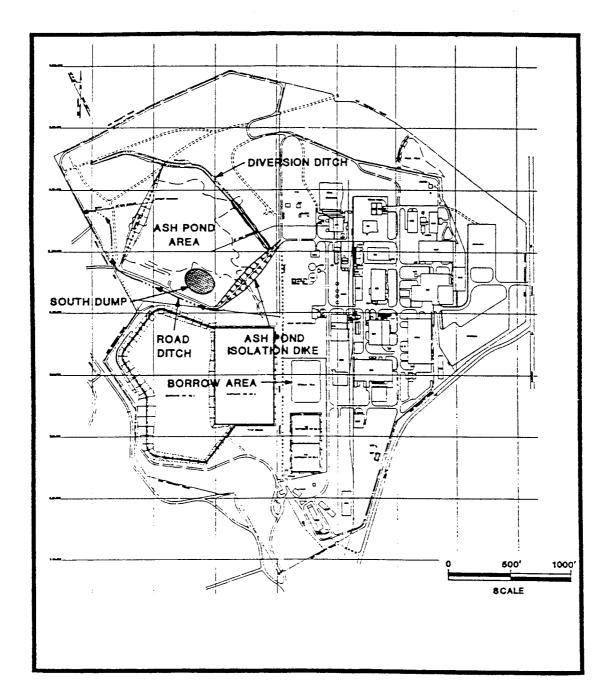


FIGURE 6 Location of Proposed Dike and Borrow Area (Source: Modified from MK-Ferguson and Jacobs Engineering 1987)

but there were no measurements above the guideline for thorium-232 in the area affected by the proposed isolation system. Uranium contamination was detected in the South Dump.

Spectrometric measurements identified two locations southeast of Ash Pond with radium-226 concentrations above the near-surface soil guideline, but no measurements of thorium-232 in the area exceeded the appropriate guideline. Exposure-rate measurements were above background levels in the South Dump.

The subsurface drilling and sampling effort identified the presence of elevated thorium-230 concentrations in the South Dump and elevated uranium concentrations in the Ash Pond/South Dump area. The near-surface soil limit of 5 pCi/g for thorium-230 was exceeded in the South Dump to a maximum depth of 1.2 m (4 ft).

Uranium was detected above 60 pCi/g at maximum depths of 1 m (3 ft) in the South Dump and at greater than 1 m (3 ft) in Ash Pond. Uranium concentrations of 15 pCi/g were detected to a maximum depth of 1.2 m (4 ft) in the South Dump and to a maximum depth of greater than 1 m (3 ft) in Ash Pond. In addition, of 217 boreholes drilled at the site, samples from only two boreholes drilled in the area of the proposed isolation system had radium-226 concentrations above the near-surface soil guideline of 5 pCi/g. A sample from the borehole located east of Ash Pond had elevated radium concentrations to a depth of 0.8 m (2.5 ft), with a maximum of 5.6 pCi/g at a depth of 0.3 m (1 ft). A sample from the borehole located in the South Dump was contaminated to 1 m (3 ft) below the ground surface, with a maximum concentration of 37.5 pCi/g at a depth of 0.3 m (1 ft) (Marutzky et al. 1988).

For comparative purposes, 9 boreholes were drilled off-site to establish background concentrations of radionuclides. The sampling locations (A, B, C, and 1 through 6) are shown in Fig. 7, and the analytical results are summarized in Table 2.

THREAT TO PUBLIC HEALTH AND THE ENVIRONMENT

A potential health and environmental hazard exists at the Weldon Spring site due to high levels of uranium in the outflow from the Ash Pond area. The contamination poses a similar hazard off-site because at least a portion of this outflow, which enters the subsurface just west of the site boundary, surfaces again at Burgermeister Spring in the Busch Wildlife Area. Lake 35 in the wildlife area also receives surface water directly from Ash Pond (MK-Ferguson and Jacobs Engineering 1987). Contamination of Lake 35 and Burgermeister Spring poses a potential health hazard to area personnel, the general public, and resident wildlife.

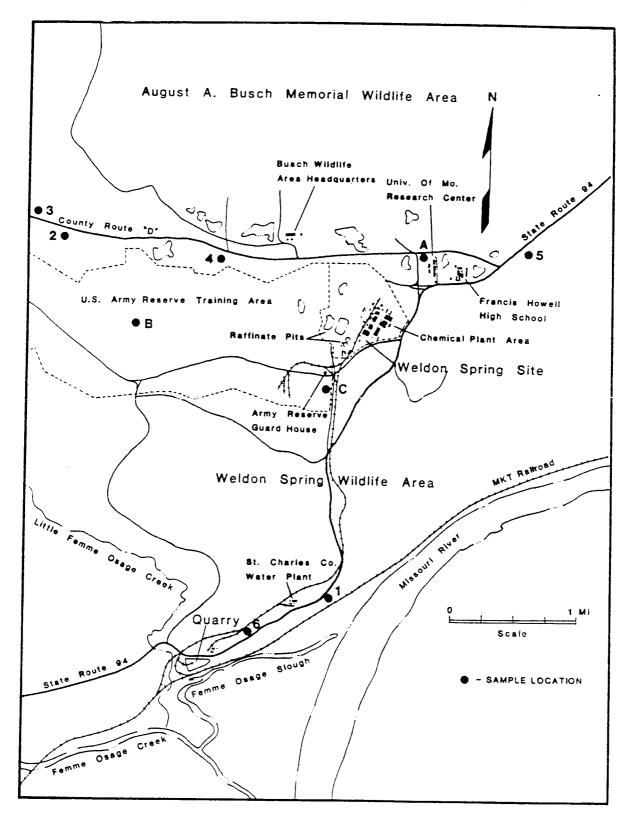


FIGURE 7 Off-site Radiological Sampling Locations (Source: MK-Ferguson and Jacobs Engineering 1987)

TABLE 2 Background Concentrations of Radionuclides in Surface Soil

Off-site Location ^a	Concentration (pCi/g)			
	Radium-226	Thorium-232	Uranium-238	
	0.8	0.9	< DLb	
2	1.1	0.9	< DL	
3	1.3	0.6	< DL	
4	0.8	0.8	< DL	
5	0.9	1.0	< DT	
6	1.1	1.0	< DL	
Α	0.9	0.7	< DL	
В	0.5	1.2	< DL	
С	1.2	0.4	< DL	

^aSamples from locations 1-6 were composited over 15 cm (6 in.); samples from locations A-C were composited over 1 m (3 ft).

Source: Data from MK-Ferguson and Jacobs Engineering (1987).

RESPONSE ACTION

Response Action Objectives

The objectives of the proposed response action are as follows:

- 1. Reduction of the potential on-site health hazard due to uranium contamination of surface water in the Ash Pond area;
- 2. Reduction of the potential off-site health hazard due to uranium contamination of receiving waters in the Busch Wildlife Area;
- 3. Reduction of the surface water infiltration rate through contaminated soils in the Ash Pond area; and
- 4. Improvement in the quality of water being discharged off-site from the Ash Pond area.

These objectives can be met by limiting surface water flow through the contaminated Ash Pond area by means of the proposed isolation system.

bDL = detection limit (about 1.9 pCi/g).

Proposed Response Action Alternatives

Interim (expedited) response actions are implemented to ensure the health and safety of on-site personnel and local populations and to minimize or preclude off-site releases of contamination. These actions are limited to those that can be performed under the Comprehensive Environmental Response, Compensation, and Liability Act, as amended by the Superfund Amendments and Reauthorization Act (SARA), and remain within the constraints of the Council on Environmental Quality's regulations for the National Environmental Policy Act (i.e., actions will be limited to those that do not have an adverse environmental impact nor limit the choice of reasonable alternatives).

The following alternatives have been identified for the proposed interim response action to reduce contamination of surface runoff from the Ash Pond area:

- 1. No action;
- 2. Excavation of contaminated material from the Ash Pond area, including the South Dump, which is responsible for radiological contamination of surface flow through the area, with on-site storage of all material that exceeds the radiological criteria for unrestricted release (and on-site interim storage of any material that exceeds limits for chemical contamination, if discovered, pending a disposal decision);
- Construction of a dike at the site boundary downstream of the Ash Pond area to provide a retention basin for the contaminated water until it can be decontaminated at an on-site water treatment plant; and
- 4. Construction of an isolation dike upstream of Ash Pond to prevent contact of surface runoff with contaminated material in the Ash Pond area (e.g., the South Dump) and construction of diversion channels to route the water away from these contaminated locations for subsequent outflow at its current off-site discharge point.

Screening and Analysis of Response Action Alternatives

The four alternatives that have been identified for the proposed action are screened and analyzed below on the basis of criteria identified in U.S. Environmental Protection Agency (EPA) guidance for removal actions. These criteria include technical feasibility, environmental impacts, cost, and institutional factors (e.g., timeliness, compliance with ARARs, and protectiveness of public health and welfare).

If no action were taken (Alternative 1), the potential health threat posed by uranium contamination of surface runoff from Ash Pond would not be reduced, nor would on-site or off-site environmental conditions be improved. Although Alternative 1 presents no technical barriers and costs nothing in the short term, it is effectively precluded by the potential for adverse environmental impacts and significant long-term

costs (e.g., for the cleanup of areas not currently contaminated but to which contaminants may migrate if no action is taken). It is also precluded by institutional factors related to the community's desire for timely response actions at the Weldon Spring site -- in particular, for a reduction in the off-site release of contaminants.

The action alternatives (Alternatives 2 through 4) are technically feasible and would reduce the potential hazards associated with uranium contamination of surface runoff. Environmental conditions, both on-site and off-site, would be improved if any of these alternatives were implemented.

Alternative 2 is expected to be more expensive than Alternatives 3 and 4. The affected area would need to be protected from surface water intrusion during the excavation period, which would be reflected in costs for constructing an isolation system. In addition to these construction costs, which would be similar to those for Alternatives 3 and 4, Alternative 2 would incur costs associated with storage — i.e., for all material exceeding radiological release criteria and for chemically contaminated material, if encountered, pending a disposal decision. Thus, a material staging area would be required for Alternative 2; the plan for such a staging area is currently being addressed as a separate interim response action because of a separately identified need. The more extensive planning and documentation that would be required prior to the implementation of Alternative 2, because of its expanded scope as compared to Alternatives 3 and 4, would increase costs and delay the initiation of any mitigative action. Therefore, Alternative 2 would not satisfy institutional factors related to timeliness, i.e., the community's desire for expedited response with regard to minimizing off-site releases of radioactively contaminated water.

The excavation of contaminated material from the area of the proposed interim response action is not unique to Alternative 2; it is being addressed in remedial action plans for the Weldon Spring site and would occur subsequent to the implementation of either Alternative 3 or 4. The excavation would likely be included in the scope of the record of decision for remedial action at the Weldon Spring site. Thus, the selection of either Alternative 3 or Alternative 4 would preclude the need for interim storage of contaminated material because a decision on waste disposal would have been made by the time of excavation. An additional advantage of selecting Alternative 3 or Alternative 4 instead of Alternative 2 is the flexibility to initiate a timely response action at the Ash Pond area, without being tied to a decision that is within the broader scope of overall remedial action for the Weldon Spring site.

Although implementation of Alternative 3 would prevent surface water from leaving the Ash Pond area (i.e., by virtue of a downstream dike), it would do nothing to mitigate the contamination of this water (i.e., the contact of inflow with contaminated materials would continue). Thus, a water treatment plant would be required to treat the contaminated water prior to its release off-site. Costs associated with the construction and operation of a water treatment plant would make Alternative 3 more expensive than Alternative 4. In addition, institutional factors associated with public pressure to minimize off-site contaminant releases would not be completely addressed by Alternative 3. Ponding of water above areas of contaminated soil would increase the local hydraulic head, thereby increasing the potential for infiltration through these areas and the resultant transport of radionuclides into the groundwater. Finally, Alternative 3

would be unsatisfactory in terms of timeliness and other institutional factors related to construction of the water treatment plant. Because approval for this construction has not yet been addressed by the appropriate federal, state, or local agencies, considerable delays could occur prior to construction of the treatment plant.

In contrast to Alternative 3, Alternative 4 would involve diversion of surface runoff away from contaminated areas in the watershed. Not only would this preclude the contamination of surface runoff resulting from contact with these areas and obviate the need for a water treatment plant, it would also effectively reduce the hydraulic head at Ash Pond, thereby decreasing the potential for contaminant transport into the groundwater. Alternative 4 could be implemented in a timely and cost-effective manner and would be protective of the public and the environment by limiting the off-site release of contaminants.

As a result of the screening and analysis process for interim response action alternatives, Alternative 4 has been identified as the preferred alternative. Alternative 4 is consistent with and will contribute to the efficient performance of remedial action being planned for the Weldon Spring site.

Description of the Proposed Response Action

Implementation of the proposed interim response action to construct an upstream dike and diversion channels would result in restricting the flow of surface water across the contaminated areas of the Ash Pond watershed. The response action would include the following operations:

- Construction of an isolation dike upstream of Ash Pond -- measuring approximately 230 m (750 ft) in length and 3 m (10 ft) at its maximum height, containing about 5,400 m³ (7,000 yd³) of uncontaminated soil material, and creating a retention pond covering a maximum of 0.6 ha (1.5 acres) when full;
- 2. Construction of diversion channels totaling approximately 610 m (2,500 ft) in length and measuring about 1 m (3 ft) in height, which would circumvent the Ash Pond area and connect the dike to the current point of surface water discharge off-site; and
- Maintenance of the discharge monitoring station currently in place for intermittent measurement of water quality and continuous measurement of the quantity of surface water discharged from the Ash Pond area.

The proposed action would be conducted in accordance with all applicable or relevant and appropriate requirements (ARARs), to ensure protection of the safety and health of on-site workers and local populations and to limit off-site releases of contaminants. Section 121(d)(4) of SARA identifies six conditions under which a waiver from compliance with ARARs may be granted. One of these conditions is that the action

is only part of a total remedial action that will attain such levels or standards of control as identified by the specific ARAR when the total remedial action is completed. If it is determined that a waiver application is necessary, e.g., for uranium discharge limits, this condition is applicable to the proposed interim response action because isolation of the Ash Pond area is by definition an interim measure to minimize the off-site migration of contaminants. It is also important to note that, because the proposed action is an interim measure, the effected reduction in the uranium discharge level is not to be interpreted as an accepted discharge limit for the remedial action project at the Weldon Spring site. Instead, this level is specific to the response action and is dictated by the conditions of that intermediate action, the purpose of which is to improve near-term environmental and safety conditions in the Ash Pond area. The DOE will establish project-specific discharge limits and cleanup criteria for the Weldon Spring site in cooperation with the EPA and the Missouri Department of Natural Resources.

Borrow material for construction of the Ash Pond isolation dike and diversion channels would be obtained from a nearby spoils pile located outside the affected area. Results of characterization studies have indicated that this spoils pile poses no chemical hazard and is not radiologically contaminated.

This interim response action would be taken to reduce the concentration of uranium in water leaving the Ash Pond watershed. It is expected that the uranium concentration would be reduced from as high as 4,000 pCi/L to less than 400 pCi/L, which is below the current DOE uranium-238 limit of 600 pCi/L for release to uncontrolled areas (U.S. Department of Energy 1986). The isolated areas responsible for this contamination (i.e., locations in the Ash Pond area, including the South Dump) would be remediated in the future. Implementation of the proposed response action at this time would minimize the potential adverse impacts on health and the environment resulting from continued runoff of highly contaminated surface water from the watershed and would support the long-term response to contaminated conditions in the Ash Pond area.

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APPENDIX A

DOE GUIDELINES FOR RESIDUAL RADIOACTIVE MATERIAL

U.S. DEPARTMENT OF ENERGY GUIDELINES
FOR RESIDUAL RADIOACTIVE MATERIAL AT
FORMERLY UTILIZED SITES REMEDIAL ACTION PROGRAM
AND
REMOTE SURPLUS FACILITIES MANAGEMENT PROGRAM SITES

(Revision 2, March 1987)

A. INTRODUCTION

This document presents U.S. Department of Energy (DOE) radiological protection guidelines for cleanup of residual radioactive material and management of the resulting wastes and residues. It is applicable to sites identified by the Formerly Utilized Sites Remedial Action Program (FUSRAP) and remote sites identified by the Surplus Facilities Management Program (SFMP).* The topics covered are basic dose limits, guidelines and authorized limits for allowable levels of residual radioactive material, and requirements for control of the radioactive wastes and residues.

Protocols for identification, characterization, and designation of FUSRAP sites for remedial action; for implementation of the remedial action; and for certification of a FUSRAP site for release for unrestricted use are given in a separate document (U.S. Department of Energy 1986) and subsequent guidance. More detailed information on applications of the guidelines presented herein, including procedures for deriving site-specific guidelines for allowable levels of residual radioactive material from basic dose limits, is contained in "A Manual for Implementing Residual Radioactive Material Guidelines" (U.S. Department of Energy 1987), referred to herein as the "supplement".

"Residual radioactive material" is used in these guidelines to describe radioactive material derived from operations or sites over which DOE has authority. Guidelines or guidance to limit the levels of radioactive material and to protect the public and the environment are provided for (1) residual concentrations of radionuclides in soil,** (2) concentrations of airborne

^{*}A remote SFMP site is one that is excess to DOE programmatic needs and is located outside a major operating DOE research and development or production area.

^{**&}quot;Soil" is defined herein as unconsolidated earth material, including rubble and debris that may be present in earth material.

radon decay products, (3) external gamma radiation levels, (4) surface contamination levels, and (5) radionuclide concentrations in air or water resulting from or associated with any of the above.

A "basic dose limit" is a prescribed standard from which limits for quantities that can be monitored and controlled are derived; it is specified in terms of the effective dose equivalent as defined by the International Commission on Radiological Protection (ICRP 1977, 1978). The basic dose limits are used for deriving guidelines for residual concentrations of radionuclides in soil. Guidelines for residual concentrations of thorium and radium in soil, concentrations of airborne radon decay products, allowable indoor external gamma radiation levels, and residual surface contamination concentrations are based on existing radiological protection standards (U.S. Environmental Protection Agency 1983; U.S. Nuclear Regulatory Commission 1982; and DOE Departmental Orders). Derived guidelines or limits based on the basic dose limits for those quantities are used only when the guidelines provided in the existing standards cited above are shown to be inappropriate.

A "guideline" for residual radioactive material is a level of radioactivity or radioactive material that is acceptable if use of the site is to
be unrestricted. Guidelines for residual radioactive material presented
herein are of two kinds: (1) generic, site-independent guidelines taken from
existing radiation protection standards and (2) site-specific guidelines
derived from basic dose limits using site-specific models and data. Generic
guideline values are presented in this document. Procedures and data for
deriving site-specific guideline values are given in the supplement. The
basis for the guidelines is generally a presumed worst-case plausible-use
scenario for the site.

An "authorized limit" is a level of residual radioactive material or radioactivity that must not be exceeded if the remedial action is to be considered completed and the site is to be released for unrestricted use. The authorized limits for a site will include (1) limits for each radionuclide or group of radionuclides, as appropriate, associated with residual radioactive material in soil or in surface contamination of structures and equipment, (2) limits for each radionuclide or group of radionuclides, as appropriate, in air or water, and, (3) where appropriate, a limit on external gamma radiation resulting from the residual material. Under normal circumstances, expected to occur at most sites, authorized limits for residual radioactive material or radioactivity are set equal to guideline values. Exceptional conditions for which authorized limits might differ from guideline values are specified in Sections D and F of this document. A site may be released for unrestricted use only if site conditions do not exceed the authorized limits or approved supplemental limits, as defined in Section F.1, at the time remedial action is completed. Restrictions and controls on use of the site must be established and enforced if site conditions exceed the approved limits, or if there is potential to exceed the basic dose limit if use of the site is not restricted (Section F.2). The applicable controls and restrictions are specified in Section E.

DOE policy requires that all exposures to radiation be limited to levels that are as low as reasonably achievable (ALARA). For sites to be released for unrestricted use, the intent is to reduce residual radioactive material to levels that are as far below authorized limits as reasonable considering technical, economic, and social factors. At sites where the residual material is not reduced to levels that permit release for unrestricted use, ALARA policy is implemented by establishing controls to reduce exposure to levels that are as low as reasonably achievable. Procedures for implementing ALARA policy are discussed in the supplement. ALARA policies, procedures, and actions shall be documented and filed as a permanent record upon completion of remedial action at a site.

B. BASIC DOSE LIMITS

The basic limit for the annual radiation dose received by an individual member of the general public is 100 mrem/yr. The internal committed effective dose equivalent, as defined in ICRP Publication 26 (ICRP 1977) and calculated by dosimetry models described in ICRP Publication 30 (ICRP 1978), plus the dose from penetrating radiation sources external to the body, shall be used for determining the dose. This dose shall be described as the "effective dose equivalent". Every effort shall be made to ensure that actual doses to the public are as far below the basic dose limit as is reasonably achievable.

Under unusual circumstances, it will be permissible to allow potential doses to exceed 100 mrem/yr where such exposures are based upon scenarios that do not persist for long periods and where the annual lifetime exposure to an individual from the subject residual radioactive material would be expected to be less than 100 mrem/yr. Examples of such situations include conditions that might exist at a site scheduled for remediation in the near future or a possible, but improbable, one-time scenario that might occur following remedial action. These levels should represent doses that are as low as reasonably achievable for the site. Further, no annual exposure should exceed 500 mrem.

C. GUIDELINES FOR RESIDUAL RADIOACTIVE MATERIAL

C.1 Residual Radionuclides in Soil

Residual concentrations of radionuclides in soil shall be specified as above-background concentrations averaged over an area of 100 m^2 . Generic guidelines for thorium and radium are specified below. Guidelines for residual concentrations of other radionuclides shall be derived from the basic dose limits by means of an environmental pathway analysis using site-specific data where available. Procedures for these derivations are given in the supplement.

If the average concentration in any surface or below-surface area less than or equal to 25 m^2 exceeds the authorized limit or guideline by a factor of $(100/A)^{1/2}$, where A is the area of the elevated region in square meters,

limits for "hot spots" shall also be applicable. Procedures for calculating these hot spot limits, which depend on the extent of the elevated local concentrations, are given in the supplement. In addition, every reasonable effort shall be made to remove any source of radionuclide that exceeds 30 times the appropriate limit for soil, irrespective of the average concentration in the soil.

Two types of guidelines are provided, generic and derived. The generic guidelines for residual concentrations of Ra-226, Ra-228, Th-230, and Th-232 are:

- 5 pCi/g, averaged over the first 15 cm of soil below the surface
- 15 pCi/g, averaged over 15-cm-thick layers of soil more than 15 cm below the surface

These guidelines take into account ingrowth of Ra-226 from Th-230 and of Ra-228 from Th-232, and assume secular equilibrium. If either Th-230 and Ra-226 or Th-232 and Ra-228 are both present, not in secular equilibrium, the appropriate guideline is applied as a limit to the radionuclide with the higher concentration. If other mixtures of radionuclides occur, the concentrations of individual radionuclides shall be reduced so that (1) the dose for the mixtures will not exceed the basic dose limit or (2) the sum of the ratios of the soil concentration of each radionuclide to the allowable limit for that radionuclide will not exceed 1 ("unity"). Explicit formulas for calculating residual concentration guidelines for mixtures are given in the supplement.

C.2 Airborne Radon Decay Products

Generic guidelines for concentrations of airborne radon decay products shall apply to existing occupied or habitable structures on private property that are intended for unrestricted use; structures that will be demolished or buried are excluded. The applicable generic guideline (40 CFR Part 192) is: In any occupied or habitable building, the objective of remedial action shall be, and a reasonable effort shall be made to achieve, an annual average (or equivalent) radon decay product concentration (including background) not to exceed 0.02 WL.* In any case, the radon decay product concentration (including background) shall not exceed 0.03 WL. Remedial actions by DOE are not required in order to comply with this guideline when there is reasonable assurance that residual radioactive material is not the cause.

^{*}A working level (WL) is any combination of short-lived radon decay products in one liter of air that will result in the ultimate emission of 1.3×10^5 MeV of potential alpha energy.

C.3 External Gamma Radiation

The average level of gamma radiation inside a building or habitable structure on a site to be released for unrestricted use shall not exceed the background level by more than 20 $\mu R/h$ and shall comply with the basic dose limit when an appropriate-use scenario is considered. This requirement shall not necessarily apply to structures scheduled for demolition or to buried foundations. External gamma radiation levels on open lands shall also comply with the basic dose limit, considering an appropriate-use scenario for the area.

C.4 Surface Contamination

The generic surface contamination guidelines provided in Table 1 are applicable to existing structures and equipment. These guidelines are adapted from standards of the U.S. Nuclear Regulatory Commission (NRC 1982)* and will be applied in a manner that provides a level of protection consistent with the Commission's guidance. These limits apply to both interior and exterior surfaces. They are not directly intended for use on structures to be demolished or buried, but should be applied to equipment or building components that are potentially salvageable or recoverable scrap. If a building is demolished, the guidelines in Section C.1 are applicable to the resulting contamination in the ground.

C.5 Residual Radionuclides in Air and Water

Residual concentrations of radionuclides in air and water shall be controlled to levels required by DOE Environmental Protection Guidance and Orders, specifically DOE Order 5480.1A and subsequent guidance. Other Federal and/or state standards shall apply when they are determined to be appropriate.

D. AUTHORIZED LIMITS FOR RESIDUAL RADIOACTIVE MATERIAL

Authorized limits shall be established to (1) ensure that, as a minimum, the basic dose limits specified in Section B will not be exceeded under the worst-case plausible-use scenario consistent with the procedures and guidance provided or (2) be consistent with applicable generic guidelines, where such guidelines are provided. The authorized limits for each site and its vicinity properties shall be set equal to the generic or derived guidelines except where it can be clearly established on the basis of site-specific data —including health, safety, and socioeconomic considerations — that the guidelines are not appropriate for use at the specific site. Consideration should also be given to ensure that the limits comply with or provide a level of protection equivalent to other appropriate limits and guidelines (i.e., state or

^{*}These guidelines are functionally equivalent to Section 4 -- Decontamination for Release for Unrestricted Use -- of NRC Regulatory Guide 1.86 (U.S. Atomic Energy Commission 1974), but they are applicable to non-reactor facilities.

TABLE 1 SURFACE CONTAMINATION GUIDELINES

	Allowable Total Residual Surface Contamination (dpm/100 cm ²) ^a		
Radionuclides ^b	Average ^{c,d}	Maximum ^d ,e	Removable ^d ,f
Transuranics, Ra-226, Ra-228, Th-230, Th-228, Pa-231, Ac-227, I-125, I-129	100	300	20
Th-Natural, Th-232, Sr-90, Ra-223, Ra-224, U-232, I-126, I-131, I-133	1,000	3,000	200
U-Natural, U-235, U-238, and associated decay products	5,000 a	15,000 a	1,000 α.
Beta-gamma emitters (radionuclides with decay modes other than alpha emission or spontaneous fission) except Sr-90 and others noted above	5,000 β-γ	15,000 β-γ	1,000 g-y

As used in this table, dpm (disintegrations per minute) means the rate of emission by radioactive material as determined by correcting the counts per minute measured by an appropriate detector for background, efficiency, and geometric factors associated with the instrumentation.

b Where surface contamination by both alpha- and beta-gamma-emitting radionuclides exists, the limits established for alpha- and beta-gamma-emitting radionuclides should apply independently.

^c Measurements of average contamination should not be averaged over an area of more than 1 m². For objects of less surface area, the average should be derived for each such object.

d The average and maximum dose rates associated with surface contamination resulting from beta-gamma emitters should not exceed 0.2 mrad/h and 1.0 mrad/h, respectively, at 1 cm.

 $^{^{\}rm e}$ The maximum contamination level applies to an area of not more than $100~{\rm cm}^2$.

f The amount of removable radioactive material per 100 cm² of surface area should be determined by wiping that area with dry filter or soft absorbent paper, applying moderate pressure, and measuring the amount of radioactive material on the wipe with an appropriate instrument of known efficiency. When removable contamination on objects of surface area less than 100 cm² is determined, the activity per unit area should be based on the actual area and the entire surface should be wiped. The numbers in this column are maximum amounts.

other Federal). Documentation supporting such a decision should be similar to that required for supplemental limits and exceptions (Section F), but should be generally more detailed because the documentation covers the entire site.

Remedial action shall not be considered complete unless the residual radioactive material levels comply with the authorized limits. The only exception to this requirement will be for those special situations where the supplemental limits or exceptions are applicable and approved as specified in Section F. However, the use of supplemental limits and exceptions should be considered only if it is clearly demonstrated that it is not reasonable to decontaminate the area to the authorized limit or guideline value. The authorized limits are developed through the project offices in the field and are approved by the headquarters program office.

E. CONTROL OF RESIDUAL RADIOACTIVE MATERIAL AT FUSRAP AND REMOTE SFMP SITES

Residual radioactive material above the guidelines at FUSRAP and remote SFMP sites must be managed in accordance with applicable DOE Orders. The DOE Order 5480.1A and subsequent guidance or superceding Orders require compliance with applicable Federal and state environmental protection standards.

The operational and control requirements specified in the following DOE Orders shall apply to interim storage, interim management, and long-term management.

- a. 5000.3, Unusual Occurrence Reporting System
- b. 5440.1C, Implementation of the National Environmental Policy Act
- c. 5480.1A, Environmental Protection, Safety, and Health Protection Program for DOE Operations, as revised by DOE 5480.1 change orders and the 5 August 1985 memorandum from Vaughan to Distribution
- d. 5480.2, Hazardous and Radioactive Mixed Waste Management
- e. 5480.4, Environmental Protection, Safety, and Health Protection Standards
- f. 5482.1A, Environmental, Safety, and Health Appraisal Program
- g. 5483.1A, Occupational Safety and Health Program for Government-Owned Contractor-Operated Facilities
- h. 5484.1, Environmental Protection, Safety, and Health Protection Information Reporting Requirements
- i. 5820.2, Radioactive Waste Management

E.1 Interim Storage

- a. Control and stabilization features shall be designed to ensure, to the extent reasonably achievable, an effective life of 50 years and, in any case, at least 25 years.
- b. Above-background Rn-222 concentrations in the atmosphere above facility surfaces or openings shall not exceed (1) 100 pCi/L at any given point, (2) an annual average concentration of 30 pCi/L over the facility site, and (3) an annual average concentration of 3 pCi/L at or above any location outside the facility site (DOE Order 5480.1A, Attachment XI-1).
- c. Concentrations of radionuclides in the groundwater or quantities of residual radioactive material shall not exceed existing Federal or state standards.
- d. Access to a site shall be controlled and misuse of on-site material contaminated by residual radioactive material shall be prevented through appropriate administrative controls and physical barriers -- active and passive controls as described by the U.S. Environmental Protection Agency (1983--p. 595). These control features should be designed to ensure, to the extent reasonable, an effective life of at least 25 years. The Federal government shall have title to the property or shall have a long-term lease for exclusive use.

E.2 Interim Management

- a. A site may be released under interim management when the residual radioactive material exceeds guideline values if the residual radioactive material is in inaccessible locations and would be unreasonably costly to remove, provided that administrative controls are established to ensure that no member of the public shall receive a radiation dose exceeding the basic dose limit.
- b. The administrative controls, as approved by DOE, shall include but not be limited to periodic monitoring as appropriate, appropriate shielding, physical barriers to prevent access, and appropriate radiological safety measures during maintenance, renovation, demolition, or other activities that might disturb the residual radioactive material or cause it to migrate.
- c. The owner of the site or appropriate Federal, state, or local authorities shall be responsible for enforcing the administrative controls.

E.3 Long-Term Management

Uranium, Thorium, and Their Decay Products

- a. Control and stabilization features shall be designed to ensure, to the extent reasonably achievable, an effective life of 1,000 years and, in any case, at least 200 years.
- b. Control and stabilization features shall be designed to ensure that Rn-222 emanation to the atmosphere from the wastes shall not (1) exceed an annual average release rate of 20 pCi/m²/s and (2) increase the annual average Rn-222 concentration at or above any location outside the boundary of the contaminated area by more than 0.5 pCi/L. Field verification of emanation rates is not required.
- c. Prior to placement of any potentially biodegradable contaminated wastes in a long-term management facility, such wastes shall be properly conditioned to ensure that (1) the generation and escape of biogenic gases will not cause the requirement in paragraph b. of this section (E.3) to be exceeded and (2) biodegradation within the facility will not result in premature structural failure in violation of the requirements in paragraph a. of this section (E.3).
- d. Groundwater shall be protected in accordance with appropriate Departmental Orders and Federal and state standards, as applicable to FUSRAP and remote SFMP sites.
- e. Access to a site should be controlled and misuse of on-site material contaminated by residual radioactivity should be prevented through appropriate administrative controls and physical barriers -- active and passive controls as described by the U.S. Environmental Protection Agency (1983--p. 595). These controls should be designed to be effective to the extent reasonable for at least 200 years. The Federal government shall have title to the property.

Other Radionuclides

f. Long-term management of other radionuclides shall be in accordance with Chapters 2, 3, and 5 of DOE Order 5820.2, as applicable.

F. SUPPLEMENTAL LIMITS AND EXCEPTIONS

If special site-specific circumstances indicate that the guidelines or authorized limits established for a given site are not appropriate for a portion of that site or for a vicinity property, then the field office may request that supplemental limits or an exception be applied. In either case, the field office must justify that the subject guidelines or authorized limits are not appropriate and that the alternative action will provide adequate

protection, giving due consideration to health and safety, the environment, and costs. The field office shall obtain approval for specific supplemental limits or exceptions from headquarters as specified in Section D of these guidelines and shall provide to headquarters those materials required for the justification as specified in this section (F) and in the FUSRAP and SFMP protocols and subsequent guidance documents. The field office shall also be responsible for coordination with the state or local government of the limits or exceptions and associated restrictions as appropriate. In the case of exceptions, the field office shall also work with the state and/or local governments to ensure that restrictions or conditions of release are adequate and mechanisms are in place for their enforcement.

F.1 Supplemental Limits

The supplemental limits must achieve the basic dose limits set forth in this guideline document for both current and potential unrestricted uses of a site and/or vicinity property. Supplemental limits may be applied to a vicinity property or a portion of a site if, on the basis of a site-specific analysis, it is determined that (1) certain aspects of the vicinity property or portion of the site were not considered in the development of the established authorized limits and associated guidelines for that vicinity property or site and, (2) as a result of these unique characteristics, the established limits or guidelines either do not provide adequate protection or are unnecessarily restrictive and costly.

F.2 Exceptions

Exceptions to the authorized limits defined for unrestricted use of a site or vicinity property may be applied to a vicinity property or a portion of a site when it is established that the authorized limits cannot be achieved and restrictions on use of the vicinity property or portion of the site are necessary to provide adequate protection of the public and the environment. The field office must clearly demonstrate that the exception is necessary and that the restrictions will provide the necessary degree of protection and will comply with the requirements for control of residual radioactive material as set forth in Section E of these guidelines.

F.3 Justification for Supplemental Limits and Exceptions

Supplemental limits and exceptions must be justified by the field office on a case-by-case basis using site-specific data. Every effort should be made to minimize use of the supplemental limits and exceptions. Examples of specific situations that warrant use of the supplemental standards and exceptions are:

a. Where remedial action would pose a clear and present risk of injury to workers or members of the general public, notwithstanding reasonable measures to avoid or reduce risk.

- b. Where remedial action even after all reasonable mitigative measures have been taken would produce environmental harm that is clearly excessive compared to the health benefits to persons living on or near affected sites, now or in the future. A clear excess of environmental harm is harm that is long-term, manifest, and grossly disproportionate to health benefits that may reasonably be anticipated.
- c. Where it is clear that the scenarios or assumptions used to establish the authorized limits do not, under plausible current or future conditions, apply to the property or portion of the site identified and where more appropriate scenarios or assumptions indicate that other limits are applicable or necessary for protection of the public and the environment.
- d. Where the cost of remedial action for contaminated soil is unreasonably high relative to long-term benefits and where the residual radioactive material does not pose a clear present or future risk after taking necessary control measures. The likelihood that buildings will be erected or that people will spend long periods of time at such a site should be considered in evaluating this risk. Remedial action will generally not be necessary where only minor quantities of residual radioactive material are involved or where residual radioactive material occurs in an inaccessible location at which site-specific factors limit their hazard and from which they are costly or difficult to remove. Examples include residual radioactive material under hard-surface public roads and sidewalks, around public sewer lines, or in fence-post foundations. A site-specific analysis must be provided to establish that it would not cause an individual to receive a radiation dose in excess of the basic dose limits stated in Section B, and a statement specifying the level of residual radioactive material must be included in the appropriate state and local records.
- e. Where there is no feasible remedial action.

G. SOURCES

Limit or Guideline Source Basic Dose Limits Dosimetry model and dose limits International Commission on Radiological Protection (1977, 1978) Generic Guidelines for Residual Radioactivity Residual concentrations of radium 40 CFR Part 192 and thorium in soil 40 CFR Part 192 Airborne radon decay products External gamma radiation 40 CFR Part 192 Surface contamination Adapted from U.S. Nuclear Regulatory Commission (1982) Control of Radioactive Wastes and Residues DOE Order 5480.1A and subsequent Interim storage guidance DOE Order 5480.1A and subsequent Long-term management guidance; 40 CFR Part 192;

DOE Order 5820.2

H. REFERENCES

- International Commission on Radiological Protection, 1977. Recommendations of the International Commission on Radiological Protection (Adopted January 17, 1977). ICRP Publication 26. Pergamon Press, Oxford. [As modified by "Statement from the 1978 Stockholm Meeting of the ICRP." Annals of the ICRP, Vol. 2, No. 1, 1978.]
- International Commission on Radiological Protection, 1978. Limits for Intakes of Radionuclides by Workers. A Report of Committee 2 of the International Commission on Radiological Protection. Adopted by the Commission in July 1978. ICRP Publication 30. Part 1 (and Supplement), Part 2 (and Supplement), Part 3 (and Supplements A and B), and Index. Pergamon Press, Oxford.
- U.S. Atomic Energy Commission, 1974. Regulatory Guide 1.86, Termination of Operating Licenses for Nuclear Reactors. June 1974.
- U.S. Department of Energy, 1986. Formerly Utilized Sites Remedial Action Program. Summary Protocol: Identification Characterization Designation Remedial Action Certification. Office of Nuclear Energy, Office of Terminal Waste Disposal and Remedial Action, Division of Remedial Action Projects. January 1986.
- U.S. Department of Energy, 1987. Supplement to U.S. Department of Energy Guidelines for Residual Radioactive Material at Formerly Utilized Sites Remedial Action Program and Remote Surplus Facilities Management Program Sites. A Manual for Implementing Residual Radioactive Material Guidelines. Prepared by Argonne National Laboratory, Los Alamos National Laboratory, Oak Ridge National Laboratory, and Pacific Northwest Laboratory for the U.S. Department of Energy. [In press.]
- U.S. Environmental Protection Agency, 1983. Standards for Remedial Actions at Inactive Uranium Processing Sites; Final Rule (40 CFR Part 192). Federal Register 48(3):590-604 (January 5, 1983).
- U.S. Nuclear Regulatory Commission, 1982. Guidelines for Decontamination of Facilities and Equipment Prior to Release for Unrestricted Use or Termination of Licenses for Byproduct, Source, or Special Nuclear Material. Division of Fuel Cycle and Material Safety, Washington, D.C. July 1982.

IRA-200-204



UNITED STATES ENVIRONMENTAL PROTECTION AGENCY

REGION VII 726 MINNESOTA AVENUE KANSAS CITY, KANSAS 66101 R. Nelson 5. McCracken K. Lawver G. Newtown Joc Control PEER

JUL 28 1988

Mr. Rodney R. Nelson
U.S. Department of Energy
Weldon Spring Site Remedial
Action Project
Route 2, Highway 94, South
St. Charles, Missouri 63303

Dear Mr. Nedson:

We have reviewed the revised interim response action proposal for construction of an isolation system at the Ash Pond. In general, the revised document is sufficiently responsive to the comments we provided on the original submission. However, there are some remaining deficiencies in the area of conceptual design that make it difficult for us to assure that the dike system will be properly engineered for the site-specific conditions, and will meet the response action objectives. Our specific concerns are outlined below.

- Design criteria on the retention capacity of the Ash Pond isolation dike should have been presented. In particular, the dike should be designed to retain surface run-off from a defined storm event.
- The conceptual design should include a detailed description of the discharge monitoring system to evaluate the quality of water leaving the site.

In addition, the plan should have provided more detailed information on erosion control during and after construction of the proposed dike, and an abatement protocol in the event erosion occurs following seeding of exposed surfaces. Also, the plan should have addressed how the action might impact remedial investigation activities such as the chemical soil sampling investigation.

In summary, the revised document is sufficiently responsive to our comments; however, we would appreciate the opportunity to review the design criteria prior to letting a contract for construction.

DOCUMENT NUMBER: 1-200 -204-1-01

Thank you for the opportunity to comment on the proposal. Please call if you have any questions.

Sincerely yours,

B. Katherine Biggs Chief, Environmental Review Branch

cc: Dave Bedan, MDNR

IRA-200-205

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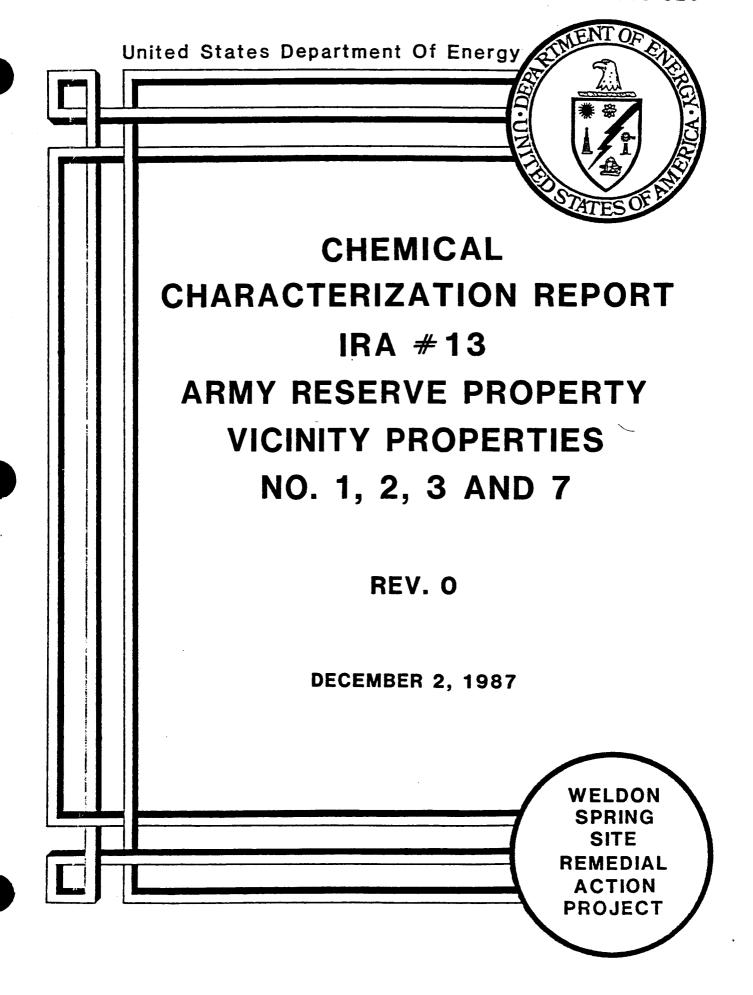
INTERIM RESPONSE ACTION (IRA) ADMINISTRATIVE RECORD FILE ARFS FILE # IR-0300

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CHEMICAL CHARACTERIZATION REPORT

IRA #13

ARMY RESERVE PROPERTY VICINITY PROPERTIES

NO. 1, 2, 3 AND 7

PREPARED FOR:

U.S. DEPARTMENT OF ENERGY
OAK RIDGE OPERATIONS OFFICE
UNDER CONTRACT NO. DE-AC05-860R21548

PREPARED BY:

MK-FERGUSON COMPANY

AND

JACOBS ENGINEERING GROUP, INC.

ROUTE 2, HIGHWAY 94 SOUTH

ST. CHARLES, MISSOURI 63303

DECEMBER 2, 1987

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FIGURE

1.1 LOCATION OF VICINITY PROPERTIES 1, 2, 3, AND 7 4

1.0 INTRODUCTION

This report presents the analytical results of chemical sampling from radiologically contaminated areas on the U.S. Army Weldon Spring Training Area. The sampling was designed to supply chemical constituent information for these properties so remedial action plans could be addressed. A brief summary is provided for background knowledge of site conditions.

The Weldon Spring Site (WSS) is located on State Highway 94, southwest of St. Charles, Missouri. Adjacent to this site is the U.S. Army Reserve Property now referred to as the Weldon Spring Training Area (WSTA). This is a fenced area covering approximately 746 hectares (Ha). Portions of the WSTA are cleared and easily accessible, while other sections are wooded or overgrown with heavy brush. An extensive system of drainage ditches exists on the property. The site contains numerous paved and unpaved roads, several inactive rail lines, a few ponds, and numerous small buildings. Remnants of rail lines and buildings, previously associated with ordnance manufacturing operations, remain on the WSTA.

During World War II, the U.S. Army acquired approximately 17000 acres of the Weldon Spring Area for the production of explosives. Up to twenty production lines were operated at the Weldon Spring Ordnance Works (WSOW) during this period. The WSOW was declared surplus after World War II and 15000 of the

original 17000 acres were conveyed as surplus property to various parties. Two hundred and five (205) acres were transferred to the U.S. Atomic Energy Commission (AEC) for use as an Uranium Feed Materials Plant. The AEC plant operated from 1957 to 1967.

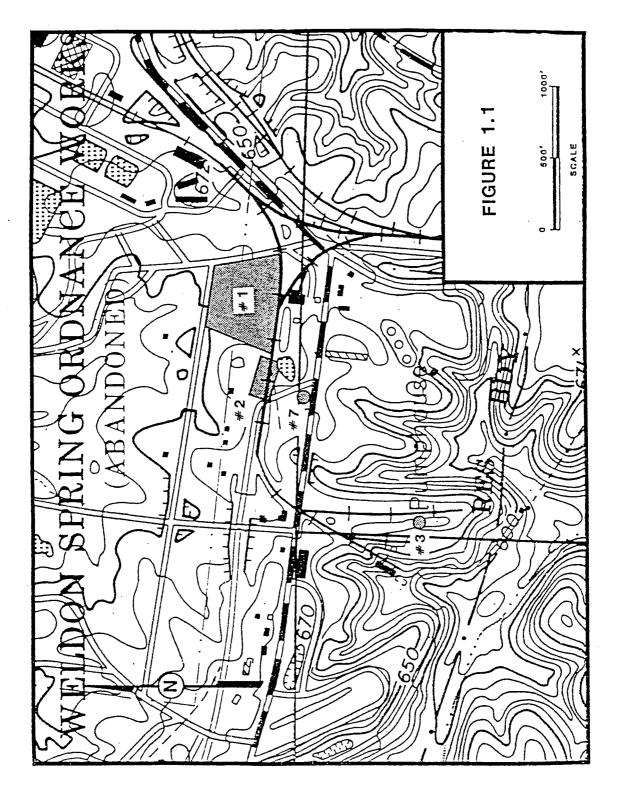
In 1967, the AEC closed the plant. The AEC transferred the property back to the Army which placed the facility in a caretaker mode until 1985. Custody was transferred back to the Department of Energy, the successor to the AEC, and the Weldon Spring Site Remedial Action Project (WSSRAP) was designated to remediate and dispose of the waste associated with the Feed Materials Plant.

A radiological survey of the WSTA was performed by Oak Ridge Associated Universities (ORAU) during March - July, 1985. The methods and procedures utilized in that survey are presented in a report prepared by ORAU (ORAU, 1986). The ORAU report identified seven (7) locations, hereinafter referred to as vicinity properties, on the WSTA. These seven (7) vicinity properties were identified by the presence of uranium, radium, and thorium contamination from former uranium processing operations at the Weldon Spring Site (WSS). This report addresses Vicinity Properties No. 1, 2, 3, and 7.

In June - August, 1987 these vicinity properties were resurveyed for the WSSRAP to determine the extent of radiological

contamination and to delineate excavation depths. This second survey was performed in order to determine the appropriate remedial action effort necessary to remove the radiological contaminants associated with the former uranium processing plant from the vicinity properties. In addition, soil samples were collected for subsequent chemical analyses.

Section 3.0 of this report lists the analytical parameters for each Vicinity Property sample. The analytical parameters were chosen based on contaminants that exist or could potentially exist due to the WSS operational history. This analytical information will be used to determine the level of personnel protection required during remedial action and to obtain chemical constituent information about the radiologically contaminated material. The location of each Vicinity Property addressed in this report is shown in Figure 1.1. A detailed description of the history of operations at the WSS can be found in a report published by the Atomic Energy Commission (U.S.A.E.C., 1960).



LOCATION OF VICINITY PROPERTIES 1, 2, 3, AND 7

2.0 DESCRIPTION OF SAMPLES

Each of six composite samples were analyzed for various parameters which exist or have the potential to exist as contaminants from the WSOW or AEC operations. The samples were collected in areas where radiological contamination is known to be present. The purpose was to obtain chemical constituent information about the radiologically contaminated material that will be excavated and transported to the WSS for temporary storage. In addition, radiometric analyses for thorium-230 were performed to determine if this contaminant was present at levels affecting the quantities of material identified as above the residual soil guidelines for U-238 and Ra-226. Details concerning the boundaries and extent of radiological contamination are presented in separate reports prepared for each vicinity property (Ref. 2-5).

At Vicinity Property No.1, three (3) samples were analyzed. The three samples were equally weighted composites of aliquots from soil samples representing three zones of contamination. The zones of contamination were identified with respect to depth and location of radioactive contaminants. Samples used to form the composites were selected from different locations and depths within each zone. These three zones are identified as tracks, outer zone, and perimeter mixes.

The "tracks" sample is a composite of four (4) samples which were collected near a mound of radiologically contaminated soil and metal debris near the abandoned railroad tracks. The "outer zone" sample is a composite of six (6) samples, collected from an area outside the mound. The "perimeter" sample is a composite of five (5) samples, collected near the outer boundary of the radiologically contaminated region.

At Vicinity Property No. 2, one composite sample was analyzed from six (6) samples collected within the area of radiological contamination. Equally weighted aliquots of those six samples were composited into one representative sample for the volume of radiologically contaminated material which requires remedial action.

At Vicinity Property No. 3, one composite sample was analyzed from four (4) samples collected within the radiologically contaminated area. Equally weighted aliquots of those four samples were composited into one representative sample for the volume of radiologically contaminated material which requires remedial action.

Only thirty-five (35) square feet of material is radiologically contaminated at Vicinity Property No. 7, therefore only one soil sample location was analyzed representing the entire Vicinity Property.

3.0 RESULTS OF ANALYSES

All six samples were analyzed by an offsite independent laboratory subcontracted by the WSSRAP. All analyses for metals, PCB's and semi-volatiles were performed according to the Environmental Protection Agency - Contract Laboratory Program (EPA-CLP) Methods. Specific procedure details can be found in the EPA Contract Laboratory Program Test Methods Manual. All nitroaromatics were analyzed by procedures developed for the U.S. Army Toxic and Hazardous Materials Agency (USATHAMA) and are referred to as the USATHAMA Method. Thorium-230 was analyzed by alpha spectroscopy according to methods developed by the EPA Eastern Environmental Radiation Facility (EERF 00/07). Individual results for each Vicinity Property location are found in Appendix A.

Concentrations of EPA-CLP Metals in soil have been studied for the environmental and geographical characteristics of this area. Elevated concentrations of some metals in soil are normal for this area and do not necessarily reflect contamination from WSOW or AEC operations. However, detectable quantities of PCB's, semi-volatiles, or nitroaromatics would be associated with those operations mentioned above and would be considered as contamination. A summary of the chemical contamination found are described below:

Vicinity Property No. 1: Tracks

The soil composite was analyzed for CLP-metals and nitroaromatics. The only elevated constituent above background concentrations was barium in the CLP-metals analysis. All other metals and nitroaromatics were at background concentrations or not detected. Radiometric results for thorium-230 were less than 1 pCi/g.

Vicinity Property No. 1: Outer Zone Mix

The soil composite was analyzed for CLP-metals, nitroaromatics, semi-volatiles, and PCB's. The only elevated constituent above background concentration was barium in the CLP-metals analysis. All other metals, nitroaromatics, semi-volatiles, and PCB's were at background concentrations or not detected. Radiometric results for thorium-230 were less than 1.0 pCi/g.

<u>Vicinity Property No. 1: Perimeter</u>

The soil composite was analyzed for CLP-metals and nitroaromatics. The constituents above background concentrations are barium, cobalt, and lead in the CLP-metal analysis. All other metals and nitroaromatics were at background concentrations or not detected. Radiometric results for thorium-230 were 2.4 pCi/g.

Vicinity Property No. 2

The soil composite was analyzed for CLP-metals, nitroaromatics, semi-volatiles, and PCB's. The constituents above background concentrations were barium, chromium, cobalt, lead, vanadium in the CLP-metals and 1,3-Dinitrobenzene in the nitroaromatics analysis. All other metals, nitroaromatics, semi-volatiles, and PCB's were at background concentrations or not detected. Radiometric results for thorium-230 were 2.9 pCi/g.

Vicinity Property No. 3

The soil composite was analyzed for CLP-metals and nitroaromatics. The only constituent above background concentration was lead in the CLP metals analysis. All other metals and nitroaromatics were at background concentrations or not detected. Radiometric results for thorium-230 were 2.2 pCi/g.

Vicinity Property No. 7

The soil composite was analyzed for CLP-metals, nitroaromatics, nitrates, sulfates, chlorides, and fluorides. The constituents above background concentrations were barium and lead in the CLP-metals analysis. All other metals, nitroaromatics, nitrates, sulfates, chlorides, and fluorides were at background concentrations or not detected. Radiometric results for

thorium-230 were 12 pCi/g, which is about the same magnitude as Radium-226 concentrations. However, this concentration of Th-230 is not expected to affect the volume of material which requires remedial action.

4.0 CONCLUSION

The results of analyses indicated that a few select metals were above background concentrations at all locations. At Vicinity Property No. 2, one nitroaromatic compound was present. The analytical results from all four properties will be used by site personnel for informational purposes during excavation but are not expected to cause any significant environmental impact.

The results of these analyses were also intended to be used in determining if any additional personnel protection would be required during excavation. Results indicate that no additional protective measures are needed beyond those required for normal radiological protection.

5.0 REFERENCES

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RESULTS OF INDIVIDUAL CHEMICAL ANALYSES

VICINITY PROPERTY NO. 1: TRACKS

METALS EPA CLP METHOD	CONCENTRATION UG/G
Aluminum Antimonv Arsenic Barium Beryllium Cadmium Calcium Chromium Cobalt Copper Iron Lead Lithium Magnesium Manganese Mercury Nickel Potassium Selenium Silver Sodium Thallium Vanadium Zinc	18278 {5.4} 12 210 1.04 3 5764 29 16 24 22451 29 12 3116 645 0.1U 25 945 0.6U 2.5 {192} 1U 47 83
Nitroaromatics USATHAMA METHOD DATE EXTRACTED :08/19/87	CONCENTRATION UG/G
DATE ANALYZED :09/18/87	
2.4.6-TNT 2.4 DNT 2.6 DNT Nitrobenzene 1.3.5-Trinitrobenzene 1.3-Dinitrobenzene	1.56 0.975 1.83 1.87 0.741 1.17
MISCELLANEOUS	PERCENT
Percent Moisture	23.1
Radiochemical	Activity +/- Error (pCi/G)
Thorium-230 EPA 00.07	₹1

VICINITY PROPERTY NO. 1: OUTER ZONE

METALS EPA CLP METHOD	CONCENTRATION UG/G
Aluminum Antimonv Arsenic Barium Beryllium Cadmium Calcium Chromium Cobalt Copper Iron Lead Lithium Magnesium Manganese Mercurv Nickel Potassium Selenium Silver Sodium Thallium Vanadium Zinc	10535 {4.0} 15.7 222 0.8 2.5 2043 18.9 14 13 1928 27.2 6U 1632 1268 0.1U 14 726 0.6U 1.5 {429} 1U 39 43
Nitroaromatics USATHAMA METHOD	CONCENTRATION UG/G
DATE EXTRACTED :08/19/87 DATE ANALYZED :09/18/87	
2,4.6-TNT 2,4 DNT 2.6 DNT Nitrobenzene 1,3.5-Trinitrobenzene 1,3-Dinitrobenzene	1.41 0.880 1.45 1.69 0.670 1.06
MISCELLANEOUS	PERCENT
Fercent Moisture	14.9

VICINITY PROPERTY NO. 1: OUTER ZONE

SEMI-VOLATILES EPA CLP METHOD

DATE EXTRACTED: 08/18/87 DATE ANALYZED: 09/03/87

CONCENTRATION UG/KG

Phenol bis(2-Chloroethvl) ether 2-Chlorophenol 1.3-Dichlorobenzene 1.4-Dichlorobenzene Benzyl Alcohol 1.2-Dichlorobenzene 2-Methvlphenol bis(2-Chloroisopropvl) ether 4-Methvlphenol N-Nitroso-Dipropvlamine Hexachloroethane Nitrobenzene Isophorone	388 388 388 388 388 388 388 388 388 388
2-Nitrophenol	
2.4-Dimethyphenol	388 U 388 U
Benzoic Acid	1879 U
bis(2+Chloroethoxy) methane	388 U
2.4-Dichlorophenol	388 U
1,2,4-Trichlorobenzene	388 U
Naohthalene \	388 U
4-Chloroaniline	388 U
Hexachlorobutadiene	388 U
4-Chloro-3-methylphenol	
(para-chloro-meta-cresol)	388 U
2-Methylnaphthalene	388 U
Hexachlorocyclopentadiene	388 U
2,4.6-Trichlorophenol	388 U
2,4,5-Trichlorophenol	1879 U
2-Chloronaphthalene 2-Nitroaniline	388 U
Dimethvl Phthalate	1879 U
Acenaphthylene	388 U
2.6-Dinitrotoluene	388 U 388 U
3-Nitroaniline	1879 U
Acenaphthene	388 U
2,4-Dinitrophenol	1877 U
4-Nitrophenol	1879 U
Dibenzofuran	388 U
2,4-Dinitrotoluene	388 U
Diethvlphthalate	388 U
4-Chlorophenyl Phenyl Ether	388 U
Fluorene	388 U
4-Nitroaniline	1879 U

VICINITY PROPERTY NO. 1: OUTER ZONE

4.6-Dinitro-2-methylphenol N-nitrosodiphenylamine	1879 388	
4-Bromophenyl Phenyl ether	388	
Hexachlorobenzene	388	U
Pentachlorophenol	1879	U
Phenanthrene	388	U
Anthracene	388	U
Di-n-butylphthalate	670	
Fluoranthene	388	-
Fyrene	388	_
Butyl Benzyl Phthalate	388	-
3.3′-Dichlorobenzidine	775	
Benzo(a)anthracene	388	
Chrysene	388	_
bis(2-ethylhexyl)phthalate	388	_
Di-n-octyl Phthalate	388	U
Benzo(b)fluoranthene	388	U
Benzo(k)fluoranthene	388	U
Benzo(a)pyrene	388	U
Indeno(1,2,3-cd)pyrene	388	IJ
Dibenzo(a.h)anthracene	388	U
Benzo(g,h,i)pervlene	388	U

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		METHUD	CONCENTRATION	ש / טט

DATE ANALYZED :08/30/87

Aroclor-1016	<10
Araclar-1221	<10
Aroclor=1232	<10
Aroclor-1242	<10
Aroclor-1248	<10
Aroclor-1254	<10
Aroclor-1260	<10

Radiochemical	Activity	+/- Error
		(pCi/G)
Thorium-230 EPA 00.07		< 1

VICINITY PROPERTY NO. 1: PERIMETER

METALS EFA CLF METHOD	CONCENTRATION UG/G
Aluminum Antimonv Arsenic Barium Beryllium Cadmium Calcium Chromium Cobalt Copper Iron Lead Lithium Magnesium Manganese Mercury Nickel Potassium Selenium Silver Sodium Thallium Vanadium Zinc	13243 {5.4} 19.2 310 1.01 3.8 3869 31 53 60.4 23408 76 8.4 2943 2044 0.1U 36.6 907 0.5U 3.8 {408} 1U 44.3 116
Nitroaromatics USATHAMA METHOD	CONCENTRATION UG/G
DATE EXTRACTED :08/19/87 DATE ANALYZED :09/18/87	
2.4.6-TNT 2.4 DNT 2.6 DNT Nitrobenzene 1.3.5-Trinitrobenzene 1.3-Dinitrobenzene	1.35 0.842 1.58 1.62 0.640 1.01
MISCELLANEOUS	FERCENT
Fercent Moisture	11.0
Radiochemical	Activity +/- Error
Thorium-230 EPA 00.07	(pCi/G) 2.4 +/- 0.4

METALS EPA CLP METHOD	CONCENTRATION UG/G
Aluminum Antimonv Arsenic Barium Bervllium Cadmium Calcium Chromium Cobalt Copper Iron Lead Lithium Magnesium Manganese Mercury Nickel Potassium Selenium Silver Sodium Thallium Vanadium Zinc	9441 24.5 18 187 1.2 25.7 16306 585 30.6 25460 104 16 2766 818 0.10 91 617 0.5 425 10 89
Nitroaromatics USATHAMA METHOD :	CONCENTRATION UG/G
DATE EXTRACTED: 08/19/87 DATE ANALYZED: 09/18/87	
2.4.6-TNT 2.4 DNT 2.6 DNT Nitrobenzene 1.3.5-Trinitrobenzene 1.3-Dinitrobenzene	1.26 0.787 1.48 1.51 0.600 9.44
Miscellaneous	PERCENT
Percent Moisture	4.7

SEMI-VOLATILES EPA CLF METHOD

DATE EXTRACTED: 08/18/87 DATE ANALYZED: 09/03/87

CONCENTRATION UG/KG

	P
Phenol	346 U
bis(2-Chloroethvl) ether	346 U
2-Chlorophenol	346 U
1.3-Dichlorobenzene	346 U
1.4-Dichlorobenzene	346 U
Benzyl Alcohol	346 U
1.2-Dichlorobenzene	346 U
2-Methylphenol	346 U
bis(2-Chloroisopropyl) ether	346 U
4-Methylphenol	346 U
N-Nitroso-Dipropylamine	346 U
Hexachloroethane	346 U
Nitrobenzene	346 U
Isophorone	346 U
2-Nitrophenol	346 U
2,4-Dimethyphenol	346 U
Benzoic Acid	1579 U
bis(2-Chloroethoxy) methane	346 U
2,4-Dichlorophenol	346 U
1.2,4-Trichlorobenzene	346 U
Naphthalene	346 U
4-Chloroaniline	346 U
Hexachlorobutadiene	346 U
4-Chloro-3-methylphenol	2.5
(para-chloro-meta-cresol)	346 U
2-Methylnachthalene	346 U
Hexachlorocyclopentadiene	346 U
2.4.6-Trichlorophenol	346 U
2,4,5-Trichlorophenol	1679 U
2-Chloronaphthalene	346 U
2-Nitroaniline	1679 U
Dimethvl Fhthalate	346 U
Acenaphthylene	346 U
2.6-Dinitrotoluene	346 U
3-Nitroaniline	1679 U
Acenaphthene	346 U
2,4-Dinitrophenol	1679 U
4-Nitrophenol	1679 U
Dibenzofuran	346 U
2.4-Dinitrotoluene	346 U
Diethylphthalate	
4-Chlorophenyl Fhenyl Ether	346 U
4-Chlorophenyl Fhenyl Ether Fluorene	346 U 346 U
	346 U

4.6-Dinitro-2-methylphenol N-nitrosodiphenylamine 4-Bromophenyl Phenyl ether Hexachlorobenzene Pentachlorophenol Phenanthrene Anthracene Di-n-butylohthalate Fluoranthene Pyrene Butyl Benzyl Phthalate 3.3'-Dichlorobenzidine Benzo(a)anthracene Chrysene bis(2-ethylhexyl)phthalate Di-n-octyl Phthalate Benzo(b)fluoranthene Benzo(k)fluoranthene Benzo(a)pyrene Indeno(1,2,3-cd)pyrene Dibenzo(a,h)anthracene	1679 346 346 346 346 346 346 346 346 346 346
·	

PCB'S	EPA	CLP	METHOD	CONCENTRATION	UG/G
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DATE ANALYZED :08/30/87

Aroclor-1016	<10
Aroclor-1221	<10
Aroclor-1232	<10
Aroclor-1242€	<10
Aroclor-1248	<10
Aroclor-1254	<10
Aroclor-1260	<10

Radiochemical	Activity	+/- Error
		(pCi/G)
Thorium-230 EPA 00.07		2.9 +/- 0.5

METALS EFA CLF METHOD	CONCENTRATION UG/G
Aluminum Antimonv Arsenic Barium Beryllium Cadmium Calcium Chromium Cobalt Copper Iron Lead Lithium Magnesium Manganese Mercury Nickel Potassium Selenium Silver Sodium Thallium Vanadium Zinc	1355 12.4 6.7 54 1.1 5.6 39721 42 16 201 34924 226 5U {496} 278 0.1U 66 634 0.5U 3.9 {532} 1U 23 235
Nitroaromatics USATHAMA METHOD	CONCENTRATION UG/G
DATE EXTRACTED :08/19/87 DATE ANALYZED :09/18/87	
2,4.6-TNT 2,4 DNT 2.6 DNT Nitrobenzene 1,3.5-Trinitrobenzene 1,3-Dinitrobenzene	1.37 0.854 1.61 1.64 0.450 1.02
MISCELLANEOUS	PERCENT
Fercent Moisture	12.2
Radiochemical	Activity +/- Error (pCi/G)
Thorium-230 EPA 00.07	2.2 +/- 0.4

METALS	CONCENTRATION MG/KG	
Aluminum Antimonv Arsenic Barium Bervllium Cadmium Calcium Chromium Cobalt Cooper Iron Lead Lithium Magnesium Manganese Mercury Nickel Potassium Selenium Silver Sodium Thallium Vanadium Zinc	9446 14.8 6.0 103 0.9 3.4 39202 28 11 61 13599 87 5U 5603 486 0.29 16 498 0.5UN 4.5 595 1U 38 69 CONCENTRATION UG/G	
DATE EXTRACTED :07/17/87 DATE ANALYZED :07/24/87	DETECTION LIMIT	RESULTS
2.4.6-TNT 2.4 DNT 2.6 DNT Nitrobenzene 1.3.5-Trinitrobenzene 1.3-Dinitrobenzene	1.33 0.83 1.56 1.59 0.63 0.99	ND ND ND ND ND
Miscellaneous	CONCENTRATION MG/KG	
Nitrate Sulfate Choride Fluoride % Moisture	13.7 33.0 18.9 5.9	
Radiochemical	ACTIVITY +/- ERROR (pCi/LITER)	
Thorium 230	12 +/- 2	

RADIOLOGICAL SURVEY
U.S. ARMY RESERVE PROPERTY
WELDON SPRING SITE
ST. CHARLES COUNTY, MISSOURI

Prepared by

E. J. DEMING

Radiological Site Assessment Program
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Prepared for

U.S. Department of Energy as part of the Formerly Utilized Sites - Remedial Action Program

FINAL REPORT

January 1986

This report is based on work performed under contract number DE-AC05-760R00033 with the U.S. Department of Energy.

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RADIOLOGICAL SURVEY U.S. ARMY RESERVE PROPERTY WELDON SPRING SITE ST. CHARLES COUNTY, MISSOURI

INTRODUCTION

From 1957 to 1966 the Weldon Spring Chemical Plant in St. Charles County, Missouri, was used to convert uranium ore concentrates and recycled uranium scrap to uranium tetrafluoride, uranium trioxide, and metallic uranium. Smaller quantities of thorium oxide were also processed at these facilities. The plant was operated by Mallinckrodt Chemical Works - Uranium Division, under contract with the Atomic Energy Commission (AEC). Residues and contaminated wastes from these operations were placed in four onsite raffinate pits. During the same period, and for several years following the termination of uranium and thorium processing operations, the AEC also disposed of residues, contaminated scrap, and building rubble in an abandoned rock quarry, approximately 5 km southwest of the chemical plant site. Since termination of AEC activities in 1969, the chemical plant and the quarry have remained essentially unused and in caretaker status.

As successor to the AEC, the Department of Energy (DOE) is responsible for the management and ultimate disposal of the radioactive wastes from previous operations of the Weldon Spring Chemical Plant. Although wastes are primarily limited to the pits, quarry, and the chemical plant, aerial and surface monitoring have identified low-level contamination on portions of the surrounding properties. Water monitoring has also indicated elevated radionuclide levels in several of the springs and lakes on the adjacent properties. At the request of the Department of Energy's Office of Nuclear Energy, the Radiological Site Assessment Program of Oak Ridge Associated Universities (ORAU) conducted a radiological survey of the United States Army Reserve Property, adjacent to the former Weldon Spring Chemical Plant, to determine the extent and levels of offsite contamination resulting from previous AEC operations and current onsite waste storage.

SITE HISTORY

In 1941, the Department of the Army acquired 6,974 hectares, surrounding what is now the Weldon Spring Chemical Plant, as the site for an explosives production facility known as the Weldon Spring Ordnance Works. Following shut down of Ordnance Works in 1944, land which was declared surplus to army needs was transferred to the Missouri State Conservation Commission for wildlife conservation; the University of Missouri for agricultural purposes; and the St. Charles County Consolidated School District. The remaining land (835 ha) was placed under the control of the Department of the Army. In 1959, 752 ha was designated as a U.S. Army Reserve Training area. Of that amount, 6 ha was transferred to the AEC in 1964 for construction of raffinate pit #4. Currently, the U.S. Army Reserve area is comprised of the remaining 746 ha. There is no history of storage, use or disposal of radioactive material on the Army Reserve Property.

SITE DESCRIPTION

The Weldon Spring site is located on State Highway 94, approximately 22 km southwest of St. Charles, Missouri (Figure 1). The nearest community is Weldon Spring at the intersection of Highway 94 with U.S. Highways 40 and 61. Vicinity properties are indicated in Figure 2. The U.S. Army Reserve Property is a fenced area occupying approximately 746 hectares. The land is relatively level. Portions of the property are cleared and easily accessible while other sections are wooded or overgrown with heavy brush. An extensive system of drainage ditches exists on the property, primarily for surface runoff. main easement, herein called the Southeast Drainage Easement, runs south from the Weldon Spring Chemical Plant to the Missouri River. This easement crosses the southeast corner of the Army Reserve Property for approximately 300 m. site contains numerous paved and unpaved roads, several inactive railroad tracks, a few small lakes or ponds and many small buildings. Remnants of other buildings, previously railroad tracks and associated with manufacturing operations, remain on the property.

SURVEY PROCEDURES

The survey of the Army Reserve Property, Weldon Spring Site was performed by the Radiological Site Assessment Program of Oak Ridge Associated Universities (ORAU) during March-July 1985. The survey was performed in accordance with a survey plan dated December 21, 1984 and approved by the Department of Energy's Office of Nuclear Energy. The methods and procedures utilized in that survey are presented in this section.

Aerial photographs and drawings of the Weldon Spring Site, including the U.S. Army Reserve Property, dating from 1957 to the present were reviewed to aid in determining the best radiological survey approach and most probable sites of contamination.

Field Procedures

- 1. Walkover surface scans were conducted at 1-2 m intervals over interior and perimeter roads, trails, railroads and major surface drainage pathways. Portable gamma NaI(T1) scintillation survey meters were used for these scans. Areas with elevated radiation levels were noted for further definitive surveys.
- 2. Gamma measurements were made at the surface and at 1 m above the surface at 100 m intervals along major roadways, railroads and drainage ditches. These measurements were made either to the left (L), right (R) or in the center (C) of the traverse unless otherwise indicated. The measurements were made using portable NaI(T1) scintillation meters and results were converted to exposure rates in microroentgens per hour (μR/h) by comparison with a calibrated pressurized ionization chamber.

Direct radiation measurements were also made at locations of elevated surface levels as identified by the walkover surface scans.

3. Surface (0-15 cm) soil samples of approximately 1 kg each were collected at 100 m intervals along major roadways, railroads and

drainage ditches. Samples were collected 1 m to the left (L), right (R) or in the center (C) of the traverse unless otherwise indicated.

4. Grid systems were established in two areas where more extensive surface contamination was identified by walkover scans.

Walkover surface scans were conducted at 1-2 m intervals over the entire gridded area using gamma scintillation detectors. Locations of elevated contact radiation levels were noted for further sampling.

Gamma measurements were made at the surface and at 1 m above the surface at each grid intersection and at elevated locations identified by the walkover scan.

Dose rate measurements were measured 1 cm above the surface at each gridline intersection using thin-window ($\langle 7 \text{ mg/cm}^2 \rangle$) G-M detectors and portable scaler/ratemeters. Measurements were also obtained with the detector shielded to evaluate the contribution of nonpenetrating beta and low-energy gamma radiations. Meter readings were converted to dose rates in microrads per hour (μ rad/h).

Surface (0-15 cm) soil samples of approximately 1 kg each were collected at each accessible grid intersection and at selected locations identified by the walkover scan.

5. Nineteen boreholes were drilled on the Army Reserve Property at locations shown on Figure 3. Boreholes were drilled by Continental Drilling using a truck-mounted hollow stem auger. Locations of the boreholes were selected based on site accessibility and results of surface measurements and analysis of surface soil samples. Locations were selected to provide representative coverage of the Army Reserve Property and further examine areas of contamination.

Radiation profiles of the boreholes were determined by measuring gamma radiation at 30 cm intervals from the surface to the bottom of the borehole using a collimated NaI(T1) gamma scintillation probe and portable scaler.

Samples of subsurface soil were obtained at various depths using a split-spoon sampler.

Ground water samples were obtained from five of the boreholes.

Soil samples were collected from shallow boreholes, ranging in depth from the soil surface to 100 cm, at selected locations of elevated surface radiation levels identified by the walkover surface scan.

- 6. Samples of surface water were collected from creeks, ponds, drainage ditches, and easements (Figure 4).
- 7. Sediment samples were obtained at locations of surface water sampling and also at 100 m intervals along major surface drainage ditches.

 Additional sediment samples were collected in areas of contamination identified in the walkover scans.
- 8. Buildings which were intact and accessible were scanned using NaI gamma scintillation meters. Direct alpha and beta-gamma surface measurements were made on floors, walls, ceilings, and equipment. Smear samples were obtained for measurement of transferable alpha and beta contamination. Buildings which were no longer intact and consisted mainly of rubble and concrete were scanned using NaI(T1) gamma scintillation meters. Figure 5 shows the location of buildings on the Army Reserve Property.
- 9. Six soil samples and five water and sediment samples were collected from the Weldon Spring area (but not on the Weldon Spring Chemical Plant or associated vicinity properties) to provide baseline concentrations of radionuclides for comparison purposes. Direct background radiation levels were measured at locations where baseline soil samples were collected. The locations of the baseline samples and background measurements are shown on Figure 6.
- 10. Civil surveys were performed by a local surveying agency to identify areas of contamination relative to the ordnance grid coordinate

system. (Because of the distances of some areas from state grid benchmarks, it was not possible to tie locations to the state grid coordinate system.) Permanent markers were installed at each of the contaminated areas.

Sample Analysis Procedures/Data Analysis Procedures

Soil and sediment samples were analyzed by gamma spectrometry. Radionuclides of interest included U-238, Th-232, and Ra-226, however, the spectra were also reviewed for other gamma emitters. Water samples were analyzed for gross alpha and gross beta concentrations. In addition, water samples which exceeded 15 pCi/1 gross alpha, were analyzed for Ra-226, Ra-228, and isotopic uranium concentrations. Two samples were also analyzed for isotopic thorium. Smear samples were analyzed for transferable gross alpha and beta contamination. Additional information concerning equipment and procedures is contained in Appendices A and B.

Results of this survey were compared to the DOE guidelines for residual radioactivity at formerly utilized and surplus facilities sites. (Refer to Appendix C)

RESULTS

The results of this survey are presented in two sections. Part A provides general survey findings, identifying locations of contamination. Further characterization of these contaminated areas is discussed in Part B.

A. General Survey Results

Background Levels and Baseline Concentrations

Background exposure rates and baseline radionuclide concentrations in soil determined for the vicinity of the Weldon Spring vicinity properties are presented in Table 1A. Exposure rates ranged from 5 to 8 μR/h. Concentration of radionuclides in soil were: Ra-226, 0.55 to 0.92 pCi/g (picocuries per gram); U-238, <0.68 to 1.62 pCi/g; and Th-232, 0.95 to 1.48 pCi/g. These

concentrations are typical of the radionuclide levels normally encountered in surface soils.

Baseline radionuclide concentrations determined in sediment are presented in Table 1B. Radium-226 concentrations ranged from 0.35 to 0.92 pCi/g; U-238 concentrations ranged from <0.66 to 1.4 pCi/g and Th-232 concentrations ranged from 0.24 to 1.02 pCi/g. These concentrations are typical of radionuclides normally encountered in sediment.

Radioactivity levels in baseline water samples are presented in Table 1C. Gross alpha concentrations ranged from 0.48 to 4.09 pCi/1 (picocuries per liter). Gross beta concentrations ranged from 3.50 to 7.39 pCi/1. These are typical concentrations normally occurring in surface water.

Army Property Railroad System

Direct Measurements

Direct radiation levels, measured at 100 m intervals along the entire railway system within the Army property (Figure 7), are presented in Tables 2 and 3. The gamma exposure rates measured at 1 m above the surface ranged from 6 to 13 μ R/h. Surface exposure rates ranged from 6 to 9 μ R/h.

The walkover survey identified two areas of generally elevated surface radiation levels containing numerous "hot spots" (designated as Locations #1 and #2), and one smaller area with a few isolated "hot spots", which has been designated as Location #3.

Radionuclide Concentrations in Surface Soil

Radionuclide concentrations measured at 100 m intervals along the Army Property Railways are presented in Tables 4 and 5. All concentrations measured are within the normal range of baseline samples, with the exception of the concentrations measured along Railroad #2' (refer to Table 5) which is adjacent to an area which has been identified as contaminated (Location #1). Uranium-238 concentrations measured at 50 m intervals along Railroad #2' ranged

from 0.98 to 143 pCi/g. Radium-226 and Th-232 concentrations are within the normal baseline ranges.

Major Army Property Roads

Direct Measurements

Direct radiation levels, measured at 100 m intervals along the major roads (Figures 8, 9 and 10), are presented in Tables 6-12. The gamma exposure rates at 1 m above the surface ranged from 5 to 9 μ R/h and surface contact gamma exposure rates were from 5 to 8 μ R/h.

The walkover survey of the major roads identified one isolated "hot spot", one meter to the north of Road #1, 1154 m from the property entrance gate. This spot has been designated as Location #7 and is shown on Figure 11.

Radionuclide Concentration in Surface Soil

Concentrations of radionuclides measured in surface soil collected at 100 m intervals are presented in Tables 13-19. Levels of Ra-226, U-238 and Th-232 were below the minimum detectable activity or within the range of the baseline samples.

Secondary Army Property Roads

Walkover surveys of approximately 75% of the secondary roadways did not indicate any areas of elevated surface radiation; further measurements and sampling were therefore not performed along these roadways.

Major Surface Drainage Ditches

Direct Measurements

Direct radiation levels measured at 100 m intervals along the major surface drainage ditches (#4, #4A, ditch from Location #1 - Figure 4) are presented in Table 20. Gamma exposure rates measured at 1 m above the surface ranged from 7 to 11 μ R/h and surface measurements ranged from 7 to 14 μ R/h.

Gamma exposure rates measured along a short section of drainage ditch paralleling the underground section of the Southeast Drainage Easement (Figure 17) ranged from 5 to 8 μ R/h at the surface. Because these were background levels and no elevated radiation levels were identified, additional measurements and sampling were not performed along this ditch.

The Southeast Drainage Easement is designated as Location #4 and is discussed in Results Section B.

Two areas of elevated surface radiation levels were identified by the walkover scan. All of the ditch running from the fenceline near the raffinate pit to ditch #4 has been designated as Location #5. The first 200 m of ditch #4, near the Army property fenceline has been designated as Location #6.

Radionuclide Concentrations in Sediment

Concentrations of radionuclides measured at 100 m intervals along the major surface drainage easements are presented in Table 21. Uranium-238 concentrations ranged from 0.46 to 59.2 pCi/g. The latter value was from a sample taken from ditch #4 at the Army property fenceline (Location #6). All other Uranium-238 values were <12.5 pCi/g. Radium-226 and Th-232 concentrations ranged from 0.53 to 1.57 pCi/g and 0.62 to 2.23 pCi/g, respectively.

Radionuclide Concentrations in Water

Radionuclide concentrations measured in water from surface drainage ditches were within the normal baseline values (Table 22).

Schote Creek

Direct Measurements

Direct radiation levels measured at 100 m intervals along Schote Creek are presented in Table 23. Gamma exposure rates measured at the surface and 1 m above the surface did not exceed 8 μ R/h.

Radionuclide Concentrations in Sediment

Radionuclide concentrations measured in sediment collected at 100 m intervals along Schote Creek are presented in Table 23. All concentrations measured were within the range of baseline concentrations.

Army Property Ponds

Radionuclide Concentrations in Surface Water

Radionuclide concentrations measured in two ponds located on the Army Reserve property (Figure 4) are presented in Table 22. Gross alpha and beta concentrations are within the range of baseline concentrations.

Radionuclide Concentrations in Sediment

Sediment samples obtained at locations of surface water sampling contain concentrations within the range normally encountered in baseline samples (Table 25).

Borehole Gamma-Logging Measurements

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The locations of boreholes drilled on the Army Property are shown on Figure 3. Borehole gamma-logging did not identify any elevated subsurface radiation levels. Logging data were not used to quantify radionuclide concentrations in the subsurface soil due to the absence of significant positive findings and the varying ratios of different gamma-emitting radionuclides in soils at this site.

Radionuclide Concentrations in Subsurface Soil from Boreholes

Radionuclide concentrations measured in subsurface soil from boreholes are presented in Table 26. None of the 19 boreholes contained subsurface radionuclide concentrations significantly different from baseline soil concentrations.

Radionuclide Concentrations in Subsurface Water from Boreholes

Gross alpha and beta concentrations in subsurface water samples collected from five boreholes are presented in Table 27. Gross alpha concentrations ranged from 0.60 to 9.10 pCi/1, which is within the range normally encountered in nature.

Building Surveys

The results of the building surveys for buildings 1-3 and 14-21 are summarized in Table 28. No areas of elevated direct radiation or surface contamination were noted. Further building measurements were therefore not necessary.

Walkover surveys of the rubble remaining from buildings 4-13 did not identify any areas of elevated surface measurements. Gamma exposure rates ranged from 5 $\mu R/h$ to 10 $\mu R/h$.

B. Results at Identified Locations of Contamination

Location #1

Location #1 is adjacent to the Bechtel/DOE access road (Figure 7). A 10 m grid system was established in the southeast corner of Location #1 covering from 0 North to 50 North and 0 West to 50 West (Figure 12). Adjacent areas within Location #1 were sampled using either a 20 m or a 40 m grid interval. The larger grid intervals were used as surface radiation levels approached background levels.

Direct Measurements

Direct radiation levels measured at grid intersections are presented in Table 29. The gamma exposure rates measured at 10 m grid intervals 1 m above the surface ranged from 7 to 52 μ R/h. Surface contact gamma exposure rates and beta-gamma dose rates ranged from 6 to 56 μ R/h, and 6 to 1280 μ rad/h, respectively. Within the 20 m and 40 m grids, gamma exposure rates 1 m above

the surface ranged from 4 to 9 μ R/h. At surface contact the exposure rates ranged from 4 to 11 μ R/h and beta-gamma dose rates ranged from 4 to 44 μ rad/h. Measurements performed with the detector shielded averaged approximately 40% less than those with the unshielded detector. This indicates that a small portion of the surface dose rate is due to nonpenetrating beta or low energy photon radiations. Exposure rates and dose rates were highest within the 10 m grid area and dropped to background levels within the 40 m grid section.

The walkover survey identified numerous locations of elevated surface radiation levels. These locations are indicated in Figure 13 and associated direct radiation levels are presented in Table 30. Contact gamma exposure rates ranged from 15 to 660 μ R/h. Gamma exposure rates at 1 m above the surface and contact beta-gamma dose rates ranged from 7 to 59 μ R/h and 610 to 60460 μ rad/h, respectively.

Radionuclide Concentration in Surface Soil from Grid Intersections

Table 31 lists the concentrations of radionuclides measured in surface soil from 10 m, 20 m and 40 m grid intervals. Samples obtained from the 10 m grid area contained Ra-226 concentrations ranging from 0.42 to 18.1 pCi/g. The concentration of U-238 ranged from 0.60 to 1100 pCi/g and Th-232 concentrations ranged from 0.13 to 4.82 pCi/g. Radionuclide concentrations collected from the 20 m and 40 m grid sections are within the range encountered in baseline soil samples.

Radionuclide Concentration in Soil from Boreholes

Radionuclide concentrations measured in soil samples from randomly spaced shallow boreholes are presented in Table 32. Radium-226 concentrations ranged from 0.89 to 1.22 pCi/g. Concentrations of U-238 and Th-232 ranged from 1.01 to 4.35 pCi/g and 0.79 to 1.42 pCi/g, respectively.

Radionuclide concentrations in borehole samples from areas identified as having surface contamination are presented in Table 33. Uranium-238 concentrations ranged from 2.76 to 29,530 pCi/g. Radium-226 concentrations ranged between 0.70 and 40.1 pCi/g. Concentrations of Th-232 ranged from <0.46

to 450 pCi/g. Uranium-238 contamination was noted to at least 1 m below the soil surface.

At many locations of elevated contact radiation levels, pieces of metal, debris, and slag were encountered while sampling, preventing deeper samples from the same area. A 1.7 kg slag sample obtained at 8N, 14W had a total U-238 activity of 1.4 mCi (millicuries).

Radionuclide Concentration in Surface Water

Two samples of surface water were collected from the drainage ditch within Location #1 (Figure 12) and the radionuclide concentrations are presented in Table 34. Gross alpha and beta measurements in the sample collected at 18N, 50W were 3070 pCi/1 and 4220 pCi/1, respectively. Uranium-238 and U-234 concentrations were 1699 pCi/1 and 1643 pCi/1; Ra-226 and Ra-228 were <0.08 pCi/1 and 0.2 pCi/1, respectively. The sample from 38N, 90W had gross alpha levels of 0.90 pCi/1 and gross beta concentrations of 6.43 pCi/1.

Radionuclide Concentration in Drainage Ditch Sediments

Radionuclide concentrations in sediment samples, collected from locations along the drainage ditches within Location #1, are presented in Table 35. Radium-226 concentrations ranged from 0.60 to 1.27 pCi/g. Uranium-238 and Th-232 ranged from 1.09 to 781 pCi/g and 0.48 to 11.40 pCi/g.

Location #2

A 10 m grid system was established along an 80 m x 20 m section of Railroad #2 (Figures 7 and 14), and is designated as Location #2.

Direct Measurements

Direct radiation levels measured at the grid points are presented in Table 36. Contact gamma exposure rates ranged from 6 to 13 μ R/h. Gamma exposure rates at 1 m above the surface and beta-gamma dose rates ranged from 6 to 14 μ R/h and 7 to 120 μ rad/h, respectively.

Direct radiation levels measured at locations identified in the walkover scan are presented in Table 37; their positions in the grid are shown in Figure 15. Gamma exposures rates at 1 m above the surface ranged from 8 to 21 μ R/h. Contact gamma exposure rates and beta-gamma dose rates ranged from 17 to 150 μ R/h and 92 to 4950 μ rad/h, respectively.

Radionuclide Concentration in Surface Soil

The concentrations of radionuclides measured in surface soil collected at grid point intersections are listed in Table 38. The concentration of U-238 ranged from <0.97 to 57 pCi/g. Thorium-232 concentrations ranged from 0.49 to 1.91 pCi/g; Ra-226 concentrations ranged from 0.64 to 1.84 pCi/g.

Radionuclide Concentration in Soil from Boreholes

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Radionuclide concentrations measured in soil samples from shallow boreholes are presented in Table 39. Uranium-238 concentrations ranged from 160 to 1350 pCi/g. Concentrations of Ra-226 and Th-232 ranged from 0.99 to 5.00 pCi/g and 0.88 to 2.28 pCi/g, respectively.

At some locations of elevated contact radiation levels, pieces of metal and debris were encountered within the top 15 cm of soil. In many cases, the contact radiation levels decreased to background levels once the objects were removed. Contamination in most locations extends to at least 30 cm below the soil surface.

Concentrations of radionuclides in soil samples collected from two isolated locations identified in the walkover scan 23 m west of the gridded area are presented in Table 39. Uranium-238 and Ra-226 concentrations ranged from 10.1 to 390 pCi/g and 2.78 to 36.2 pCi/g, respectively.

Location #3

Location #3 is a wooden loading dock situated adjacent to Railroad #2, approximately 450 m from Location #2 (Figure 15). The structure rises approximately 4.5 m above railroad #2. Railroad #2B ends at the top of the loading dock.

Direct Measurements

Gamma exposure rates of 33 $\mu R/h$ were measured in the area directly beneath the corners of the loading dock and 22 $\mu R/h$ was measured on the southeast corner on the top of the loading dock.

Radionuclide Concentration in Surface and Subsurface Soil

Location 3

Radionuclide concentrations measured in samples obtained from the top of the loading dock and directly beneath the corners of the loading dock are presented in Table 40. Concentration of U-238 ranged from 436 to 2640 pCi/g and Ra-226 concentrations ranged from 0.82 to 4.46 pCi/g. Thorium-232 concentrations were below the minimum detectable activity.

The concentration of U-238 in the sample collected at the southeast corner of the top of the loading dock was 1042 pCi/g and the Ra-226 concentration was 4.46 pCi/g. The range of Ra-226 concentration in soil from the shallow borehole dug at the north east corner, lower level, of the loading dock was 0.82 to 3.14 pCi/g. The U-238 concentration ranged from 2640 pCi/g in the top 15 cm of soil to 477 pCi/g in the sample obtained from 45-60 cm below the soil surface.

Location #4

Location #4 is a short section of the main drainage easement running from the Imhoff Tanks within the Weldon Spring Chemical Plant to the Missouri River (Figure 17). It crosses the southeast corner of the Army property and is referred to as the Southeast Drainage Easement. The runoff flows through a 20 cm diameter underground cement pipe and surfaces approximately 200 m southeast of the chemical plant property fenceline. This drainage ditch continues above ground for approximately 305 m to the perimeter fence between the Army Property and Weldon Spring Wildlife Area.

Direct Measurements

Direct radiation levels, measured at soil sampling locations and at locations identified in the walkover scan along the length of the drainage

easement crossing Army Reserve Property are presented in Table 41. Gamma exposure rates at 1 m above the surface ranged from 8 to 29 $\mu R/h$ and exposure rates at the surface ranged from 7 to 120 $\mu R/h$.

Radionuclide Concentration in Soil

Radionuclide concentrations measured in surface soil at 100 m intervals along the easement are presented in Table 42. Concentrations of U-238 ranged from 1.26 pCi/g to 42.0 pCi/g. Radium-226 and Th-232 concentrations were from 0.76 to 8.36 pCi/g and 0.43 to 2.69 pCi/g, respectively.

Radionuclide Concentrations in Sediment

Concentrations measured in sediment samples collected at 100 m intervals are presented in Table 42. Uranium-238 concentration ranged from <1.56 to 39.7 pCi/g and Ra-226 concentration ranged from 4.34 to 6.57 pCi/g. The concentration of Th-232 ranged from 1.41 to 2.14 pCi/g.

Radionuclide Concentrations in Soil from Shallow Boreholes

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Concentrations of radionuclides measured in soil samples from shallow boreholes are presented in Table 43. Five of the six boreholes contained elevated U-238 concentrations ranging up to 1010 pCi/g. Radium-226 concentrations ranged from 2.04 to 210 pCi/g and Th-232 levels ranged from 0.88 to 69.1 pCi/g. Radionuclide concentrations measured in the borehole at the origin of the drainage easement did not differ significantly from concentrations measured in baseline samples. Contamination extends to at least 60 cm below the soil surface in most of the borehole locations.

Radionuclide Concentrations in Surface Water

Two water samples were collected from the southeast drainage easement. The radionuclide concentrations measured in the drainage water are presented in Table 44. Both samples contained elevated gross alpha and beta concentrations. The sample collected at the origin of the easement had a gross alpha

concentration of 147 pCi/l. The sample collected at the property fenceline had a gross alpha concentration of 159 pCi/l. Isotopic uranium, radium and thorium concentrations measured for both samples indicate that uranium is essentially the only contaminant.

Location #5

Location #5 is a surface drainage ditch leading from the raffinate pits to drainage ditch #4 (Figures 11 and 18).

Direct Measurements

Direct radiation levels measured at 100 m intervals along the drainage ditch are presented in Table 45. Gamma exposure rates were from 6 to 8 $\mu R/h$ at the surface and 6 to 13 $\mu R/h$ measured 1 m above the surface.

Direct measurements from areas of elevated surface readings identified in the walkover scan are presented in Table 46. The highest surface gamma exposure rate (45 μ R/h) was measured at 202 m from the origin of the ditch.

Radionuclide Concentrations in Sediment

Radionuclide concentration in sediment samples collected at 100 m intervals are presented in Table 47. Uranium-238 concentrations ranged from <1.05 to 6.00 pCi/g; Ra-226 and Th-232 concentrations were from 0.61 to 8.22 pCi/g and 0.61 to 1.24 pCi/g, respectively.

Concentrations of radionuclides measured in sediment from areas of elevated surface contamination are presented in Table 48. Radium-226 concentrations ranged from 0.94 up to 62.6 pCi/g. U-238 and Th-232 levels were within the levels normally encountered in baseline samples.

Radionuclide Concentrations in Surface Water

Radionuclide concentration measured in a water sample obtained at the culvert 22 m from the origin of the ditch are presented in Table 22. Gross alpha concentration was 4.28 pCi/l and gross beta concentration was 3.22 pCi/l.

Location #6

Location #6 consists of approximately 200 m of Ditch #4 beginning at the Bechtel/DOE fenceline (Figures 11 and 19).

Direct Measurements :

Direct radiation levels measured at locations identified in the walkover scan of ditch #4 are presented in Table 49. The surface gamma exposure rates were 14 μ R/h and 15 μ R/h measured at 95 m and 160 m, respectively. The exposure rate did not change significantly after a 1 kg surface soil sample was removed.

Radionuclide Concentrations in Soil from Shallow Boreholes

The radionuclide concentrations measured in two boreholes (at 95 m and 160 m) are presented in Table 50. Uranium-238 concentrations ranged from 40.1 to 123 pCi/g. Thorium-232 and Ra-226 concentrations were within the range normally encountered in baseline samples.

Location #7

Location #7 is an isolated "hot spot" located 1 m to the north of Road #1, 1154 m from the entrance to the Army Reserve Property.

Direct Measurements

Table 51 presents the direct radiation levels measured at this location. A maximum surface dose rate measurement of 500 µrad/h was recorded and the maximum surface exposure rate measured was 290 µR/h.

Radionuclide Concentrations In Soil From Shallow Boreholes

Radionuclide concentrations from a shallow borehole at location #7 indicate that the Ra-226 contamination is confined to the top 15 cm of the soil surface. The maximum Ra-226 concentration measured at this location was 215 pCi/g.

Uranium-238 and Th-232 concentrations did not vary significantly from baseline levels. The results from the analysis of soil samples obtained at Location #7 are presented in Table 52.

Th-230 Analyses on Selected Samples

Thorium-230 analysis was performed on selected samples from areas which had elevated uranium concentrations. Thorium-230 concentrations ranged from 1.39 $^+$ 0.13 pCi/g to 53.4 $^+$ 0.8 pCi/g (Table 53).

COMPARISON OF SURVEY RESULTS WITH GUIDELINES

The guidelines applicable to cleanup of vicinity properties at the Weldon Spring Site are presented in Appendix C. The Department of Energy criteria for residual uranium in soil specified for the Weldon Spring Vicinity Properties, is 60 pCi/g of U-238 above background provided that the isotopic uranium distribution is normal. The exposure rate criteria at 1 m above the surface is $20~\mu\text{R/h}$ above background or $27~\mu\text{R/h}$ for the Weldon Spring area. Radiation levels and radionuclide concentrations exceed these guideline values at seven locations.

Exposure rates at 1 m exceed the 27 µR/h criteria at Location #1 and at one sampling point (10 m) of Location #4 (Table 41). Uranium-238 concentrations in soil samples collected from locations of elevated contact radiation levels exceed the maximum concentrations in samples collected at all of the seven contaminated locations. Subsurface soil concentrations exceed the criteria for uranium at Locations #1, 2, 3, 4, and 6. Pieces of metal and rock prevented examination of the depth of contamination below approximately 1 m at Location #1. Simple regression of the data from several sample points within Location #1 indicate that contamination levels will likely decrease to below the DOE criteria for uranium within the first 125 cm of soil. The soil criteria for Ra-226 (5 pCi/g) are exceeded at Locations #5 and #7 to a maximum depth of approximately 1 m. Thorium-230 measured in selected samples from several different locations was found to exceed the guideline of 5 pCi/g at location #1. Uranium concentrations measured in these same samples are orders of magnitude greater then the measured Th-230 concentrations, therefore,

uranium-238 will be the determining guideline factor. Table 54 and Figures 20-25 summarize the areas of residual contamination exceeding the DOE guidelines. The estimated total volume of soil to be removed to meet the guidelines is approximately $6 \times 10^3 \text{ m}^3$.

The two samples of surface water from the southeast drainage easement and one sample from the ditch within Location #1 contained gross alpha concentrations in excess of EPA guidelines (15 pCi/1)1. Combined Ra-226 plus Ra-228 concentrations in water samples were less than 5 pCi/1. Isotopic analyses for uranium indicate that the total concentrations of U-234 plus U-238 exceed the measured gross alpha concentrations. When the uranium isotopic concentrations are subtracted from the total gross alpha concentration measurement the values are less than 15 pCi/1. It should be noted that gross alpha measurements are determined by calibrating against an alpha energy which is higher than the actual uranium energy; therefore, the gross alpha measurements will be slightly lower than the total isotopic measurements. The EPA standards are used here for comparison purposes only, because this water does not represent a source of drinking water.

SUMMARY

A radiological survey of the U.S. Army Reserve property located at Weldon Spring, Missouri was conducted at the request of the U.S. Department of Energy. The survey included surface radiation scans, measurements of direct radiation levels, and analysis of radionuclide concentrations in soil and water, both surface and subsurface. In addition, sediment samples from drainage ditches were analyzed and representative buildings (intact and destroyed) were surveyed.

The results of the survey identified seven areas of U-238, Ra-226, and/or Th-232 contamination exceeding DOE guidelines. Subsurface sampling and measurements indicate that contamination is generally limited to the upper 1 m of soil, although contamination at Location #1 may extend to approximately 1.25 m below the surface. Reduction of contamination to guideline levels would require the removal of approximately 6 x 10³ m³ of material. In addition, portions of the wooden loading dock at Location #3 may need to be removed.

Although there are areas of contaminated residues on portions of this property, the contaminants do not currently pose potential health risks to personnel on the property or to the general public. There is evidence that offsite migration of radioactive materials is continuing via the Southeast Drainage Easement and the drainage ditches from the Raffinate Pit and Ash Pond on the adjacent DOE property.

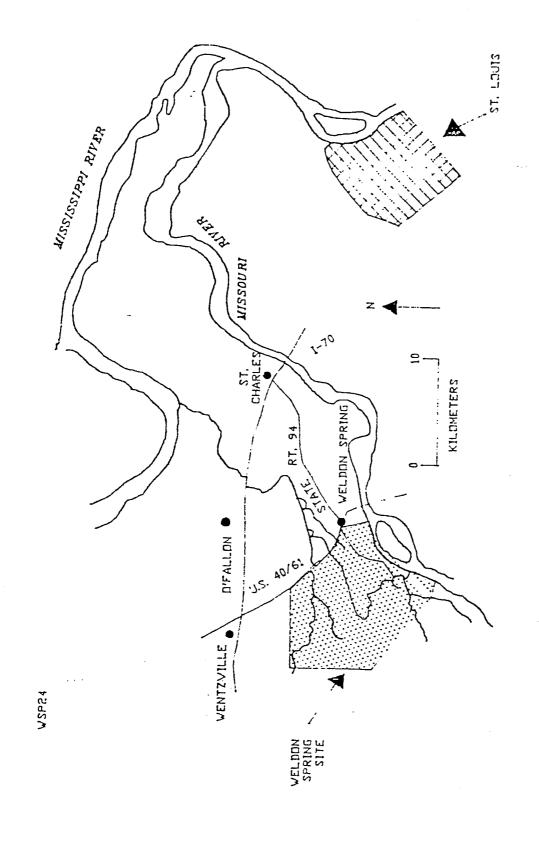


FIGURE 1: Eastern Missouri Indicating the Location of the Weldon Spring Site and Vicinity Properties,

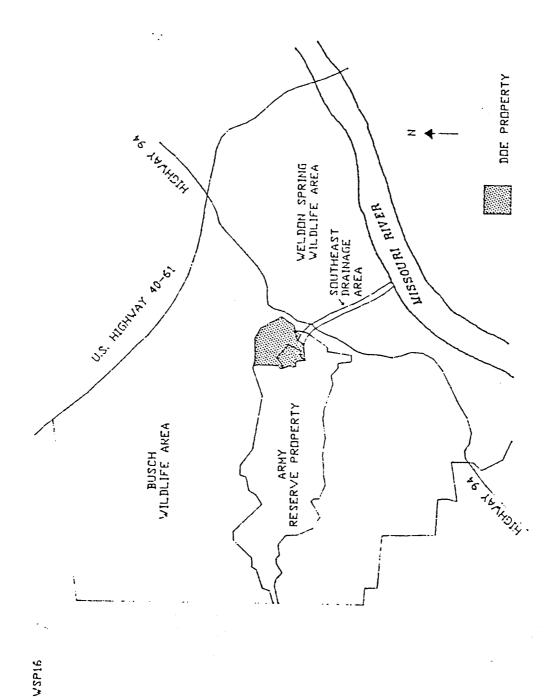


FIGURE 2: Property Bordering U.S. Army Reserve Weldon Spring, Mo.

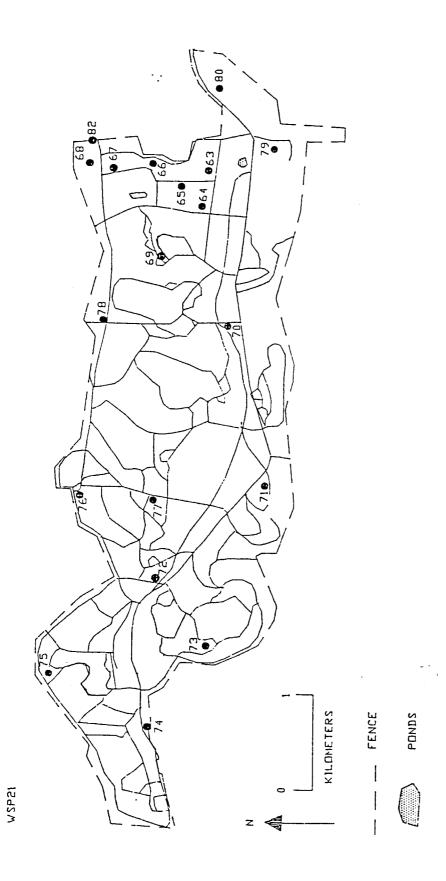
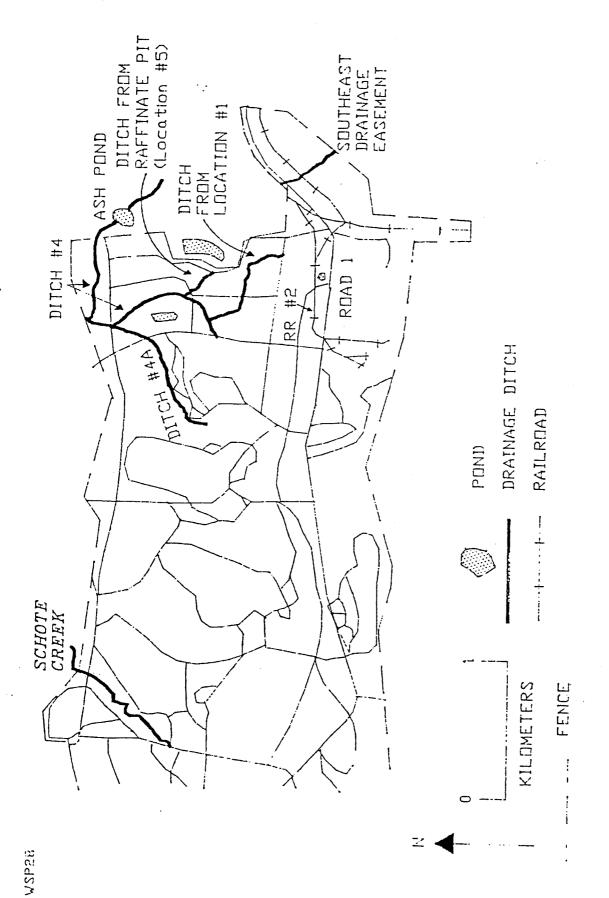


FIGURE 3: Location of Boreholes on Army Reserve Property.



Ditches. FIGURE 4: Location of Major Surface Drainage

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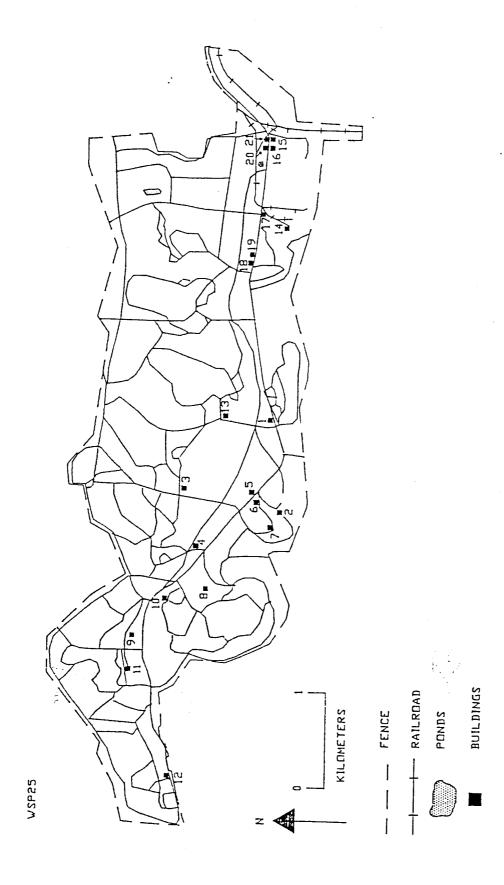
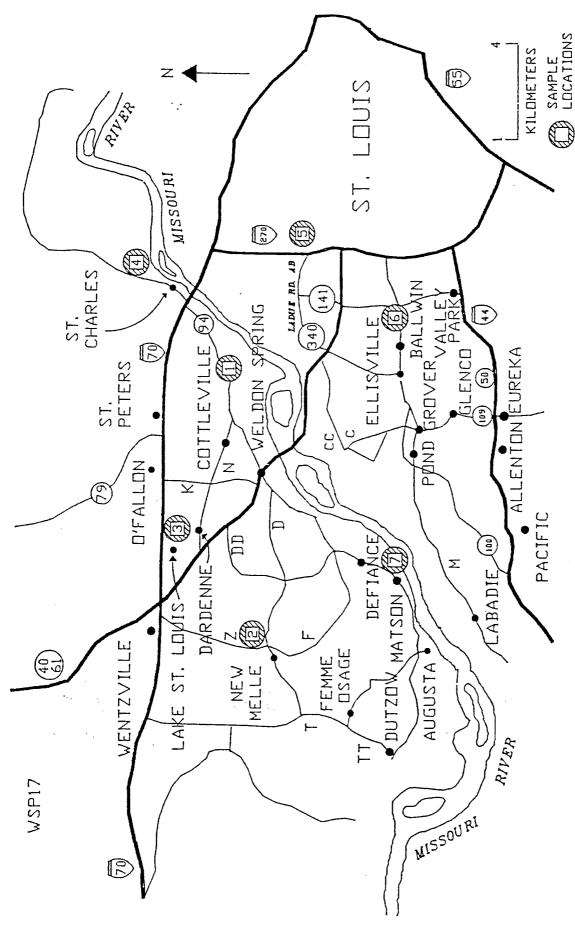


FIGURE 5: Location of Buildings on Army Reserve Property,



Baseline Samples, Locations of of St. Louis Area Showing Background Measurements and Map 9

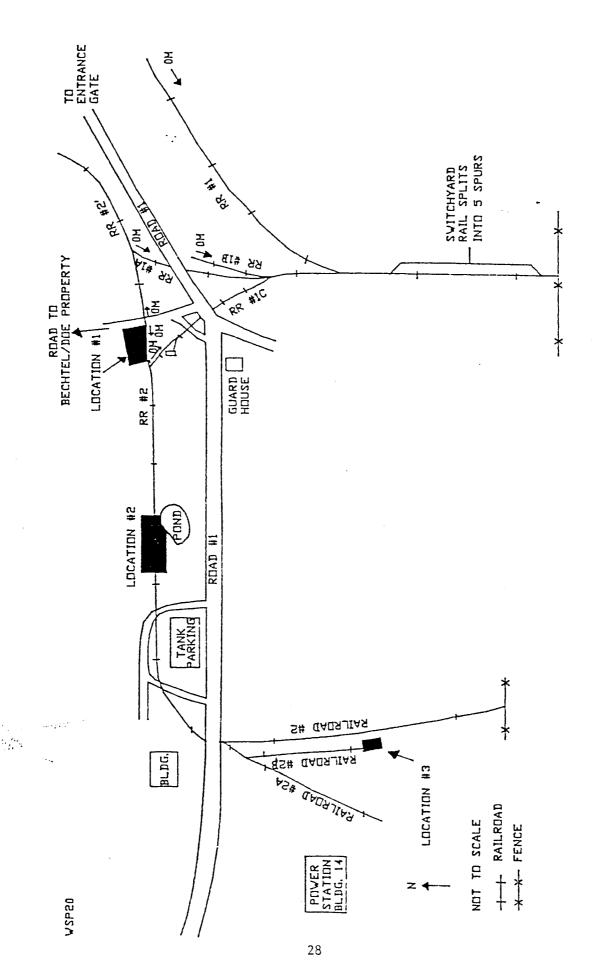


FIGURE 7: Location of Army Railroad #1, #1A, #1B, #1C, #2B, and #2'. #2, #2A,

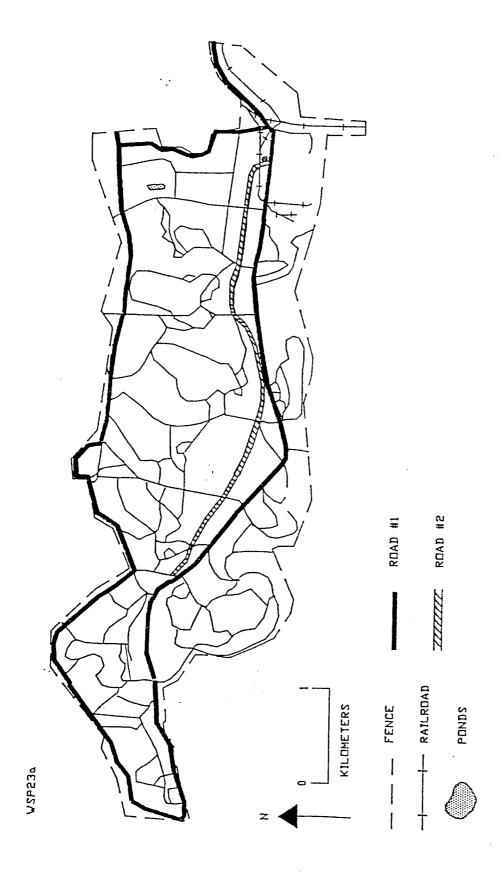


FIGURE 8: Location of Major Roadways on Army Reserve Property.

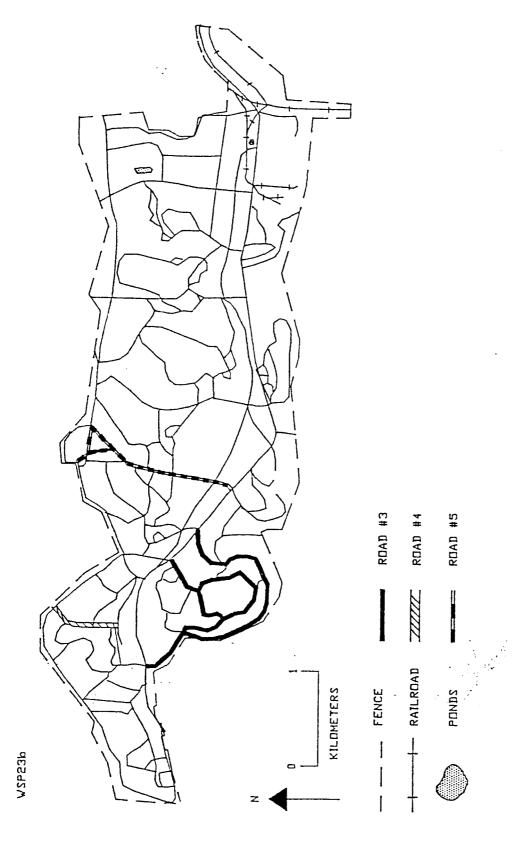


FIGURE 9: Location of Major Roadways on Army Reserve Property,

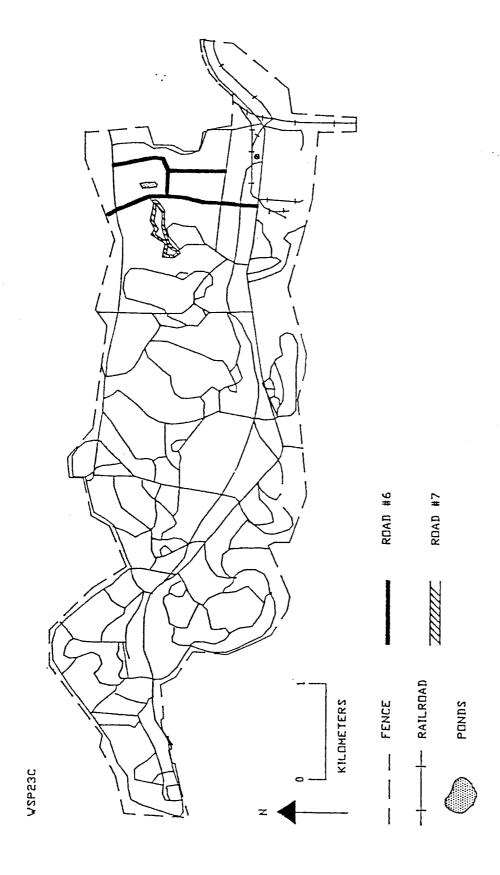


FIGURE 10: Location of Major Roadways Army Reserve Property,

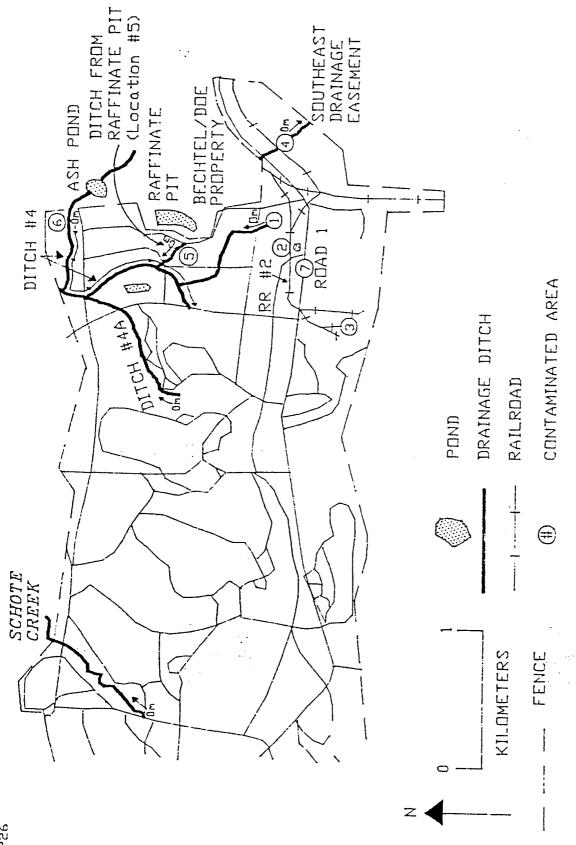


FIGURE 11 Location of Contaminated Areas On Army Reserve Property.

WSP26

:

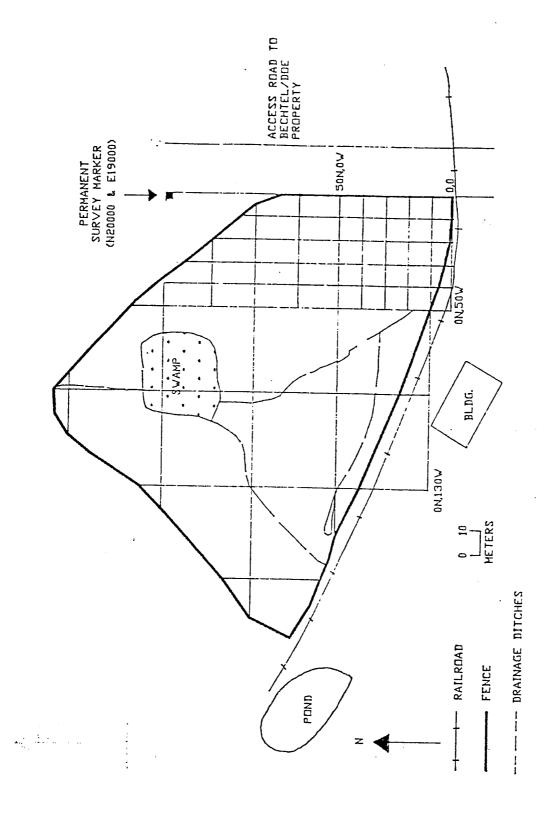


FIGURE 12: Location #1--Grid Established for Survey Reference,

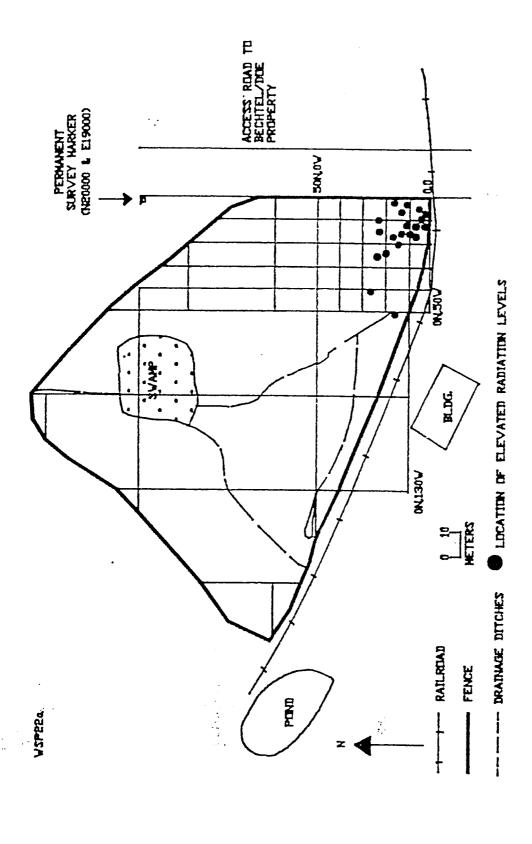


FIGURE 13 Location #1--Locations of Elevated Radiation Identified by Surface Scanning,

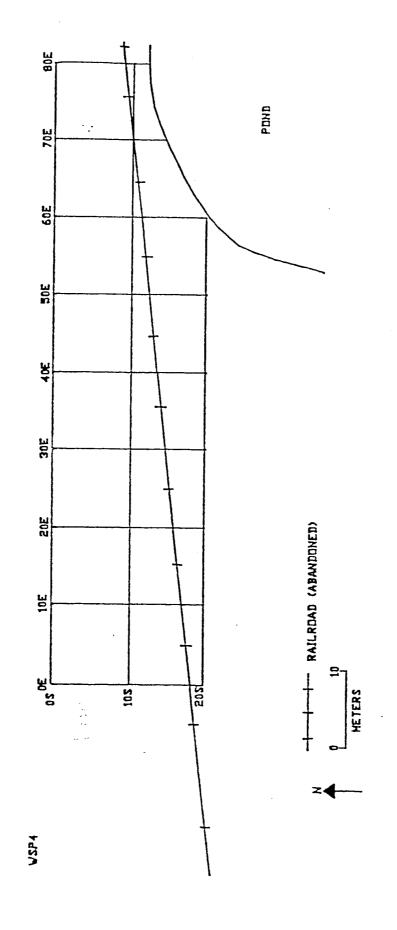
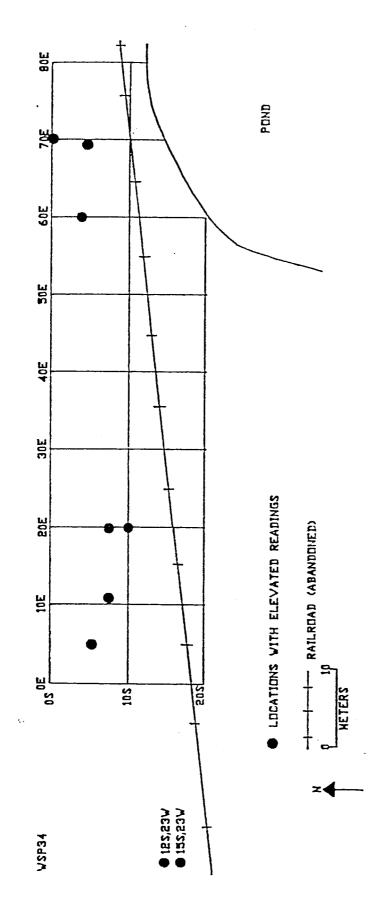


FIGURE 141 Location #2--Grid Established for Survey Reference,



Identified by the Walkover Scan and Systematic FIGURE 15' Location #2--Locations of Elevated Radiation Soil Sampling.

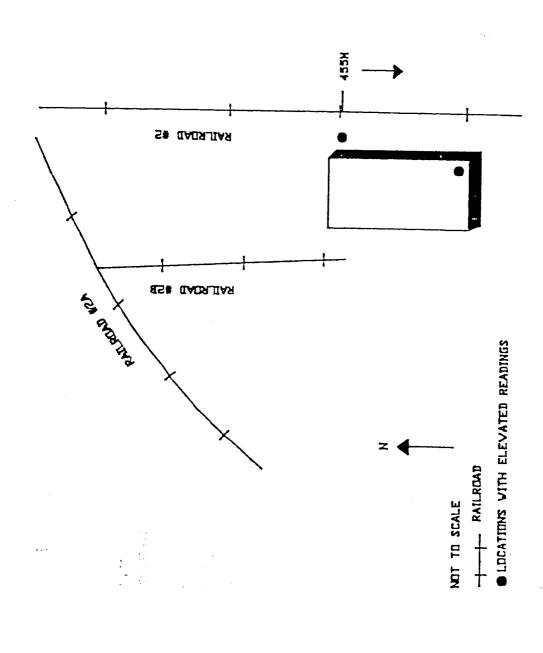
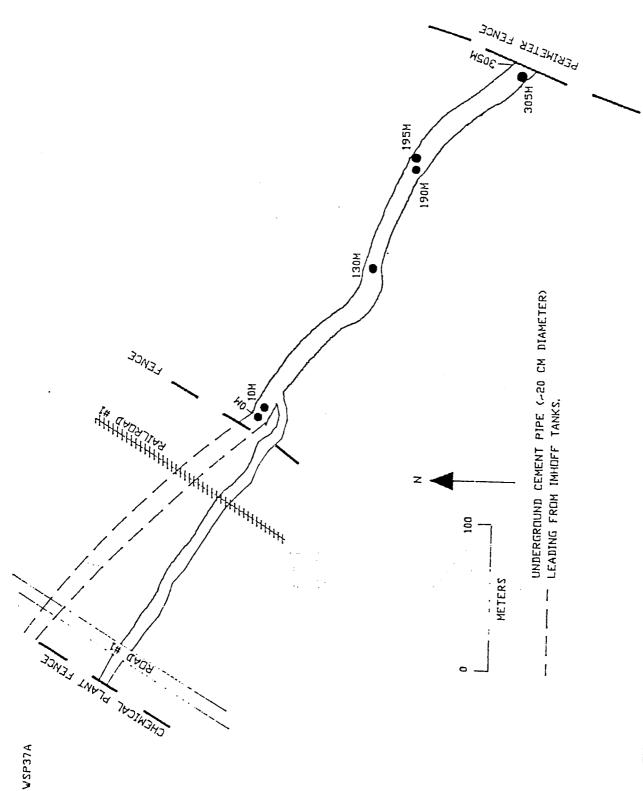
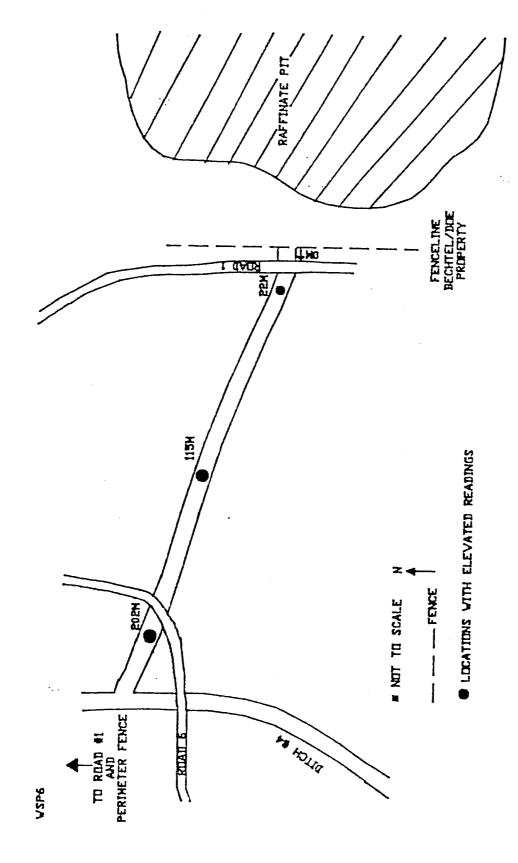


FIGURE 16: Location #3---Loading Dock Area Indicating Locations of Elevated Radiation Identified by the Walkover Scan,

USP35



Easement Indicating the Reference Grid and Locations of Elevated Radiation Identified by the Walkover Scan, FIGURE 17: Location #4--Southeast Drainage



Locations of Elevated Radiation Identified by the Raffinate Pit Indicating the Reference Grid and FIGURE 18' Location #5--Drainage Ditch from DDE Walkover Scan,

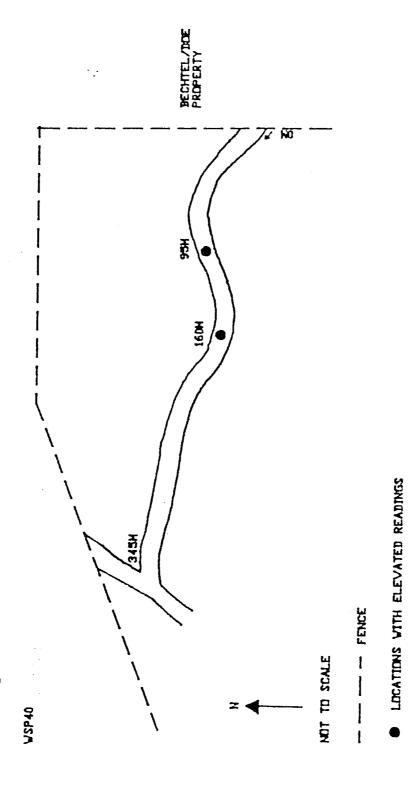
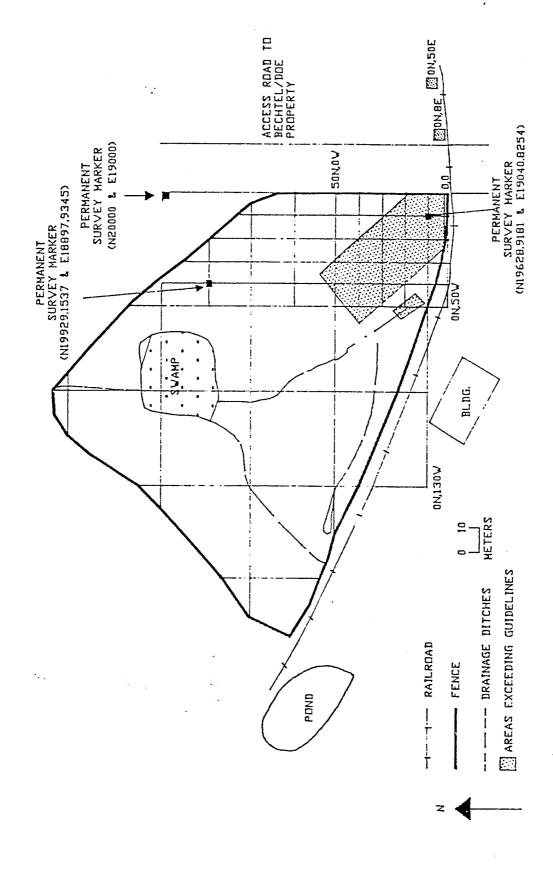


FIGURE 19 Location #6--Drainage Ditch from Ash Pond Indicating Locations of Elevated Radiation Identified by the Walkover Scan,



DDE Guidelines, FIGURE 20: Location #1--Areas Which Exceed the

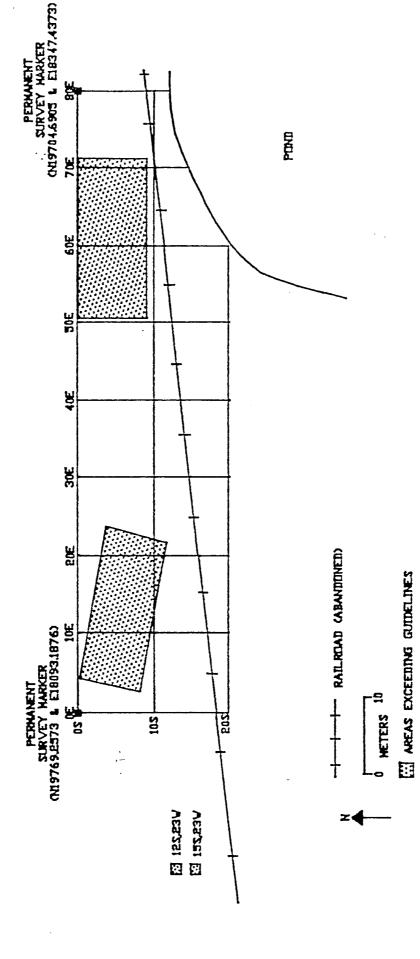


FIGURE 21 Location #2--Areas Which Exceed the DDE Guidelines,

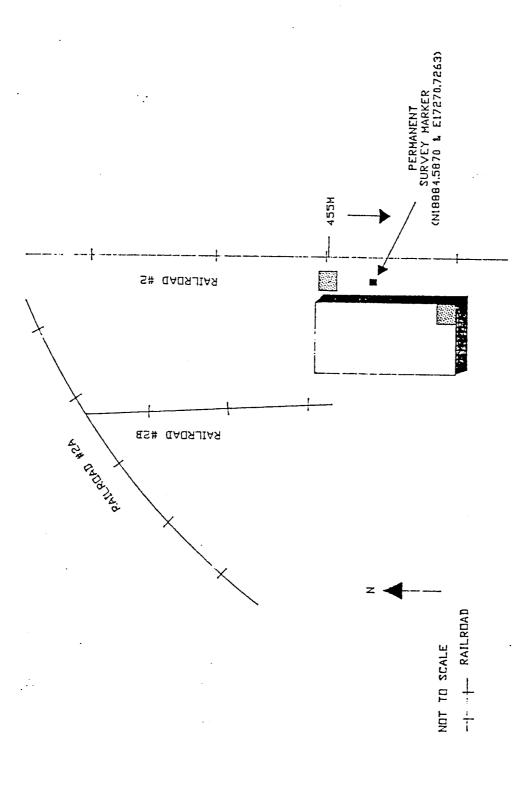
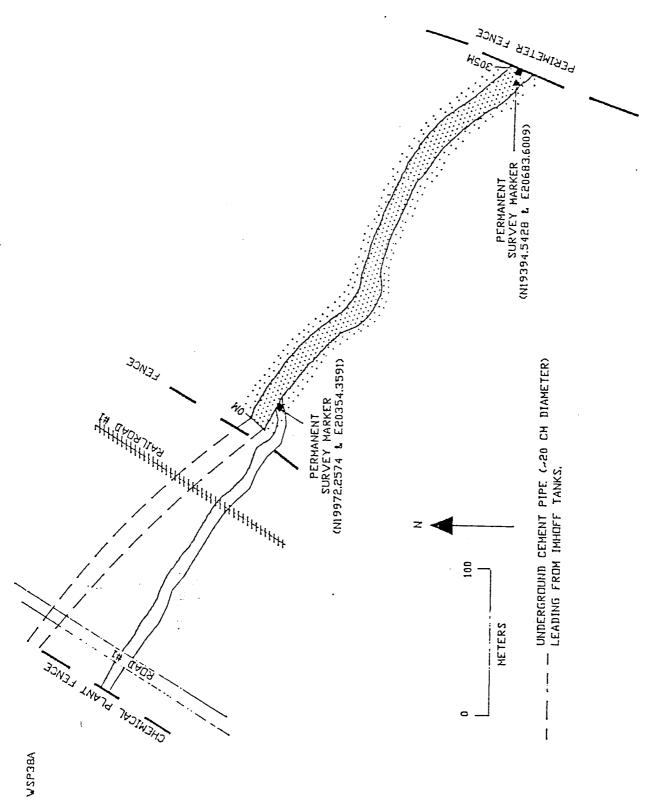
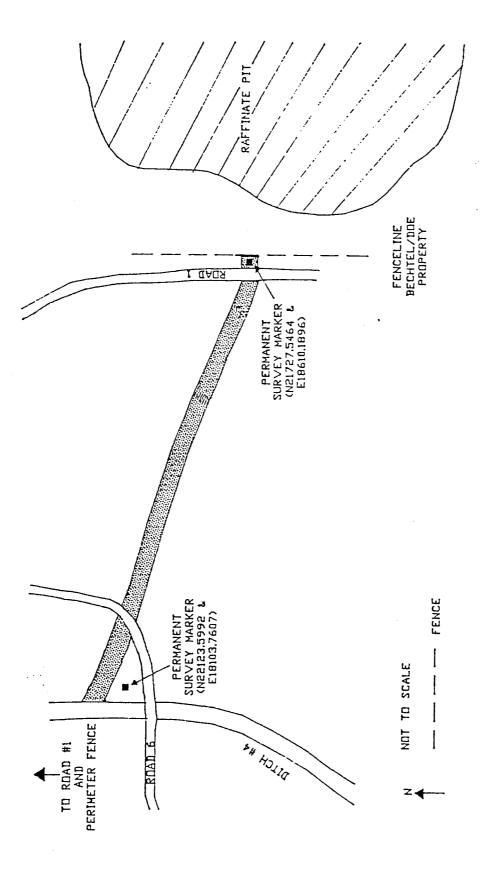


FIGURE 22: Location #3---Loading Dock Areas Which Exceed the DOE Guidelines,



Area Easement Guidelines, FIGURE 23: Location #4---Southeast Drainage DOE Which Exceeds the



DOE Guidelines, 24: Location #5--Area Along the Drainage Ditch Pit Which Exceeds from the Raffinate FIGURE

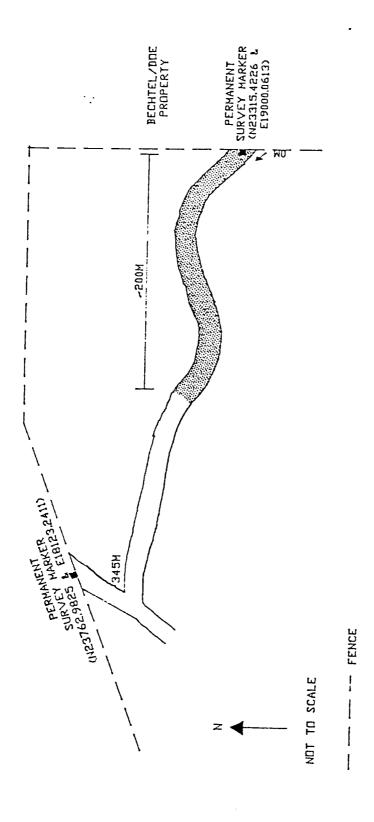


FIGURE 25: Location #6--Area Along the Ash Pond Drainage Ditch Which Exceeds DOE Guidelines,

BACKGROUND EXPOSURE RATES AND
RADIONUCLIDE CONCENTRATIONS IN BASELINE SOIL SAMPLES
WELDON SPRING, MISSOURI

TABLE 1A

Locationa	Exposure Rateb	Radionuclide Concentrations (pCi/g)		
· · · · · · · · · · · · · · · · · · ·	(µR/h)	Ra-226	U-238	Th-232
1	. 7	0.89 + 0.13°	<0.86	1.48 + 0.21
2	7	0.55 ± 0.13	<0.68	0.97 + 0.17
3	5	0.71 ± 0.08	1.33 + 0.74	1.19 + 0.22
4	7	0.98 + 0.15	0.87 ± 0.69	0.95 + 0.26
5	8	0.92 + 0.12	1.52 + 0.83	1.48 + 0.28
6	7	0.69 ± 0.15	1.62 + 0.99	1.36 + 0.32
7	6	<u>_</u> d	= 0.,,	1.30 + 0.32

aRefer to Figure 6.

bMeasured at 1 m above the surface.

CErrors are 20 based on counting statistics.

dDash indicates no sample collected.

TABLE 18 RADIONUCLIDE CONCENTRATIONS IN BASELINE SEDIMENT SAMPLES WELDON SPRING, MISSOURI

Locationa	Radionu	clide Concentration	ons (pCi/g)
٠.	Ra-226	U-238	Th-232
2	0.73 + 0.15b	<0.83	1.02 + 0.19
3	0.35 ± 0.06	0.50 + 0.31	0.24 + 0.07
4	0.92 + 0.12	1.38 + 0.93	0.89 + 0.20
6	0.62 + 0.09	<0.60	0.87 + 0.13
7	0.91 ± 0.09	1.44 ± 0.39	1.02 ± 0.18

aRefer to Figure 6. bErrors are 2g based on counting statistics.

TABLE 1C RADIONUCLIDE CONCENTRATIONS IN BASELINE WATER SAMPLES WELDON SPRING, MISSOURI

ocationa		Radionuclide Concentrations (pCi/l)		
	• :	Gross Alpha	Gross Beta	
2		0.86 + 0.53 ^b	3.68 + 0.95	
3		1.05 + 0.54	4.61 + 0.99	
4		3.19 ± 0.96	7.39 + 1.13	
5		0.48 + 0.44	3.75 ± 0.94	
6		1.02 ± 0.53	3.50 + 0.94	
7		4.09 + 1.06	6.71 + 1.11	

aRefer to Figure 6. bErrors are 2g based on counting statistics.

TABLE 2

DIRECT RADIATION LEVELS MEASURED AT SOIL SAMPLE LOCATIONS RAILROAD #1

WELDON SPRING, MISSOURI

<u>I</u>	ocati (m)	on ^a	Gamma Exposure Rates at 1 m Above the Surface (µR/h)	Gamma Exposure Rates at the Surface (µR/h)
Main Lir	ıe			
(#1)	0	C	8	8
	100	L	7	7
	200	R	7	7
	300	L	7	7
	400	L	7	7
	500	R	7	7
	600	C ·	7	7
	700	L	. 8	8
	800	R	7	7
	900	С	6	6
#1A	0	L	6	6
	100	R	7	6
	200	C	6	6
	300	L	6	6
	350	R	6	6
#1B	0	С	6	6
	100	R	6	6
	127	L	6	6
#1C	0	L	6	6
	100	R	6	6
	200	L	6	6
	300	R	6	6
Switchya	ırd			
(Spurs)	100	R	6	6
	200	L	6	6
	300	R	6	6 6 6
	400	L	6	6
	500	R	6	6
	524	L	6	6

Refer to Figure 7.

TABLE 3

DIRECT RADIATION LEVELS MEASURED AT SOIL SAMPLE LOCATIONS
RAILROAD #2
WELDON SPRING, MISSOURI

<u>I</u>	Locati (m)	on	Gamma Exposure Rates at 1 m Above the Surface $(\mu R/h)$	Gamma Exposure Rates at the Surface (µR/h)
ain Lir	ne			
(#2)	100	L	7	8
	200	C	6	6
	300	R	7	7
	400	L	6	6
	500	L	6	6
	600	R	6	б
	630	С	6	6
#2 A	0	C	6	6
	100	2L	6	7
	200	R	6	6
	300	С	7	6
#2B	0	R	6	6
	100	L	6	6
#2 '	0	L	9	11
	0	R	8	8
	50	L	9	13
	50	R	8	9
	100	L	8	10
	100	R	7	6
	150	L	8	9
	150	R	8	8
	200	L	7	8
	200	R	8	9
	207	L	7	7
	207	R	7	8

aRefer to Figure 7.

TABLE 4

RADIONUCLIDE CONCENTRATIONS IN SURFACE SOIL SAMPLES COLLECTED AT 100 M INTERVALS - RAILROAD #1 WELDON SPRING, MISSOURI

Location ^a .	Radion	uclide Concentrati	ons (pCi/g)
	Ra-226	U−238	Th-232
Mainline			
0, C	$1.34 \pm 0.34b$	8.22 + 3.16	1.27 + 0.58
100, L	0.91 ± 0.32	<0.65	<0.20
200, R	0.95 ± 0.36	1.12 ± 1.30	<0.18
300, L	0.50 ± 0.31	1.33 + 1.54	<0.15
400, L	0.83 + 0.20	<0.70	0.28 ± 0.27
500, R	0.77 + 0.29	<0.87	1.10 ± 0.50
600, C 700, L	0.74 ± 0.33	<0.79	<0.23
800, R	$\begin{array}{c} 0.92 + 0.21 \\ 0.50 + 0.29 \end{array}$	$\frac{2.28 + 1.93}{1.07}$	1.14 + 0.35
900, C	0.30 ± 0.29 $0.88 + 0.32$	1.27 ± 1.97	<0.19
,,,,	0.00 ± 0.32	1.12 ± 1.40	<0.13
Railroad #1A	·		
0, L	0.97 ± 0.28^{b}	<0.75	<0.20
100, R	0.76 ± 0.41	<0.68	<0.20
200, C	0.81 ± 0.27	<0.58	<0.22
300, L	0.66 + 0.32	1.19 ± 1.32	<0.16
350, R	1.45 ± 0.31	<0.62	<0.20
Railroad #1B			
0, C	$0.29 + 0.21^{b}$	<0.57	0.27 + 0.20
100, R	0.41 + 0.27	0.82 + 1.52	<0.18
127, L	0.91 ± 0.31	<0.67	<0.11
Railroad #1C			
0, L	$0.46 + 0.22^{b}$	<0.62	<0.16
100, R	0.34 + 0.15	<0.41	0.24 + 0.17
200, L	0.57 ± 0.23	<0.66	<0.16
300, R	1.58 ± 1.82	1.44 ± 0.81	<0.21
Switchyard Spur .	A		
100, R	$.0.83 + 0.28^{b}$	<0.73	0.50 + 0.28
200, L	; 0.77 + 0.41	1.38 + 1.45	<0.19
300, R	0.77 ± 0.30	<0.72	<0.19
400, L	0.93 + 0.29	1.72 + 1.29	<0.18
500, R	0.88 ± 0.28	<0.67	<0.43
524, L	0.75 ± 0.46	<0.64	<0.28
Switchyard Spur	B		
100, R	0.97 + 0.37b	0.92 + 1.53	<0.21
200, L	1.09 + 0.35	1.60 + 1.39	<0.18
300, R	1.06 ± 0.37	1.78 + 1.57	<0.17
400, L	0.84 ± 0.28	<0.78	<0.21

TABLE 4 (Continued)

RADIONUCLIDE CONCENTRATIONS IN SURFACE SOIL SAMPLES COLLECTED AT 100 M INTERVALS - RAILROAD #1 WELDON SPRING, MISSOURI

Location	Radionu	clide Concentration	ons (pCi/g)
	Ra-226	U-238	Th-232
Switchyard Spur C			:
100, R	$0.75 + 0.31^{b}$	<0.68	<0.21
200, L	0.57 + 0.29	<0.67	<0.19
300, R	0.53 + 0.20	1.49 + 0.93	<0.23
400, L	0.87 ± 0.34	1.79 ± 1.20	<0.24
Switchyard Spur D			
100, R	0.76 + 0.41 ^b		•
200, L	0.48 + 0.39	<0.76	<0.23
300, R	0.54 + 0.21	1.04 + 1.54	<0.16
400, L	0.58 ± 0.29	<0.67	<0.19
Switchyard Spur E			
100, R	$0.96 + 0.30^{b}$	<0.76	<0.23
200, L	0.66 + 0.27	1.28 + 0.90	0.48 + 0.45
300, R	0.83 + 0.27	<0.72	<0.18
400, L	0.91 ± 0.28	1.63 ± 1.30	<0.21

 $^{^{}a}\mbox{Refer}$ to Figure 7. $^{b}\mbox{Errors}$ are 2σ based on counting statistics.

TABLE 5 RADIONUCLIDE CONCENTRATIONS IN SURFACE SOIL SAMPLES COLLECTED AT 100 M INTERVALS - RAILROAD #2 WELDON SPRING, MISSOURI

Locationa	: Radion	uclide Concentratio	nns (nCi/a)
	Ra-226	U-238	Th-232
Mainline			٠.
100, L	1.16 <u>+</u> 0.33 ^b	<1.03	1.51 ± 0.45
200, C	0.46 ± 0.26	<0.76	<0.11
300, R	0.93 ± 0.24	1.51 + 1.99	0.62 ± 0.28
400, L	0.44 ± 0.17	1.62 + 1.64	0.40 ± 0.33
500, L	0.78 ± 0.22	1.48 ± 1.17	<0.18
600, R	0.54 ± 0.20	2.04 ± 2.05	<0.21
630, C	0.52 ± 0.21	1.19 ± 1.40	<0.16
Railroad #2A			
0, C	0.90 ± 0.26^{b}	1.37 ± 1.81	0.32 + 0.26
100, 2L	0.39 ± 0.16	3.95 + 1.84	0.76 + 0.32
200, R	0.36 ± 0.31	<0 . 93	0.59 + 0.32
300, C	0.39 ± 0.35	<1.09	1.41 ± 0.52
Railroad #2B			
0, R	$0.69 + 0.21^{b}$	2.64 + 0.91	0.50 + 0.24
100, L	0.26 ± 0.14	<0.61	<0.13
Railroad #2'			
0, L	0.74 + 0.28	108 + 7	1.10 + 0.39
0, R	1.30 + 0.27	<0.98	2.20 + 0.48
50, L	1.16 ± 0.26	143 + 50	1.39 ± 0.47
50, R	0.68 + 0.30	3.66 + 2.29	1.14 + 0.41
100, L	0.70 ± 0.19	<2.11	1.18 + 0.48
100, R	0.43 ± 0.23	<0.52	<0.11
150, L	0.64 ± 0.28	28.2 <u>+</u> 2.9	1.12 + 0.51
150, R	0.90 ± 0.19	$ \begin{array}{cccccccccccccccccccccccccccccccccccc$	1.07 ± 0.47
200, L	0.51 ± 0.21	0.98 ± 1.55	1.01 ± 0.66
200, R	1.02 ± 0.28	14.5 + 3.3	2.02 + 0.55
207, L	0.83 ± 0.27	8.23 + 2.70	1.12 ± 0.52
207, R	0.77 ± 0.30	10.7 \pm 3.7	1.69 ± 0.52

aRefer to Figure 7. bErrors are 20 based on counting statistics.

TABLE 6

DIRECT RADIATION LEVELS MEASURED AT SOIL SAMPLE LOCATIONS
ROAD #1
WELDON SPRING, MISSOURI

Locati	on a	Gamma Exposure Rates	Gamma Exposure Rates
(m)		at 1 m Above the Surface	at the Surface
		(µR/h)	(μR/h)
0	1 R	7	7
100	1L	8	8
200	1R	8	8
300	1L	8	8
400	1 R	7	7
500	1L	8	8
600	1R	7	7
700	1 L	7	7
800	1 R	7	7
900	1L	7	6
1000	1 R	6	7
1100	1L	7	7
1200	1 R	6	7
1300	1L	. 6	6
1400	1R	7	7
1500	1L	7	7
1600	1 R	. 7	7
1700	1L	7	7
1800	1R	7	7
1900	1L	7	7
2000	1R	7	7
2100	1R	7	7
2200	1L	8	8
2300	1L	7	8
2400	1L	7	7
2500	1 R	7	7
2600	1L	6	6
2700	1 R	7	7
2800	1L	7	7
2900	1R	7	7
3000	1L	7	7
3100	1R	7	7
3200	1L	6	7
3300	1 R	7	7
3400	1L	7	7
3500	1R	6	6
3600	1L	7	8
3700	1R	7	7
3800	1L	7	7
3900	1R	7	7
4000	1L	7	8
4100	1 R	7	7

TABLE 6 (Continued)

DIRECT RADIATION LEVELS MEASURED AT SOIL SAMPLE LOCATIONS ROAD #1 WELDON SPRING, MISSOURI

<u>Locat</u> (m		Gamma Exposure Rates at 1 m Above the Surface (µR/h)	Gamma Exposure Rates at the Surface (µR/h)
4200	1 T	7	
		7	7
4300		7	7
4400		7	7
4500		7	7
4600		6	6
4700		7	7
4800	1L	6	7
4900	1 R	7	7
5000		7	7
5100		7	7
5200		7	7
5300 5400		7	7
5500	1L 1R	6	7
5600		6 7	7
	1R		7
5700 5800	1L	7	7
5 900	1R	7 7	7
6000		7	7
6100		7	7
6200	1L	7	7
6300	1 R	7	7
6400	1L	7	7
6500	1R	7	7
6600	1L	7	7
6700		7	7
6800	1L	6	7 6
6900	1R	7	
7000	1L	7	7
7100		7	. 7
. 7200		7	7 7
7300	1R	7	7
7400	~1L	7	7
7500	1 R	7	7
7600	1L	6	7
7700 7700	1R	6	6
7800	1L	7	7
7900	1 R	7	7
8000	1L	7	7
8100	1R	7	7
8200	1L	7	
8300	1R	7	7 7
24(111)			

TABLE 6 (Continued)

DIRECT RADIATION LEVELS MEASURED AT SOIL SAMPLE LOCATIONS ROAD #1

WELDON SPRING, MISSOURI

Locat (m		Gamma Exposure Rates at 1 m Above the Surface (µR/h)	Gamma Exposure Rates at the Surface (µR/h)
8500	1 R	7	7
8600	1L	7	7
8700	1R	7	7
8800	1L	7	7
8900	1 R	7	7
9000	1L	7	7
9100	1R	7	7
9200	1L	7	7
9300	1R	6	7
9400	1L	6	6
9500	1R	7	7
9600	1L	7	7
9700	1 R	7	7
9800	1L	7	7
9900	1 R	7	7
10000	1L	7	
10100	1R	7	7
10200	1L	7	7
10300	1R	7	7
10400	1L		7
10500	1 R	7	7
10600	1L	7	7
10700	1 R	7	7
10700	1L	6	6
10900	1 R	7	7
11000	1L	7	7
11100	1R	7	7
11200		7	7
11300	1L	7	7
11300	1R	7	7
	1L	7	7
11500 11600	1 R	7	7
	1L	7	7
11700	1R	7	7
11800	1L	7	7
11900	1 R	7	7
12000	1L	7	8
12100	1 R	7	7
12200	1L	7	7
12300	1R	7	8
12400	1L	8	7
12500	1 R	7	7
12600	1L	7	7
12700	1 R	7	7

DIRECT RADIATION LEVELS MEASURED AT SOIL SAMPLE LOCATIONS ROAD #1

TABLE 6 (Continued)

WELDON SPRING, MISSOURI

<u>Locati</u> (m)		Gamma Exposure Rates at 1 m Above the Surface (µR/h)	Gamma Exposure Rates at the Surface (µR/h)
12800	1L	7	7
12900	1 R	7	8
13000	1L	8	8
13100	1 R	7	8
13200	1L	8	8
13300	1 R	7	7
13400	1L	7	7
13500	1R	7	7
13600	1L	8	8
13700	1 R	7	7
13800	1L	7	8
13900	1R	8	7
14000	1L	7	7
14100	1 R	9	8
14200	1 L	7	6

aRefer to Figure 8.

TABLE 7

DIRECT RADIATION LEVELS MEASURED AT SOIL SAMPLE LOCATIONS ROAD #2

WELDON SPRING, MISSOURI

Location ^a (m)	Gamma Exposure Rates at 1 m Above the Surface (μR/h)	Gamma Exposure Rates at the Surface (µR/h)
0	7	7
100	5	6
200	7	7
300	7	7
400	7	7
500	6	7
600	7	8
700	8	8
800	7	8
900	7	7
1000	7	8
1100	7	8
1200	7	7
1300	7	7
1400	6	7
1500	7	7
1600	7	7
1700	7	8
1800	8	8
1900	7	7
2000	7	7
2100	7	7
2200	7	7
2300	7	8
2400	7	7
2500	7	7
2600	7	8
2700	8	9
2800	8	8
2900	7	8
3000	7	8
3100	8	8
3200	7	7

Refer to Figure 8.

TABLE 8

DIRECT RADIATION LEVELS MEASURED AT SOIL SAMPLE LOCATIONS ROAD #3

WELDON SPRING, MISSOURI

Locati (m)	on a	Gamma Exposure Rates at 1 m Above the Surface (µR/h)	Gamma Exposure Rates at the Surface (µR/h)
0	1L	6	6
100	1 R	6	7
200	1L	6	7
300	1 R	7	6
400	1L	6	7
500	1 R	6	6
600	1L	6	6
700	1 R	7	6
800	1L	6	6
900	1 R	6	6
1000	1 L	7	6
1100	1 R	7	6
1200	1L	6	6
1300	1 R	6	6
1400	1L	б	7
1500	1R	7	7
1600	1L	7	7
1700	1R	7	7
1800	1L	6	7
1900	1R	7	7
2000	1L	6	6
2100	1R	7	7
2200	1L	6	6
2300	1 R	6	6
2400	1L	7	7
2500	1 R	7	7
2600	1L	7	7
2700	1R	7	7
2800	1L	7	8
2900	1 R	8	8
3000	1L	7	7
3100	1 R	6	. 6
3200	1L	7	6
3300	1 R	7	6
3400	1L	6	6
3500	1L	7	7
3600	1 R	7	7
3700	1L	6	6

Refer to Figure 9.

TABLE 9

DIRECT RADIATION LEVELS MEASURED AT SOIL SAMPLE LOCATIONS
ROAD #4

WELDON SPRING, MISSOURI

Locat		Gamma Exposure Rates at 1 m Above the Surface (µR/h)	Gamma Exposure Rate: at the Surface (µR/h)
0	1R	7	7
100	.5 L	7	7
200	.5 R	7	7
300	.5 L	7	7
400	.5 R	7	7
500	.5L	7	7
600	1R	6	6
700	·1L	6	7
717	1R	6	6

Refer to Figure 9.

TABLE 10

DIRECT RADIATION LEVELS MEASURED AT SOIL SAMPLE LOCATIONS
ROAD #5
WELDON SPRING, MISSOURI

y ,.	8.		
 Locati (m)	<u>on</u>	Gamma Exposure Rates at 1 m Above the Surface (µR/h)	Gamma Exposure Rates at the Surface (µR/h)
0	1 R	7	7
100	1 L	6	6
200	1 R	6	6
300	1 L	7	7
400	1 R	7	7
500	1 L	7	7
600	1 R	7	7
700	1L	7	7
800	1 R	7	7
900	1L	7	7
1000	1 R	7	7
1100	1L	7	7
1200	1R	6	6
1300	1L	7	6
1400	1 R	7	7
1500	1R	7	7
1600	1L	7	7
1700	1 R	7	6
1800	1L	7	7
1900	1 R	7	7

aRefer to Figure 9.

TABLE 27

RADIONUCLIDE CONCENTRATIONS IN H2O SAMPLES COLLECTED FROM BOREHOLES WELDON SPRING, MISSOURI

Borehole ^a :	Radionuclide Conce	ntrations (pCi/l)
No.	Gross Alpha	Gross Beta
68	9.10 + 1.65 ^b	17.3 + 1.6
73	0.60 + 0.78	2.50 + 1.09
74	4.85 + 1.87	9.67 + 1.76
7 7	4.75 + 1.42	5.55 + 1.25
82	2.05 + 1.45	11.9 + 1.5

a Refer to Figure 3.
b Errors are 2g based on counting statistics.



SUPPLARY OF SURFACE CONTAMINATION MEASUREMENTS IN BUILDING

BUILDING	SURFACE	TOTAL CO (dmp ALPIIA	TOTAL CONTAMINATION (dmp/100cm ²) ALPHA BETA-GAMMA	TRAN CONT (dpm ALPHA	TRANSFERABLE CONTAMINTION (dpm/100cm ²) A BETA-GAMMA	GANMA EXPOSURE RATE (µR/h)
	Walls	<56	<570 - 1720	-1>	<3	. 9 - 5
2	Floors Walls/Equipment	656	<570 - 1000	<1 - 2	\$3	5 - 7
3	Walls	<56	<570 - 930	7	< 3	7 - 8
1.4	Floors Walls	<56 - 150	<570 - 1360	<1 - 2	<3 - 12	5 - 6
1.5	Flodrs	<56	<570 - 720	<u>~</u>	16 - 23	5 - 7
16	Floors	<56	<570 - 720	1>	14 - 21	5 - 6
17	Floors Walls/Equipment	6 – 90 – 90 – 90 – 90 – 90 – 90 – 90 – 9	<570 - 720	₽	6 - 18	5 - 7
18	Floors Walls	<56	<570 - 2220	₩	<3 - 5	6 - 10
19	Floors	<56	<570 - 1650		6 - 26	01 - 9
20	Equipment	<56	<570	1>	11 - 14	5 - 6
21	Floors Equipment	<56	<57 0	<1	12 - 22	2 - 6
	** WITH REAL PROPERTY OF THE P					

ARefer to Figure 5.

TABLE 29

DIRECT RADIATION MEASUREMENTS AT GRID LINE INTERSECTIONS

LOCATION #1

WELDON SPRING, MISSOURI

Gr <u>Loca</u> (m		Gamma Exposure Rates at 1 m Above the Surface (µR/h)	Gamma Exposure Rates at the Surface (µR/h)	Beta-Gamma Dose Rates at 1 cm Above the Surface (µrad/h)
0 N	ΟW	11	9	13
0 N	10W	20	8	8
ON	20₩	9	6	6
ON	30W	8	7	7
0N	40 W	9	9	9
ON	50W	9	9	35
1 N	10W 1	33	21	1280
2 N	20W 1	14	8	11
5 N	30W 1	9	8	8
6 N	40W t	8	7	26
9 N	50W 1	7	. 7	18
10N	1 E	11	11	98
10N	OW	13	9	5 5
10N	10W	52	56	1010
10N	20W	42	43	880
10N	30W	15	20	360
10N	40 W	9	. 9	9
10N	50 W	. 7	7	7
10N	90 W	8	8	21
10N	130W	6	7 .	21
20N	OW	9	8	48
20N	10W	22	26	720
20N	20₩	38	40	1200
20 N	30W	20	12	12
20N	40 W	12	9	9
20N	50 W	11	9	29
25N	90 W 1		6	12
30N	OA	8	8	22
30N	10W	11	9	34
30N	20W	17	22	510
30N	3 O W	22	17	77
30N	40W	11	9	26
30N	50W	9	9	10
40 N	OW	8	8	15
40 N	10W	9	9	16
40 N	20 W	11	9	45
40 N	30W	13	16	320
40 N	40 W	13	15	220
40 N	50W	9	9	33
40 N	130W	6	6	18
50N	OΨ	7	7 .	27

TABLE 29 (Continued)

DIRECT RADIATION MEASUREMENTS AT GRID LINE INTERSECTIONS LOCATION #1 WELDON SPRING, MISSOURI

			•		
	id tion 1)		Gamma Exposure Rates at 1 m Above the Surface (µR/h)	Gamma Exposure Rates at the Surface (µR/h)	Beta-Gamma Dose Rates at 1 cm Above the Surface (µrad/h)
50 N	10W		9	9	25
50 N	20W		9	9	25
50N	30W		9	9	19
50N	40 W		11	11	2 9 6 9
50N	50W		9	8	18
50N	90 W		7	7	23
50 N	130W		7	7	37
50N	150W		6	6	6
50N	160W		7	7	9
57N	170W	ъ	6	6	6
70N	OW		4	4	4
70N	1 W	b	8	8	9
70N	10W	_	8	9	44
70N	20W		9	9	33
70N	30W		9	9	14
70N	40 W		9	9	16
70N	50 W		9	9	40
71N	195W	ъ	7	7	7
90 N	0 W	•	4	4	4
90 N	6 W	ь	8	9	13
90 N	10%	•	8	8	15
90 N	2017		9	9	17
90 N	3 O W		9	9	9
90N	40 W		9	9	15
90 N	50₩		. 9	8	11
90 N	90 W		7	7	10
90 N	130W		7	7	10
90 N	170W		7	8	20
	187W	b	7	7	14
5 95N	10W		8	8	8
102N	170W		8	8	8
106N	20W		8	9	23
110N	30W		8	8	8
110N	40 W		8	9	20
110N	50 W		8	8	21
118N	3 O W	ъ	9	9	26
130N	40 W	-	8	8	18
130N	50 W		8	8	24
130N	90 W		7	7	7
130N	130W		7	8	9
130N	140W		7	7	22
~~ 011	_ , \ , "		•	ı	22

TABLE 29 (Continued)

DIRECT RADIATION MEASUREMENTS AT GRID LINE INTERSECTIONS LOCATION #1 WELDON SPRING, MISSOURI

Gr Loca (m	tion		Gamma Exposure Rates at 1 m Above the Surface (µR/h)	Gamma Exposure Rates at the Surface (µR/h)	Beta-Gamma Dose Rates at 1 cm Above the Surface (µrad/h)	
139N	130W	ъ	7	7	20	
142 N	50 W	ъ	8	8	20	
170N	90 W		. 7	7	10	

Refer to Figure 12.
Measurement taken at fenceline.

TABLE 30 DIRECT RADIATION LEVELS AT LOCATIONS OF ELEVATED SURFACE READINGS LOCATION #1 WELDON SPRING, MISSOURI

rid L	ocation a		ure Rate (μR/h)	Surface Dose Rat	te Contact Exposure
· · · · · · · · · · · · · · · · · · ·	·	Contact	1 m Above Surface	(µrad/h)	Rate After Sample Removal (μR/H)
0 N	8 E	17	7	610	21
0 N	50E	15	8	240	17
2 N	W 8	82	48	3070	93
2 N	14W	410	46	55660	540
4N	9₩	150	51	5370	130
6 N	5₩	200	37	26400	93
8 N	14W	>660 c	5 9	3120	>660 c
8 N	18W	160	42	53 50	130
9 N	17W	190	43	27790	70
11 N	4 W	82	48	1480	190
11 N	11 W	130	53	19900	120
12N	14W	93	40	16830	70
13N	7 W	120	42	9780	93
13N	19₩	420	34	58850	100
18N	2 W	44	15	1130	35
18N	17W	170	37	34120	82
19N	52W	39	14	1180	- đ
20N	25W	59	42	2300	15
21N	14W	42	30	1810	39
23 N	10\	160	15	3480	93
26 N	26₩	240	37	60460	130
27 N	41 W	70	20	1790	130

Refer to Figure 13.
Radionuclide concentrations in samples are presented in Table 33. Observations exceed instrument capability. 1.7 Kg. slag sample collected dat this location contained a total U-238 activity of 31 mCi. No sample collected.

TABLE 31

RADIONUCLIDE CONCENTRATIONS IN SURFACE SOIL SAMPLES COLLECTED FROM GRID POINT INTERVALS LOCATION #1 WELDON SPRING, MISSOURI

				·
Gr	ida	Radion	uclide Concentration	s (pCi/g)
Loca	tion	Ra-226	U-238	Th-232
	W	1.0	3 230	
14	"			•
				
0,	0	$0.92 + 0.24^{b}$	15.2 + 2.8	0.31 + 0.38
		0.44 + 0.14		
0,	10		6.07 + 1.61	0.26 + 0.33
0,	20	0.42 ± 0.14	1.37 ± 0.69	0.25 ± 0.15
0,	30	0.71 ± 0.27	1.48 ± 1.25	<0.14
0,	40	2.27 + 0.48	3.16 + 2.43	2.16 + 0.63
0,	50	1.03 ± 0.28	9.22 + 3.09	1.12 + 0.65
,				
1,	10c	6.48 ± 0.62	668 <u>+</u> 7	2.45 ± 0.63
2,	20°	0.71 <u>+</u> 0.26	6.42 <u>+</u> 1.83	0.13 ± 0.14
5,	30c	0.53 <u>+</u> 0.17	1.25 <u>+</u> 1.30	<0.10
6,	40°	0.60 <u>+</u> 0.15	0.86 <u>+</u> 0.59	0.40 <u>+</u> 0.20
9,	50°	0.85 <u>+</u> 0.20	2.07 <u>+</u> 0.80	0.70 <u>+</u> 0.29
10, 10, 10, 10, 10, 10,	0 10 20 30 40 50 90	$ \begin{array}{c} 1.71 & + & 0.30 \\ 4.93 & + & 0.70 \\ 18.1 & + & 1.0 \\ 1.08 & + & 0.33 \\ 1.27 & + & 0.40 \\ 1.14 & + & 0.24 \\ 0.85 & + & 0.29 \\ 0.64 & + & 0.20 \end{array} $	$ \begin{array}{r} 83.4 & \pm & 2.5 \\ 1,100 & \pm & 16 \\ 694 & \pm & 11 \\ 198 & \pm & 4 \\ <1.24 \\ 1.79 & \pm & 0.84 \\ 0.92 & \pm & 1.48 \\ 1.04 & \pm & 1.17 \end{array} $	$\begin{array}{c} 0.72 & \pm & 0.27 \\ 1.01 & \pm & 0.73 \\ 4.82 & \pm & 0.93 \\ 1.18 & \pm & 0.42 \\ 1.52 & \pm & 0.67 \\ 0.68 & \pm & 0.42 \\ 1.34 & \pm & 0.58 \\ 0.82 & \pm & 0.32 \\ \end{array}$
20, 20, 20, 20, 20, 20,	0 10 20 30 40 50	$ \begin{array}{c} 1.05 \pm 0.26 \\ 1.74 \pm 0.44 \\ 2.68 \pm 0.47 \\ 0.96 \pm 0.32 \\ 1.21 \pm 0.30 \\ 1.00 \pm 0.34 \end{array} $	$\begin{array}{c} 9.18 \pm 2.42 \\ 4.15 \pm 9 \\ 763 \pm 10 \\ 11.8 \pm 2.8 \\ 7.62 \pm 3.35 \\ < 1.17 \end{array}$	$\begin{array}{c} 0.55 & \pm & 0.33 \\ 1.48 & \pm & 0.61 \\ 1.64 & \pm & 0.60 \\ 0.88 & \pm & 0.55 \\ 0.97 & \pm & 0.41 \\ 1.27 & \pm & 0.51 \end{array}$
25,	90c	0.43 <u>+</u> 0.17	<0.55	0.31 <u>+</u> 0.33
30, 30, 30, 30,	0 10 20 30	$\begin{array}{c} 0.81 \pm 0.21 \\ 1.23 \pm 0.28 \\ 1.4 \pm 0.45 \\ 1.28 \pm 0.30 \end{array}$	$ \begin{array}{cccccccccccccccccccccccccccccccccccc$	$\begin{array}{c} 0.53 + 0.27 \\ 1.36 + 0.49 \\ 0.79 + 0.56 \\ 0.98 + 0.46 \end{array}$

TABLE 31 (Continued)

RADIONUCLIDE CONCENTRATIONS IN SURFACE SOIL SAMPLES COLLECTED FROM GRID POINT INTERVALS LOCATION #1 WELDON SPRING, MISSOURI

Grid Location N W	Radionu Ra-226	clide Concentration U-238	s (pCi/g) Th-232
30, 40 30, 50	$\begin{array}{c} 0.97 \pm 0.23 \\ 1.12 \pm 0.28 \end{array}$	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	
40, 0 40, 10 40, 20 40, 30 40, 40 40, 50	$\begin{array}{c} 0.84 & + & 0.21 \\ 1.00 & + & 0.24 \\ 1.05 & + & 0.34 \\ 1.02 & + & 0.29 \\ 0.94 & + & 0.45 \\ 1.23 & + & 0.28 \end{array}$	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	0.62 + 0.42 $1.42 + 0.43$ $1.12 + 0.46$ $0.91 + 0.55$ $0.90 + 0.90$ $1.45 + 0.39$
40, 130°	0.59 <u>+</u> 0.23	0.88 <u>+</u> 1.49	0.48 <u>+</u> 0.35
50, 0 50, 10 50, 20 50, 30 50, 40 50, 50 50, 130 50, 150 50, 160 50, 170 70, 10 70, 10 70, 20	$\begin{array}{c} 0.83 & \pm & 0.26 \\ 0.86 & \pm & 0.18 \\ 1.26 & \pm & 0.25 \\ 0.87 & \pm & 0.27 \\ 0.90 & \pm & 0.28 \\ 1.33 & \pm & 0.32 \\ 1.02 & \pm & 0.24 \\ 1.09 & \pm & 0.29 \\ 0.76 & \pm & 0.22 \\ 0.62 & \pm & 0.28 \\ 0.85 & \pm & 0.22 \\ \hline 0.90 & \pm & 0.22 \\ 1.06 & \pm & 0.24 \\ 1.28 & \pm & 0.29 \\ 0.92 & \pm & 0.27 \\ \end{array}$	3.71 + 1.89 4.13 + 1.12 4.32 + 0.80 9.54 + 2.21 44.9 + 3.7 2.34 + 2.51 2.26 + 0.65 3.41 + 1.12 7.48 + 2.26 1.44 + 1.29 0.81 + 0.66 $ 3.90 + 1.22 0.97 + 0.72 3.41 + 1.98 2.18 + 1.98 2.18 + 1.98 2.18 + 1.98$	$ \begin{array}{cccccccccccccccccccccccccccccccccccc$
70, 30 70, 40 70, 50	$\begin{array}{c} 0.92 \pm 0.27 \\ 0.89 \pm 0.33 \\ 1.15 \pm 0.27 \end{array}$	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	$\begin{array}{c} 1.28 \pm 0.39 \\ 0.90 \pm 0.40 \\ 1.39 \pm 0.52 \end{array}$
71, 195 ^c	0.65 <u>+</u> 0.22	1.78 <u>+</u> 1.29	<0.15
90, 6 ^c 90, 10 90, 20 90, 30 90, 40 90, 50 90, 90	$ \begin{array}{c} 1.29 + 0.27 \\ 0.83 + 0.34 \\ 1.07 + 0.28 \\ 0.88 + 0.23 \\ 0.84 + 0.28 \\ 1.13 + 0.22 \\ 0.69 + 0.29 \end{array} $	3.49 ± 0.90 2.26 ± 2.96 4.90 ± 1.89 1.45 ± 1.79 3.41 ± 2.77 3.73 ± 0.99 4.15 ± 1.54	$ \begin{array}{cccccccccccccccccccccccccccccccccccc$

TABLE 31 (Continued)

RADIONUCLIDE CONCENTRATIONS IN SURFACE SOIL SAMPLES COLLECTED FROM GRID POINT INTERVALS LOCATION #1 WELDON SPRING, MISSOURI

		·····		······································
Grid Location N W		Radionuo Ra-226	clide Concentration U-238	rs (pCi/g) Th-232
90,	130	1 00 ± 0 22	0.87 + 0.67	1 (0 1 0 (0
	170	$\frac{1.09 + 0.33}{0.86 + 0.37}$	0.87 + 0.67	1.68 ± 0.60
90,		0.86 ± 0.27	1.56 ± 0.69	1.50 ± 0.56
90,	187 ^c	0.90 ± 0.26	2.31 ± 0.96	1.42 ± 0.49
94,	10°	1.17 ± 0.29	<1.00	1.35 ± 0.49
102,	70°	0.95 <u>+</u> 0.32	2.97 <u>+</u> 1.51	1.33 <u>+</u> 0.46
105,	20	1.07 <u>+</u> 0.22	5.14 <u>+</u> 0.94	0.92 <u>+</u> 0.65
110,	30	1.24 + 0.36	2.80 + 1.34	1.11 + 0.47
	40	1.13 + 0.29	<1.19	1.25 + 0.50
110,	50	0.43 + 0.34	6.05 + 0.28	1.62 + 0.62
,				
120,	30	1.21 ± 0.52	4.46 <u>+</u> 2.92	1.04 <u>+</u> 1.18
130,	40	1.10 + 0.27	2.70 + 0.79	1.43 + 0.55
•	50	0.89 + 0.21	1.01 + 2.83	1.43 ± 0.53 $1.26 + 0.42$
	90	1.20 + 0.41	2.28 + 2.43	$\frac{1.20 + 0.42}{1.24 + 0.47}$
	130	1.04 + 0.40	4.25 + 2.93	1.50 + 0.65
130,	140	1.05 + 0.38	1.55 + 2.92	1.50 ± 0.05 1.58 ± 1.00
150,	1,0	<u> </u>	1.33 . 2.72	1.50 - 1.00
139,	130°	1.09 ± 0.27	2.09 ± 0.73	1.03 ± 0.51
142,	50°	0.88 <u>+</u> 0.21	1.80 <u>+</u> 0.63	1.06 <u>+</u> 0.55
170,	90	1.02 + 0.24	2.07 + 1.47	1.31 + 0.40
170,	109c	1.02 + 0.24 $1.21 + 0.22$	2.07 + 1.47 2.20 + 1.97	$\frac{1.51 \pm 0.40}{1.57 \pm 0.43}$
	103	1.21 1 0.22	2.20 - 1.37	1.0/ 7 0.40
176,	90c	0.74 + 0.21	<0.54	0.35 + 0.22
176,	108°		2.48 + 1.20	1.40 + 0.50
,		· —		

a Refer to Figure 12.
b Errors are 2σ based on counting statistics.

c Samples collected at fenceline.

TABLE 32 RADIONUCLIDE CONCENTRATIONS IN RANDOM SOIL SAMPLES LOCATION #1 WELDON SPRING, MISSOURI

Grid ^a Location N W	Depth (cm)	Radionuo Ra-226	Clide Concentrat U-238	ions (pCi/g) Th-232
40, 10 40, 10 40, 10	0-15 60 90	$ \begin{array}{r} 1.00 + 0.24^{b} \\ 1.22 + 0.24 \\ 1.08 + 0.22 \end{array} $	$\begin{array}{c} 4.35 \pm 2.00 \\ 1.17 \pm 2.68 \\ 1.49 \pm 1.04 \end{array}$	$ \begin{array}{c} 1.42 \pm 0.43 \\ 0.84 \pm 0.53 \\ 1.11 \pm 0.63 \end{array} $
50, 50 50, 50 50, 50	0-15 60 80	$ \begin{array}{cccccccccccccccccccccccccccccccccccc$	$\begin{array}{c} 2.34 & + & 2.51 \\ 1.96 & + & 0.89 \\ 2.84 & + & 2.41 \end{array}$	$\begin{array}{c} 1.21 \pm 0.72 \\ 1.11 \pm 1.89 \\ 1.81 \pm 0.60 \end{array}$
50, 90 50, 90 50, 90	0-15 60 90	$\begin{array}{c} 1.02 \pm 0.24 \\ 1.22 \pm 0.25 \\ 1.33 \pm 0.24 \end{array}$	$\begin{array}{c} 2.26 \pm 0.65 \\ 1.86 \pm 1.61 \\ 1.45 \pm 0.94 \end{array}$	$\begin{array}{c} 1.01 + 0.34 \\ 1.23 + 0.57 \\ 1.40 + 0.50 \end{array}$
70, 30 70, 30 70, 30	0-15 60 90	$\begin{array}{c} 0.92 \pm 0.27 \\ 0.99 \pm 0.24 \\ 1.18 \pm 0.34 \end{array}$	2.17 <u>+</u> 1.05 <0.88 1.91 <u>+</u> 2.35	$ \begin{array}{c} 1.28 \pm 0.39 \\ 1.11 \pm 0.43 \\ 0.79 \pm 0.57 \end{array} $
130, 50 130, 50	0-15 60	$\begin{array}{c} 0.89 \pm 0.21 \\ 0.95 \pm 0.24 \end{array}$	$\begin{array}{c} 1.01 \pm 2.83 \\ 2.54 \pm 1.79 \end{array}$	$\begin{array}{c} 1.26 \pm 0.42 \\ 1.31 \pm 0.41 \end{array}$

a Refer to Figure 12. b Errors are 2σ based on counting statistics.

RADIONUCLIDE CONCENTRATIONS IN SOIL SAMPLES
FROM LOCATIONS IDENTIFIED BY THE WALKOVER SCAN - LOCATION #1
WELDON SPRING, MISSOURI

Grid ^a D		Depth	Radionuclide Concentrations (pC1/g)			
Loca	ation	(cm)	Ra-226	U-238	Th-232	
N	W			0 230	111-252	
າ	8	0 (12.7 . 2.25	0.010		
2 2	8	0 6 30 35	$13.7 + 0.9^{b}$	$ \begin{array}{cccccccccccccccccccccccccccccccccccc$	4.29 <u>+</u> 1.02	
2	o	30 - 35	10.7 ± 1.1	818 + 24	2.78 ± 1.28	
2	14	0 - 6	12.5 + 12.7	29,530 + 220	Z10. 2	
2	14	15 - 25	40.1 ± 1.3	$\frac{1}{2},430 + \frac{1}{4}10$	<10.2	
2	14	30 - 35	11.1 + 0.9	$\frac{1}{986} + \frac{1}{14}$	6.85 + 1.54	
2	14	60 - 75	8.04 ± 0.70	$\frac{760}{1,360} \frac{+}{+} 10$	3.10 ± 0.88	
2	14	90 - 110	2.38 + 0.46	7,300 + 10	2.36 + 1.10	
-	1.4	50 110	2.30 + 0.40	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	1.93 ± 0.51	
4	9	0 - 6	3.03 + 0.71	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	2.54 + 0.91	
4	9	20 - 30	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	$\frac{1,430}{+10}$	1.52 + 0.96	
4	9	60 - 75	1.15 + 0.22	77.1 + 3.3	$\frac{1.32}{1.47} + 0.38$	
4	9	90 - 105	0.70 + 0.20	36.0 + 1.8	1.09 + 0.40	
			_		1.07 - 0.40	
6	5	0 - 6	4.91 <u>+</u> 1.83	15,200 + 100	<1.34	
8	14	0 - 15	<4.83	<133	<5.27	
8	18	0 - 6	10.8 + 1.7	666 <u>+</u> 11	450 <u>+</u> 6	
9	17	0 - 5	17.0 . 0.0	105.0		
9	17	40 - 50	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	<25.0		
9	17	95 - 100		2.76 + 1.16	1.35 ± 0.39	
,	17	33 - 100	7.50 + 0.90	<17 <u>.</u> 6	2.65 ± 1.14	
11	4	0 - 10	2.62 + 0.63	1,460 + 20	1.35 + 0.76	
11	4	10 - 15	<6 . 98	$ \begin{array}{cccccccccccccccccccccccccccccccccccc$	<7 <u>.</u> 47	
11	4	25 - 30	1.36 + 0.33	$131 \overline{+} 7$	1.36 + 0.44	
11	4	30 - 60	1.08 + 0.32	82.3 ± 4.6	1.36 + 0.53	
			_	4	1.30 1 0.55	
11	11	c. 0 - 10	3.39 ± 0.77	1.780 + 20	1.73 + 0.96	
Ll		10 - 15	1.87 ± 0.42	608 \pm 13	1.34 + 0.81	
11	11	30 – 35	1.26 + 0.27	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	1.48 + 0.43	
1.2	1.6	0 7	7.00 . 0.07		<u></u>	
12	14	0 - 7	7.28 + 0.87	<17.1	2.58 ± 1.11	
12	14	45 - 50	0.77 ± 0.30	179 <u>+</u> 5	0.88 ± 0.42	
12	14	90 – 95	1.31 ± 0.30	7.16 ± 1.36	1.12 ± 0.40	
13	7	0 - 6	8.35 + 1.05	948 + 24	0.83 + 0.97	
13	7	30 - 50	1.32 + 0.34	$\frac{136}{+}$ 5	$\frac{0.83}{1.51} + 0.43$	
13	7	90 - 110	0.88 + 0.22	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	$\frac{1.51 \pm 0.43}{1.56 \pm 0.49}$	
			-	1.JJ	1. JU T U.47	

RADIONUCLIDE CONCENTRATIONS IN SOIL SAMPLES
FROM LOCATIONS IDENTIFIED BY THE WALKOVER SCAN - LOCATION #1
WELDON SPRING, MISSOURI

TABLE 33 (Continued)

Gr	id	Depth	Radio	nuclide Concent	rations	(pCi/g)
	tion W	(cm)	Ra-226	U-23		Th-232
13 13 13	19 19 19	0 - 6 60 - 75 90 - 110	$ \begin{array}{c} 11.3 & \pm 1.6 \\ 1.63 & \pm 0.33 \\ 2.49 & \pm 0.34 \end{array} $	19,810 + . 323 + 128 +	50 6 7	$\begin{array}{c} 1.74 \pm 1.68 \\ 1.14 \pm 0.44 \\ 1.33 \pm 0.63 \end{array}$
18	2	0 - 15	0.87 <u>+</u> 0.42	2,110 <u>+</u>	30	<0.46
18 18 18	17 17 17	0 - 10 60 - 80 90 - 110	3.27 + 1.28 1.04 + 0.34 1.27 + 0.23		40 6.5 31.8	$\begin{array}{c} 2.44 & + & 1.40 \\ 1.23 & + & 0.36 \\ 1.44 & + & 0.47 \end{array}$
19	52	0 - 15	1.25 ± 0.75	670 <u>+</u>	26	1.42 ± 0.65
20 20	25 25	0 - 15 65 - 70	$\begin{array}{c} 1.22 \pm 0.58 \\ 1.08 \pm 0.31 \end{array}$	$735 + \\ 32.1 + \\ \hline$	24 3.1	$\begin{array}{c} 0.98 \pm 1.21 \\ 1.49 \pm 0.53 \end{array}$
21 21	14 14	0 - 15 60 - 75	2.93 <u>+</u> 0.53	1,099 <u>+</u> 48.4 <u>+</u>	11 4.5	$\begin{array}{c} 1.16 \pm 0.77 \\ 1.79 \pm 0.52 \end{array}$
23 23 23	10 10 10	0 - 15 30 - 45 60 - 75	<1.13 0.88 ± 0.17 1.37 ± 0.29	$ \begin{array}{r} 14,260 & + \\ 186 & + \\ \hline 13.4 & + \\ \end{array} $	50 4 3.3	<1.26 1.37 ± 0.35 1.17 ± 0.59
26 26	26 26	0 - 15 50 - 80	$\begin{array}{c} 13.7 & + 2.1 \\ 1.08 & + 0.27 \end{array}$	15,020 $\frac{+}{+}$ 39.1 $\frac{+}{+}$	50 3.6	<1.41 0.94 <u>+</u> 0.44
27 27	41 41	0 - 15 40 - 55	$\begin{array}{c} 2.66 \pm 1.01 \\ 1.30 \pm 0.26 \end{array}$	$\frac{792}{130} \frac{+}{+}$	28 4	$\begin{array}{c} 1.34 \pm 1.32 \\ 1.23 \pm 0.44 \end{array}$
N 0,		0 - 10 0 - 15	$1.24 + 0.56 \\ 1.02 + 0.28$	551 <u>+</u> 265 <u>+</u>	10 5	$\begin{array}{c} 1.01 \pm 0.51 \\ 0.79 \pm 0.55 \end{array}$
	, 50 , 50	0 - 15 30 - 40	1.04 <u>+</u> 0.43 1.28 <u>+</u> 0.29	139 <u>+</u> 7.04 <u>+</u>	8	$\begin{array}{c} 1.11 \pm 0.49 \\ 1.10 \pm 0.39 \end{array}$

a Refer to Figure 13.

b Errors are 20 based on counting statistics.

RADIONUCLIDE CONCENTRATIONS IN H₂O SAMPLES COLLECTED FROM DRAINAGE DITCH - LOCATION #1 WELDON SPRING, MISSOURI

Grid Locationa	Radionuclide	Concentrations (pCi/l
	Gross Alph	na Gross Beta
18N, 50W	3073 <u>+</u> 21 ¹	9 4221 <u>+</u> 18°
38n, 90W	0.90 <u>+</u> 0.	.56 6.43 ± 1.0

a Refer to Figure 12.

U-238: 1699 + 29 U-235: 66 + 6 U-234: 1643 + 28 Ra-228: .20 + .64 Ra-226: <.08

b Errors are 20 based on counting statistics.

c Isotopic Uranium levels were:

TABLE 35

RADIONUCLIDE CONCENTRATIONS IN SEDIMENT SAMPLES COLLECTED FROM DRAINAGE DITCHES - LOCATION #1

WELDON SPRING, MISSOURI

Grid ^a		clide Concentratio	ns (pCi/g)
Location N W	Ra-226	U-238	Th-232
18, 43	0.95 <u>+</u> 0.46 ^b	781 <u>+</u> 13	1.43 <u>+</u> 0.57
65, 207	0.60 <u>+</u> 0.15	1.09 <u>+</u> 0.41	0.48 <u>+</u> 0.52
70, 170	0.96 <u>+</u> 0.23	1.74 <u>+</u> 1.70	11.40 <u>+</u> 0.62
169, 95	0.97 ± 0.37	2.13 <u>+</u> 2.22	1.25 <u>+</u> 0.39
186, 90	1.27 <u>+</u> 0.27	1.43 <u>+</u> 1.96	1.32 ± 0.47

a Refer to Figure 12.

b Errors are 2σ based on counting statistics.

TABLE 36

DIRECT RADIATION LEVELS MEASURED AT GRID LINE INTERSECTIONS
LOCATION #2
WELDON SPRING, MISSOURI

		Gamma Exposure	Gamma Exposure	Beta-Gamma	
Loca	tion	Rates at 1 m Above	Rates at the	Dose Rates at 1 cm	
S	E	the Surface (µR/h)	Surface (µR/h)	Above the Surface (µrad/h)	
0	0	7	7	16	
0	10	7	7	32	
0	20	7	7	39	
0	30	8	8	11	
0	40	7	7	7	
0	50	8	7	6 6	
0	60	14	13	120	
0	70	12	11	120	
0	80	8	8	21	
10	0	8	8	38	
10	10	9	9	43	
10	20	8	9	70	
10	30	8	7	45	
10	40	8	8	40	
10	50	9	. 8	31	
10	60	8	7	31	
10	70	6	6	15	
10	80	7	8	8	
20	0	7	8	8	
20	10	7	8	31	
20	20	7	8	18	
20	30	7	8	23	
20	40	8	8	8	
20	50	7 .	7	37	
20	60	7	7	7	

aRefer to Figure 14.

TABLE 37 DIRECT RADIATION LEVELS AT LOCATIONS OF ELEVATED SURFACE READINGS LOCATION #2 WELDON SPRING, MISSOURI

Grid Location a		Exposure Rate (μR/h)		Surface Dose Rate	Contact Exposure
		Contact	1 m Above Surface	(µrad/h)	Rate After Sample Removal (µR/H)
4 S	6 E	93	12	93	15
4 S	5 9E	53	21	500	51
5 S	68E	150	20	4950	70
6 S	11E	26	14	400	35
7 S	20 E	17	8	130	16

Refer to Figure 15.
Radionuclide concentrations in samples are presented in Table 39.

TABLE 38 RADIONUCLIDE CONCENTRATIONS IN SURFACE SOIL SAMPLES COLLECTED FROM 10 M GRID INTERVALS - LOCATION #2 WELDON SPRING, MISSOURI

Loca	ation ^a	Radionucl	ide Concentrati	ons (pCi/g)
S	E	Ra-226	U-238	Th-232
		·.		
0	0	1.27 + 0.28 ^b	4.81 + 1.67	1.05 + 0.42
Ŏ	10	0.80 + 0.22	3.56 + 2.21	1.08 + 0.41
Ō	20	0.91 ± 0.25	7.15 + 1.28	0.49 ± 0.23
0	30	1.03 + 0.25	11.7 + 2.8	1.16 + 0.55
0	40	1.00 ± 0.36	11.4 + 3.2	1.13 ± 0.43
0	50	0.64 + 0.22	3.29 + 1.99	<0.18
0	60	1.17 ± 0.25	<2.98	1.02 + 0.48
O	7υ	1.23 + 0.41	97.4 + 7.2	1.33 + 0.62
0	80	1.05 ± 0.32	8.31 ± 2.12	1.06 + 0.41
		_	· —	_
10	0	1.44 + 0.29	<1.39	1.27 ± 0.40
10	10	1.52 ± 0.37	32.6 ± 3.0	1.91 ± 0.51
10	20	1.84 + 0.37	57.0 + 5.1	0.87 + 0.43
10	30	0.96 ± 0.24	<2.08	0.87 ± 0.34
10	40	1.31 ± 0.34	33.1 + 4.6	1.22 + 0.54
10	50	1.77 ± 0.32	30.9 ± 4.2	1.80 ± 0.49
10	6 0	1.72 ± 0.33	<1.76	1.28 ± 0.44
10	80	0.97 + 0.26	1.13 + 2.31	1.13 ± 0.41
10	7 0	0.92 ± 0.20	1.19 ± 0.20	0.40 ± 0.45
20	0	1.13 ± 0.26	3.92 <u>+</u> 1.19	1.34 ± 0.52
20	10	0.97 ± 0.32	3.59 ± 3.35	0.99 ± 0.45
20	20	1.14 ± 0.28	4.85 <u>+</u> 1.28	1.18 ± 0.37
20	30	1.28 ± 0.27	2.81 ± 0.82	1.02 ± 0.34
20	40	0.87 ± 0.27	<1.04	1.23 ± 0.41
20	50	1.14 ± 0.24	<0.97	0.86 ± 0.32
20	60	1.08 ± 0.21	1.09 ± 0.62	1.33 ± 0.52

aRefer to Figure 14. ** bErrors are 2g based on counting statistics.

TABLE 39 RADIONUCLIDE CONCENTRATIONS IN SOIL SAMPLES FROM LOCATIONS IDENTIFIED BY THE WALKOVER SCAN LOCATION #2 WELDON SPRING, MISSOURI

Location ^a	Depth (cm)	Radionucl: Ra-226	ide Concentratio U-238	ons (pCi/g) Th-232
4S, 6E	0 - 15	38.8 <u>+</u> 1.3 ^b	34.6 <u>+</u> 4.7	1.28 + 0.93
4s, 59E	0 - 10 10 - 15 15 - 30	$\begin{array}{c} 2.86 \pm 0.47 \\ 2.36 \pm 0.36 \\ 2.23 \pm 0.46 \end{array}$	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	$ \begin{array}{c} 1.17 + 0.70 \\ 1.25 + 0.46 \\ 1.79 + 1.37 \end{array} $
5s, 68E	0 - 10 15 - 30	$\begin{array}{c} 3.72 \pm 1.76 \\ 5.06 \pm 0.62 \end{array}$	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	<1.23 2.28 <u>+</u> 0.83
6S, 11E	0 - 10 15 - 30	$\begin{array}{c} 2.78 \pm 0.44 \\ 1.94 \pm 0.42 \end{array}$	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	$\begin{array}{c} 1.36 + 0.48 \\ 1.19 + 0.49 \end{array}$
7S, 20E 7S, 20E	0 - 10 10 - 15	$\begin{array}{c} 1.62 \pm 0.38 \\ 0.99 \pm 0.29 \end{array}$	160 <u>+</u> 6 <1.94	$\begin{array}{c} 1.15 \pm 0.46 \\ 0.88 \pm 0.39 \end{array}$
12S, 23W	0 - 8 8 - 16	$\begin{array}{c} 6.16 \pm 0.55 \\ 2.78 \pm 0.36 \end{array}$	$ \begin{array}{c} 390 & \pm & 10 \\ 74.6 & \pm & 5.0 \end{array} $	$\begin{array}{c} 0.47 \pm 0.51 \\ 1.17 \pm 0.38 \end{array}$
15S, 23W	0 - 8 8 - 16 16 - 32 32 - 46	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	$\begin{array}{c} <2.16 \\ 60.2 \pm 5.0 \\ 10.1 \pm 2.2 \\ 32.4 \pm 3.5 \end{array}$	1.60 ± 0.73 <0.53 <0.31 0.76 ± 0.57

aRefer to Figure 15. bErrors are 20 based on counting statistics.

RADIONUCLIDE CONCENTRATIONS IN SOIL SAMPLES
FROM LOCATIONS [DENTIFIED BY THE WALKOVER SCAN - LOCATION #3
WELDON SPRING, MISSOURI

Location	<u>Depth</u>	Radionuc.	lide Concentra	tions (pCi/g)
	(cm)	Ra-226	U-238	Th-232
455, 2R	0 - 7	3.14 + 0.68 ^b	1683 + 20	<0.40
455, 2R	7 - 15	1.34 + 0.58	$\frac{-}{2640 + 10}$	<0.42
455, 2R	15 - 30	0.82 ± 0.40	880 + 70	<0.23
455, 2R	30 - 45	0.92 ± 0.29	436 + 15	<0.24
455, 2R	45 - 60	1.28 ± 0.38	477 + 11	0.58 ± 0.3
Top of loading o		4.46 <u>+</u> 0.78	1042 + 20	<0.46

aRefer to Figures 16.

bErrors are 20 based on counting statistics.

TABLE 41

DIRECT RADIATION LEVELS MEASURED AT SOIL SAMPLE LOCATIONS
S.E. DRAINAGE EASEMENT
WELDON SPRING, MISSOURI

<u>Locati</u> (m)	on a	Gamma Exposure Rates at 1 m Above the Surface (µR/h)	Gamma Exposure Rates at the Surface (µR/h)
0	1L	12	8
0	2L	8	7
10	С	29	120
100	1 R	10	14
130	1L	10	44
190	4R	22	43
195	1L	14	25
200	2L	14	15
300	2R	14	16
305	1R	15	29

Refer to Figure 17.

RADIONUCLIDE CONCENTRATIONS IN SURFACE SOIL AND SEDIMENT SAMPLES COLLECTED FROM 100 M INTERVALS ALONG THE SOUTHEAST DRAINAGE EASEMENT WELDON SPRING, MISSOURI

Location ^a	Radionuclide Concentrations (pCi/g)				
	Ra-226	U-238	Th-232		
Soil					
0, L	$0.76 + 0.18^{b}$	<0.76	0.43 + 0.1		
100, R	5.51 + 0.45	1.26 + 3.03	1.66 + 0.5		
200, 2L		41.3 ± 4.1	0.87 ± 0.3		
300, 2R	8.36 ± 0.54	42.0 ± 2.0	2.69 ± 0.5		
Sediment					
0, C	6.57 ± 0.59	39.7 $+$ 7.0	2.14 <u>+</u> 0.7		
100, C	5.42 ± 0.46	<1 <u>.</u> 56	2.18 ± 0.4		
200, C	4.34 ± 0.53	11.7 ± 3.5	1.41 ± 0.8		

 $^{^{\}rm a}$ Refer to Figure 17. $^{\rm b}$ Errors are 2 σ based on counting statistics.

RADIONUCLIDE CONCENTRATIONS IN SOIL SAMPLES FROM LOCATIONS IDENTIFIED IN THE WALKOVER SCAN - S.E. DRAINAGE EASEMENT WELDON SPRING, MISSOURI

Location ^a	Depth	Radionuc	lide Concentration	ıs (pCi/g)
	(cm)	Ra-226	U-238	Th-232
Soil				
OM, 2L	0-15	$0.85 + 0.24^{b}$	4.20 + 2.26	0.82 + 0.44
OM, 2L	15-30	1.02 + 0.23	2.19 + 1.37	1.05 + 0.35
OM, 2L	30-45	0.70 + 0.19	1.52 + 0.52	
OM, 2L	45-60	0.81 ± 0.23	2.10 ± 0.14	0.99 + 0.10
OM, 2L	60-75	0.71 ± 0.20	4.24 ± 2.30	0.97 ± 0.33
10M, C	0-15	170 + 2	300 + 6	13.6 + 1.91
10M, C	15-30	$\begin{array}{ccc} 170 & + 2 \\ 210 & + 3 \end{array}$	$ \begin{array}{cccccccccccccccccccccccccccccccccccc$	9.21 ± 2.31
130M, L	0-15	37.5 + 1.6	231 + 9	69.1 + 30.4
130M, L	15-30	68.6 + 1.6	81.8 + 5.8	10.9 + 1.6
130M, L	. 30–60	32.5 ± 1.4	$ \begin{array}{cccccccccccccccccccccccccccccccccccc$	4.95 <u>+</u> 1.16
190M, 4R	0-15	59.0 + 1.6	601 + 11	1.33 + 0.98
190M, 4R	15-30	31.0 $\frac{-}{+}$ 1.1	$\begin{array}{cccc} 601 & + & 11 \\ 216 & + & 5 \end{array}$	1.52 ± 0.90
100v 6p	30-45	4 07 1 0 46	101	0.00 + 0.46
190M, 4R	45-60	4.07 <u>+</u> 0.46	181 <u>+</u> 6	0.88 <u>+</u> 0.46
195M, L	0-15	20.9 + 0.9	<3.92	1.28 + 0.88
195M, L	15-30	12.7 + 0.8	150 + 6	0.98 + 0.75
195M, L	30-45	3.04 + 0.44	$ \begin{array}{cccccccccccccccccccccccccccccccccccc$	1.03 + 0.65
195M, L	45-60	2.04 ± 0.34	<3.77	1.08 ± 0.40
305M, R	0-15	18.1 + 0.9	5.31 + 3.52	1.82 + 1.02
305m, R	15-30	27.2 + 1.2	66.5 + 5.2	8.15 + 1.43
305M, R	45-60	6.87 ± 0.47	25.1 \pm 2.2	1.16 ± 0.49

a Refer to Figure 17.

b Errors are 20 based on counting statistics.

TABLE 44

RADIONUCLIDE CONCENTRATIONS IN WATER SAMPLES COLLECTED FROM THE SOUTHEAST DRAINAGE EASEMENT WELDON SPRING, MISSOURI

Locationa	Radionuclide Concentrations (pCi/l)		
	Gross Alpha	Gross Beta	
Origin	147 <u>+</u> 5 ^b	44 + 2°	
Fenceline	159 <u>+</u> 5	39 <u>+</u> 2 ^đ	

a Refer to Figure 17.

^c Isotopic Uranium, Radium, and Thorium levels were:

U-238: 129 + 4 4.95 + 0.78 25 + 4U-235: U-234: 125 Ra-228: 2.06 + 0.53Ra-226: 0.50 ± 0.09 Th-232: 0.05 + 0.06Th-230: 0.13 + 0.09 0.08 ± 0.07 Th-228:

 $^{\rm d}$ Isotopic Uranium, Radium, and Thorium levels were:

U-238: 134 + 4 4.76 ± 0.79 U-235: U-234: - 4 128 1.94 ± 0.47 Ra-228: Ra-226: 0.59 + 0.10Th-232: 0.04 ± 0.05 0.24 + 0.11Th-230: 0.05 ∓ 0.05 Th-228:

b Errors are 20 based on counting statistics.

TABLE 45

DIRECT RADIATION LEVELS MEASURED AT SOIL SAMPLE LOCATIONS
LOCATION #5 - DITCH FROM RAFFINATE PIT
WELDON SPRING, MISSOURI

Locati (m)		Gamma Exposure Rates at 1 m Above the Surface $(\mu R/h)$	Gamma Exposure Rates at the Surface (µR/h)
0	С	6	6
100	2L	7	8
200	С	13	18
237	С	9	13

aRefer to Figure 18.

TABLE 46 DIRECT RADIATION LEVELS AT LOCATIONS OF ELEVATED SURFACE READINGS LOCATION #5 - DITCH FROM RAFFINATE PIT WELDON SPRING, MISSOURI

Grid Location a	Expos	ure Rate (µR/h)	Contact Exposure Rate
		1 m Above Surface	After Sample Removal (µR/h)
22	42	12	- c
115 2L	17	8	20
202	45	17	160

Refer to Figure 18.

Badionuclide concentrations of samples are presented in Table 48.

CDash indicates measurement not performed.

RADIONUCLIDE CONCENTRATIONS IN SEDIMENT SAMPLES COLLECTED AT 100 M INTERVALS ALONG THE DITCH

FROM THE RAFFINATE PITS (LOCATION #5) WELDON SPRING, MISSOURI

Locationa	Radionucl	ide Concentration	ons (pCi/g)
	Ra-226	U-238	Th-232
0, C	0.61 + 0.23 ^b	1.25 + 1.50	0.61 <u>+</u> 0.4
100, 2L	1.18 + 0.25	<1.05	1.24 + 0.3
200, C	8.22 ± 0.60	6.00 ± 1.16	1.21 ± 0.5
237, C	1.81 + 0.37	<1.10	0.87 + 0.3

a Refer to Figure 18.b Errors are 20 based on counting statistics.

TABLE 48

RADIONUCLIDE CONCENTRATIONS IN SEDIMENT

COLLECTED FROM THE DITCH FROM THE RAFFINATE PIT (LOCATION #5)

WELDON SPRING, MISSOURI

Locationa	Depth	Radionucli	lde Concentrat	ions (pCi/g)
	(cm)	Ra-226	U-238	Th-232
22M, C	0- 6	27.3 <u>+</u> 0.9 ^b	<1.10	0.62 <u>+</u> 0.9
115M, 2L	0-15	0.94 ± 0.26	1.66 ± 0.76	0.16 <u>+</u> 0.4
115M, 2L	15-30	2.96 ± 0.52	<1.34	1.32 ± 0.7
115M, 2L	45-60	7.47 + 0.53	1.60 + 1.33	1.20 + 0.5
115M, 2L	60-75	2.31 ± 0.32	2.06 ± 2.09	1.32 ± 0.4
202M, C	0-15	52.6 <u>+</u> 1.4	<1.78	1.32 + 0.6
202M, C	15-30	62.6 + 1.6	<3.42	<0.70
202M, C	45-60	26.7 + 1.0	<1.30	1.42 + 0.7

a Refer to Figure 18.

b Errors are 20 based on counting statistics.

TABLE 49 DIRECT RADIATION LEVELS AT LOCATIONS OF ELEVATED SURFACE READINGS LOCATION #6 WELDON SPRING, MISSOURI

Grid Lo	ocation a	Expos	ure Rate (μR/h)	Contact Exposure Rate
····		Contact	1 m Above Surface	After Sample Removal (µR/h)
95	1L	14	7	17
160	1L	15	8	14

Refer to Figure 19.
Radionuclide concentrations of samples are presented in Table 50.

RADIONUCLIDE CONCENTRATIONS IN SOIL FROM AREAS
IDENTIFIED BY THE WALKOVER SCAN - DITCH #4, LOCATION #6
WELDON SPRING, MISSOURI

Locationa	Depth	Radionucl:	lde Concentrati	ons (pCi/g)
(m)	(cm)	Ra-226	U-238	Th-232
95	0-15	1.28 + 0.29b	113 + 6	1.43 + 0.7
95	15-30	1.09 ± 0.26	<3.38	1.54 + 0.4
95	45-60	0.84 ± 0.29	40 <u>+</u> 5	1.11 ± 0.4
160	0-15	1.60 + 0.38	123 + 80	1.56 + 0.7
160	15-30	1.09 ± 0.28	91 + 7	0.78 ± 0.5

^a Refer to Figure 19.

b Errors are 20 based on counting statistics.

DIRECT RADIATION LEVELS AT LOCATIONS OF ELEVATED SURFACE READINGS

LOCATION #7 WELDON SPRING, MISSOURI

Grid Location a	Exposu	ure Rate (µR/h) 1 m Above Surface	Surface Dose Rate (µrad/h)	Contact Exposure b Rate After Sample Removal (µR/H)
1154 0.5R	120	14	190	200
1154 1.5R	290	17	500	150

Refer to Figure 11.
bRadionuclide concentrations in samples are presented in Table 52.

TABLE 52 RADIONUCLIDE CONCENTRATIONS IN SOIL FROM AREA IDENTIFIED IN WALKOVER SCAN OF ROAD #1 - LOCATION #7 WELDON SPRING, MISSOURI

Locationa	Depth		ide Concentration	
	(cm)	Ra-226	U-238	Th-232
1154, 0.5R	0 - 5	12.0 + 1.0 ^b	<1.19	<0.22
1154, 0.5R	10 - 15	206 + 3	<4.48	1.41 ± 1.5
1154, 0.5R	30 - 35	1.73 ± 0.41	0.53 ± 1.13	1.24 ± 1.19
1154, 0.5R	60 - 65	1.02 ± 0.21	1.21 ± 1.40	1.29 ± 0.44
1154, 1.5R	0 - 5	215 + 3	<4.69	<1.05
1154, 1.5R	30 - 3 5	9.0 + 1.0	<0.73	0.82 + 0.43

aRefer to Figure 11. bErrors are 2g based on counting statistics.

TABLE 53

TH-230 AND U-238 CONCENTRATIONS IN SELECTED SOIL SAMPLES WELDON SPRING, MISSOURI

Location	Depth	Radionuclide Conce	ntrations (pCi/g)
·		U-238	Th-230
Location #1, 4N, 9W	0- 6 ст	3620 <u>+</u> 20	19.2 + 0.5
Location #1, 2N, 8W	0- 6 ст	2010 <u>+</u> 18	53.4 <u>+</u> 0.8
Location #1, ON, 8E	0-10 cm 10-15 cm	551 + 10 $ 265 + 5$	$\begin{array}{c} 2.07 \pm 0.17 \\ 1.39 \pm 0.13 \end{array}$
Location #4, 1290M	0-15 cm	123 <u>+</u> 80	2.52 <u>+</u> 0.18
Location #4, 1450M	. 0—15 ст	113 <u>+</u> 6	3.47 <u>+</u> 0.21
SEDE 195M, IL 195M, IL	15-30 cm 30-45 cm	150 + 6 $190 + 6$	$\begin{array}{c} 11.5 & \pm & 0.38 \\ 4.15 & \pm & 0.21 \end{array}$

TABLE 54

AREAS OF ARMY RESERVE PROPERTIES WHICH EXCEEDS RESIDUAL CONTAMINATION CRITERIA®

LOCATION	LOCATION DESCRIPTION	RAD I ONUCL I DES	ESTIMATED	TED QUANTITIES OF MA'	ESTIMATED QUANTITIES OF MATERIAL EXCEEDING GUIDELINES	REMARKS
			AREA(m ²)	DEPTH(m)	VOLUME (m ³)	
q_	Army Reserve Access Area to Bechtel/DOE Property	U-238	1520	1,25	1900	Depth estimated from regression of existing data
2°C	Army Rallroad #2	U-238	400	٤,	200	
34	Army Railroad #2 Loading Dock	U-238	2	0.1	2	Isolated "hot" spots
9	Southeast Drainage Easement	U-238, Th-232, Ra-226	2500	0.1	2500	Estimated width of contamination is 5m
ţv.	Urainage Oitch From Raffinate Pit Area	Ra-226	750	0.1	750	Estimated width of contamination is 3m
69	Drainage Ditch #4 (near property fenceline)	U-258	009	0.1	, 600	Estimated width of contamination is 3m
ηĻ	Army Reserve Road #1	Ra-226	,	, . , .	<.5 .5	Isolated "hot" spot
^a Refer to	^a Refer to Appendix C. ^b Refer to Floure 20.				4452.5 co. m.tu	, M. Jus

DRafer to Figure 20.
CRefer to Figure 21.
dRafer to Figure 22.
RRefer to Figure 23.
fRefer to Figure 24.
9Refer to Figure 25.

REFERENCES

1. Title 40, Code of Federal Regulations, Part 141, <u>Interim Primary Drinking</u>
Water Standards, Federal Register, July 1976.

APPENDIX A

MAJOR SAMPLING AND ANALYTICAL EQUIPMENT

TABLE 11

DIRECT RADIATION LEVELS MEASURED AT SOIL SAMPLE LOCATIONS
ROAD #6
WELDON SPRING, MISSOURI

Locat	ion a	Gamma Exposure Rates	Gamma Exposure Rate
(n	1)	at 1 m Above the Surface	at the Surface
		(μR/h)	(μR/h)
0	R	5	5
100	L	6	6
200	R	7	6
300	L	6	6
400	R	б	6
500	L	6	5
600	R	. 6	6
700	L	6	6
800	R	6	6
900	L	6	6
1000	R	6	5
1100	L	6	5
1200	R	6	6
1300	L	6	. 6
1400	R	6	5
1500	L	6	6
1600	R	6	5
1700	L	6	6
1800	R	6	5
1900	L	6	5
2000	R	.6	5
2100	L	6	5
2200	R	6	6
2300	L	5	5
2400	R	5	5
2500	L	6	6
2600	R	6	5

Refer to Figure 10.

TABLE 12

DIRECT RADIATION LEVELS MEASURED AT SOIL SAMPLE LOCATIONS ROAD #7

WELDON SPRING, MISSOURI

Locat		Gamma Exposure Rates at 1 m Above the Surface (µR/h)	Gamma Exposure Rates at the Surface (µR/h)
0	1 R	7	7
100	1L	8	8
200	1 R	7	8
300	1L	7	7
400	1 R	8	7
<i>5</i> 00	1L	7	8
600	1 R	8	8
700	1L	7	7

aRefer to Figure 10.

RADIONUCLIDE CONCENTRATIONS IN SURFACE SUIL SAMPLES COLLECTED AT 100 M INTERVALS ALONG ROAD #1 WELDON SPRING, MISSOURI

		- · · · · · · · · · · · · · · · · · · ·	
Locationa	Radionu	clide Concentrati	ons (nCi/a)
	Ra-226	U-238	Th-232
 			111 232
O, R	$1.32 + 2.19^{b}$	6.12 + 1.50	1.31 + 0.79
100, L	0.75 + 0.29	1.40 + 0.72	0.84 + 0.29
200, R	0.87 + 0.22	5.50 ± 2.04	0.99 + 0.48
300, L	1.37 + 0.33	5.50 ± 2.04 6.19 ± 2.39	0.94 + 0.37
400, R	0.84 ± 0.22	0.31 ± 0.56	0.62 + 0.26
500, L	0.85 ± 0.31	1.81 + 1.79	0.45 ± 0.29
600, R	0.51 ± 0.21	<0.73	0.46 ± 0.37
700, L	0.62 + 0.17	1.17 + 0.53	0.43 + 0.21
800, R	0.75 ± 0.22	<0.62	0.38 + 0.54
900, L	0.65 + 0.20	<0.77	0.41 + 0.32
1000, R	0.34 ± 0.17	0.79 + 0.68	<0.10
1100, L	1.20 + 0.29	2.77 + 1.29	1.47 + 0.56
1200, R	1.26 + 0.29	3.00 ± 2.19	1.50 + 0.45
1300, L	0.69 + 0.53	1.04 + 0.71	0.25 + 0.18
1400, R	0.69 ± 0.22	1.94 + 1.54	0.60 ± 0.42
1500, L	0.75 + 0.38	1.40 + 0.94	0.40 + 0.29
1600, R	0.78 ± 0.24	1.56 + 0.82	0.53 + 0.54
1700, L	0.40 + 0.13	2.47 + 1.19	<0.09
1800, R	0.67 ± 0.19	1.10 + 1.02	0.47 + 0.24
1900, L	0.64 + 0.21	0.82 + 0.69	0.37 + 0.17
2000, R	$^{\prime}$ 0.84 \pm 0.34	<0.66	0.45 + 0.27
2100, R	0.85 ± 0.26	2.55 + 1.53	<0.30
2200, L	0.60 ± 0.21	1.06 ± 0.88	0.61 + 0.23
2300, R	0.89 ± 0.24	<0.88	1.23 + 0.52
2400, L	0.46 ± 0.15	<0.52	0.50 ± 0.24
2500, R	0.73 ± 0.16	2.26 + 0.57	0.47 + 0.22
2600, L	0.67 ± 0.18	<0.57	0.33 + 0.30
2700, R	0.91 ± 0.26	1.29 + 1.20	0.80 + 0.31
2800, L	0.50 ± 0.16	1.19 ± 0.79	0.37 ± 0.25
2800, L	0.53 ± 0.16	1.03 ± 0.58	0.19 + 0.15
2900, R	0.79 ± 0.27	<0.59	0.80 ± 0.32
 3000, L	0.82 ± 0.26	2.46 ± 1.68	1.06 + 0.36
3100, R	ე ₅ <0√18	<0.69	0.44 + 0.38
3200, L	0.53 ± 0.16	1.25 <u>+</u> 0.56	0.73 + 0.13
3300, R	0.61 ± 0.17	0.41 ± 0.42	0.60 ± 0.30
3400, L	0.85 ± 0.22	<0.70	0.46 + 0.23
3500, R	0.73 ± 0.17	1.54 ± 1.22	0.48 ± 0.32
3600, L	0.99 ± 0.23	1.50 ± 0.67	0.64 ± 0.40
3700, R	0.93 ± 0.24	<0.86	0.71 ± 0.27
3800, L	0.88 ± 0.37	1.35 + 1.01	0.78 ± 0.47
3900, R	0.84 ± 0.28	1.10 ± 1.69	0.51 ± 0.48
4000, L	0.91 ± 0.21	1.65 ± 0.53	1.25 ± 0.36
			

TABLE 13 (Continued)

RADIONUCLIDE CONCENTRATIONS IN SURFACE SOIL SAMPLES COLLECTED AT 100 M INTERVALS ALONG ROAD #1 WELDON SPRING, MISSOURI

			
• • • • • • • • • • • • • • • • • • • •			
Location		nuclide Concentratio	ns (pCi/g)
	Ra-226	U-238	Th-232
4100, R	0.70 + 0.16	0 52 ± 1 61	0.51 + 0.25
		0.52 + 1.41	0.51 ± 0.25
4200, L	0.70 ± 0.19	1.01 + 1.10	0.52 ± 0.30
4300, R	0.91 ± 0.20	1.25 + 1.28	0.42 ± 0.58
4400, L	0.76 ± 0.22	1.05 <u>+</u> 0.93	1.11 ± 0.40
4500, R	0.76 ± 0.18	1.67 <u>+</u> 1.26	0.45 ± 0.23
4600, L	0.59 ± 0.21	0.91 ± 1.25	<0.15
4700, R	0.73 + 0.21	<0.70	0.58 + 0.40
4800, L	0.75 + 0.16	1.05 + 0.82	0.36 ± 0.22
4900, R	0.56 + 0.24	<0.66	0.32 + 0.45
5000, L	0.64 + 0.22	<0.64	0.45 + 0.25
5100, R	0.46 + 0.16	0.50 + 0.85	0.36 + 0.21
5200, L	0.63 + 0.19	<0.70	<0.14
5300, R	0.72 + 0.18	0.95 + 1.16	
5400, L	0.67 ± 0.18 0.67 ± 0.16	0.65 ± 0.77	<0.11
			0.27 + 0.18
5500, R	0.93 ± 0.28	<0.73	0.66 ± 0.33
5600, L	0.61 ± 0.15	1.72 ± 0.55	0.45 ± 0.22
5700, R	0.63 ± 0.18	0.84 ± 0.60	0.42 + 0.30
5800, L	0.74 ± 0.21	<0.80	<0.16
5900, R	0.88 + 0.23	<0.72	0.60 ± 0.35
6000, L	0.58 + 0.21	1.01 + 1.10	0.41 + 0.28
6100, R	0.80 + 0.27	1.82 + 1.67	0.65 + 0.37
6200, L	0.93 + 0.20	<0.79	<0 <u>.</u> 19
6300, R	0.67 ± 0.18	1.57 + 0.61	0.40 + 0.55
6400, L	0.46 + 0.14	<0.48	<0.09
6500, R	0.47 ± 0.15	<0.54	
-			0.27 + 0.20
6600, L	0.91 ± 0.21	0.88 ± 0.52	0.06 ± 0.07
6700, R	0.54 ± 0.16	<0.37	<0.10
6800, L	0.66 ± 0.17	<0.45	<0.10
6900, R	0.61 ± 0.15	0.75 ± 0.62	0.25 ± 0.15
7000, L	0.67 ± 0.18	0.99 ± 0.71	0.16 ± 0.15
7100, R	0.72 ± 0.17	0.62 ± 0.64	0.23 + 0.15
7200, L	0.48 + 0.15	0.52 + 0.84	0.31 + 0.14
7300, R	0.47 + 0.23	<0.54	0.25 + 0.22
7400, L	0.67 + 0.13	0.65 + 0.49	0.34 + 0.17
	0.69 ± 0.18	0.69 + 1.66	0.16 + 0.12
7600, L	0.49 ± 0.12	1.17 + 0.34	<0.09
7700, R	0.43 + 0.17	0.93 + 1.08	
7800, L	0.64 + 0.20	0.93 ± 1.08 0.61 + 1.81	<0.12 0.50 ± 0.22
•	0.83 ± 0.26		0.59 ± 0.22
7900, R		<0.64	0.21 + 0.24
8000, L	0.99 ± 0.31	<0.72	<0.21
8100, R	0.87 + 0.23	<0.72	<0.18
8200, L	0.64 ± 0.19	0.47 + 0.70	0.35 ± 0.25

TABLE 13 (Continued)

RADIONUCLIDE CONCENTRATIONS IN SURFACE SOIL SAMPLES COLLECTED AT 100 M INTERVALS ALONG ROAD #1 WELDON SPRING, MISSOURI

Location	Radion	uclide Concentration	ons (nCi/g)
	Ra-226	U-238	Th-232
8300, R	0.65 ± 0.17	2.14 <u>+</u> 0.96	<0.13
8400, L	0.60 ± 0.13	1.02 ± 0.55	0.30 ± 0.19
8500, R	0.77 ± 0.20	<0.68	0.42 ± 0.23
8600, L	0.64 ± 0.15	0.64 <u>+</u> 0.77	0.18 ± 0.11
8700, R	0.68 ± 0.16	<0.51	0.30 ± 0.17
8800, L	0.66 ± 0.17	0.94 ± 1.00	0.25 ± 0.20
8900, R	0.69 ± 0.20	0.87 ± 0.68	0.56 ± 0.30
9000, L	0.62 ± 0.21	<0.43	0.56 ± 0.24
9100, R	0.78 ± 0.28	<0.70	0.39 ± 0.18
9200, L	0.63 ± 0.17	0.59 ± 0.74	0.44 ± 0.26
9300, R 9400, L	$\begin{array}{c} 0.67 + 0.18 \\ 0.44 + 0.20 \end{array}$	<0.50 <0.39	<0.12
9500, R	0.69 + 0.15	1.86 + 0.83	0.29 ± 0.22
9600, L	0.67 + 0.17	0.70 ± 0.83	$\begin{array}{c} 0.45 \pm 0.21 \\ 0.24 \pm 0.23 \end{array}$
9700, R	0.76 + 0.17	1.35 + 0.66	0.24 ± 0.23 0.36 + 0.23
9800, L	1.00 + 0.18	1.65 + 0.67	0.36 ± 0.23 0.37 ± 0.35
9900, R	0.78 + 0.23	1.29 + 1.35	0.59 ± 0.27
10000, L	0.55 + 0.13	<0.48	0.18 + 0.24
10100, R	0.44 + 0.21	1.02 + 1.29	0.24 + 0.29
10200, L	0.84 + 0.25	1.20 + 0.48	0.29 + 0.22
10300, R	0.60 + 0.23	0.84 + 1.72	0.55 + 0.27
10400, L	0.85 ± 0.23	<0.71	1.12 ± 0.32
10500, R	0.82 + 0.23	1.48 + 0.58	0.29 + 0.21
10600, L	0.77 ± 0.29	1.56 + 1.92	1.09 ± 0.39
10700, R	0.81 ± 0.29	<0.72	0.20 + 0.39
10800, L	0.85 ± 0.25	0.76 ± 0.84	0.60 ± 0.29
10900, R	0.83 ± 0.22	<0.71	0.72 ± 0.36
11000, L	0.62 ± 0.15	<0.51	0.31 ± 0.17
11100, R	0.66 ± 0.14	1.23 + 0.49	0.13 ± 0.08
11200, L	0.59 ± 0.20	1.01 ± 1.33	0.37 ± 0.31
11300, R	1.15 ± 0.23	1.21 + 1.57	0.55 ± 0.37
11400, L	0.81 ± 0.25	1.39 ± 0.75	0.45 ± 0.33
-	0.60 ± 0.17	0.78 ± 0.90	0.51 ± 0.23
11600, L 11700, R	$\begin{array}{c} 0.92 \pm 0.21 \\ 0.82 + 0.19 \end{array}$	0.91 ± 1.06	0.33 ± 0.28
11700, K 11800, L	0.62 ± 0.19 $0.68 + 0.24$	1.84 ± 0.68 <0.51	0.42 ± 0.20
11900, R	0.88 ± 0.24 0.99 ± 0.26	1.60 + 0.96	0.29 ± 0.20
12000, K	0.84 + 0.22	0.64 + 0.62	0.83 ± 0.42
12100, E	0.54 + 0.17	0.78 + 1.07	$\begin{array}{c} 0.83 \pm 0.44 \\ 0.23 \pm 0.13 \end{array}$
12200, L	0.53 + 0.25	<0.78	0.23 ± 0.13 0.43 + 0.37
12300, R	0.90 + 0.23	0.74 + 1.08	0.43 ± 0.37 0.39 ± 0.24
12400, L	0.56 + 0.16	1.24 + 0.94	0.59 ± 0.24 0.51 ± 0.21
, -	<u> </u>	~ · · · · · · · · · · · · · · · · · · ·	0.51 ± 0.21

TABLE 13 (Continued)

RADIONUCLIDE CONCENTRATIONS IN SURFACE SOIL SAMPLES COLLECTED AT 100 M INTERVALS ALONG ROAD #1 WELDON SPRING, MISSOURI

Location	Radion	uclide Concentrati	ons (pCi/g)
	Ra-226	U-238	Th-232
12500 R	0.50 + 0.23	1.25 + 2.74	<0.16
12600, L	0.86 + 0.20	1.31 + 0.75	0.43 + 0.2
12700, R	0.91 + 0.30	5.52 + 2.78	1.19 + 0.5
12800, L	0.57 + 0.20	2.21 + 0.87	0.55 + 0.2
12900, R	0.73 + 0.30	1.81 + 2.06	0.98 + 0.4
13000, L	0.74 ± 0.17	1.73 ± 1.45	0.87 ± 0.4
13100, R	0.47 + 0.12	0.95 + 1.18	0.34 + 0.2
13200, L	0.92 ± 0.25	1.41 ± 1.04	0.90 ± 0.4
13300, R	0.53 ± 0.22	1.08 + 1.10	0.34 + 0.2
13400, L	0.80 ± 0.20	0.84 ± 2.19	0.83 ± 0.3
13500, R	0.71 ± 0.23	1.69 ± 0.86	0.49 ± 0.4
13600, L	1.05 ± 0.27	2.05 ± 1.66	1.17 ± 0.4
13700, R	0.62 ± 0.15	1.64 ± 0.99	0.27 ± 0.2
13800, L	1.16 ± 0.26	3.07 ± 1.65	1.14 ± 0.4
13900, R	0.29 ± 0.05	0.43 ± 0.12	0.14 + 0.0
14000, L	1.21 ± 0.23	2.06 ± 1.64	1.11 ± 0.5
14100, R	0.97 ± 0.19	3.13 ± 0.51	0.35 + 0.26
14200, L	0.62 + 0.27	1.22 + 1.49	0.44 ± 0.39

aRefer to Figure 8.

bErrors are 20 based on counting statistics.

TABLE 14 RADIONUCLIDE CONCENTRATIONS IN SURFACE SOIL SAMPLES COLLECTED AT 100 M INTERVALS ALONG ROAD #2 WELDON SPRING, MISSOURI

$\begin{array}{c ccccccccccccccccccccccccccccccccccc$	-
100, R	2
200, L $0.67 + 0.27$ $1.38 + 1.90$ $0.56 + 0.30$ $0.76 + 0.19$ 0.73 $0.58 + 0.30$ $0.58 + 0.30$ 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50 0.50	.42
300, R $0.76 + 0.19$ < 0.73 $0.58 + 0.49$ < 0.69 $0.49 + 0.50$ < 0.69 $0.49 + 0.50$ < 0.69 $0.49 + 0.50$ < 0.60 , L $0.81 + 0.26$ $1.19 + 1.01$ $0.72 + 0.60$ < 0.60 , L $0.91 + 0.31$ $1.57 + 1.38$ $0.36 + 0.60$ $< 0.68 + 0.25$ < 0.82 $0.56 + 0.68 + 0.19$ $0.88 + 0.50$ $0.55 + 0.68 + 0.19$ $0.88 + 0.50$ $0.55 + 0.68 + 0.19$ $0.84 + 0.92$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ $0.85 + 0.19$ 0.19 0.19 0.19 0.19 0.19 0.19 0.19 0.19 0.19 0.19 0.19 0.19 0.19 0.19 0.19 0.19 0.19 0.19 0.19 0.19 0.19 0.19 0.19 0.19 0.19 0.19 0.19 0.19 0.19 0.19 0.19 0.19 0.19 0.19	
400, L 1.03 ± 0.21 < 0.69 0.49 ± 0.50 0.00 , R 0.81 ± 0.26 1.19 ± 1.01 0.72 ± 0.50 0.00 , L 0.91 ± 0.31 1.57 ± 1.38 0.36 ± 0.50 0.56 ± 0.50 0.56 ± 0.50 0.55 ± 0.50 0.65 ± 0.23 0.60 ± 0.20 $0.60 \pm$	
500, R	
600, L $0.91 + 0.31$ $1.57 + 1.38$ $0.36 + 0.36$ 700 , R $0.96 + 0.25$ < 0.82 $0.56 + 0.36$ 900 , L $0.68 + 0.19$ $0.88 + 0.50$ $0.55 + 0.36$ 900 , R $0.84 + 0.22$ $1.48 + 0.92$ $0.85 + 0.36$ 1000 , L $0.65 + 0.23$ < 0.85 $0.46 + 0.36$ 1100 , R $1.21 + 0.68$ $0.62 + 1.01$ $0.60 + 0.36$ 1200 , L $0.57 + 0.21$ $1.46 + 1.78$ $1.27 + 0.36$ 1300 , R $0.87 + 0.20$ $3.47 + 2.66$ $0.75 + 0.36$ 1400 , L $0.64 + 0.15$ $0.75 + 0.75$ < 0.09	
700, R $0.96 + 0.25$ < 0.82 $0.56 + 0.80$ $0.68 + 0.19$ $0.88 + 0.50$ $0.55 + 0.90$ $0.84 + 0.22$ $1.48 + 0.92$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ 0.90 $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ $0.85 + 0.90$ 0.90 0.90 0.90 0.90 0.90 0.90 0.90 0.90 0.90 0.90 0.90 0.90 0.90 0.90 0.90 0.90 0.90 0.90 0.90	-32
800, L 0.68 ± 0.19 0.88 ± 0.50 0.55 ± 0.90 900 , R 0.84 ± 0.22 1.48 ± 0.92 0.85 ± 0.92 1000 , L 1000 , L 1000 , L 1000 , R 1000 , L 1000 , R 1000	. 50
900, R 0.84 ± 0.22 1.48 ± 0.92 0.85 ± 0.00 1000 , L 0.65 ± 0.23 $0.85 \pm 0.046 \pm 0.00$ 1100 , R 1.21 ± 0.68 0.62 ± 1.01 0.60 ± 0.00 1200 , L 0.57 ± 0.21 1.46 ± 1.78 1.27 ± 0.00 1300 , R 0.87 ± 0.20 3.47 ± 2.66 0.75 ± 0.00 1400 , L 0.64 ± 0.15 0.75 ± 0.75	. 35
1000, L 0.65 ± 0.23 < 0.85 0.46 ± 0.100 , R 1.21 ± 0.68 0.62 ± 1.01 0.60 ± 0.120 , L 0.57 ± 0.21 1.46 ± 1.78 1.27 ± 0.120 , R 0.87 ± 0.20 3.47 ± 2.66 0.75 ± 0.120 , L 0.64 ± 0.15 0.75 ± 0.75 < 0.09	.43
1100, R	. 40
1200, L 0.57 ± 0.21 1.46 ± 1.78 1.27 ± 0 1300, R 0.87 ± 0.20 3.47 ± 2.66 0.75 ± 0 1400, L 0.64 ± 0.15 0.75 ± 0.75 <0.09	•26
1300, R 0.87 ± 0.20 3.47 ± 2.66 0.75 ± 0.1400 , L 0.64 ± 0.15 0.75 ± 0.75 < 0.09	
1400, L 0.64 \pm 0.15 0.75 \pm 0.75 $<$ 0.09	
1500, R 0.76 ± 0.23 < 0.55 < 0.11	
1600, L 0.90 ± 0.25 < 0.92 0.33 ± 0	
1700, R 0.87 ± 0.28 0.60 ± 0.59 1.10 ± 0	
1800, L 1.02 \pm 0.26 1.83 \pm 1.07 0.99 \pm 0	
1900, L 0.72 ± 0.20 < 1.34 0.55 ± 0	
2000, L 0.39 ± 0.10 0.51 ± 0.28 0.21 ± 0	
2100, R 0.58 ± 0.19 5.81 ± 1.34 0.39 ± 0	
2200, L 0.76 ± 0.22 < 0.54 < 0.14	
2300, R 0.65 ± 0.26 2.50 ± 1.58 0.56 ± 0	
2400, L 0.85 ± 0.29 1.38 ± 1.01 1.04 ± 0	
2500, R 0.61 ± 0.30 $(0.62 0.78 \pm 0.00)$	
2600, L \cdot 1.10 \pm 0.34 1.34 \pm 1.91 1.26 \pm 0	
2700, R 1.18 \pm 0.26 1.12 \pm 0.61 1.30 \pm 0	
2800, L 0.87 ± 0.31 0.97 ± 2.03 1.19 ± 0	
2900, R 1.16 \pm 0.24 4.13 \pm 2.21 1.05 \pm 0	
3000, L 0.72 \pm 0.23 1.31 \pm 0.87 1.08 \pm 0	
3100, R 1.26 ± 0.42 1.66 ± 1.12 1.40 ± 0	
3200, L. 1.13 ± 0.29 1.00 ± 2.59 1.06 ± 0	.33

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aRefer to Figure 8. bErrors are 20 based on counting statistics.

TABLE 15

RADIONUCLIDE CONCENTRATIONS IN SURFACE SOIL SAMPLES COLLECTED AT 100 M INTERVALS ALONG ROAD #3

WELDON SPRING, MISSOURI

L	ocatio	ρηa	•		Radio	onucl:	ide Cor	ncentra	tions	(pCi/	g)
				Ra-22				-238			-232
	0,	L		.66 + (1.25	<u>+</u> 0.48		0.12	+ 0.11
	100,			.82 <u>+</u> (<u>+ 1.20</u>			+ 0.47
	200,		0.	.83 I (.20			F 1.30			+ 0.50
	300,			.80 + 0			1.50				<u>+</u> 0.51
	400,			23 ± 0			<1.				+ 0.45
	500,			$\frac{.72 + (}{.}$			0.89				+ 0.21
	600,			$.81 \pm 0$			0.87	_			<u>+</u> 0.23
	700,			$\frac{.55 + (}{.53 + (}$			0.67				.09
	800,			69 + 0			<07				+ 0.43
	900,			.47 + (2.13				+ 0.25
	1000,			$\frac{.68 + 0}{.51 + 0}$			< 0.				.16
	1100,			.51 <u>+</u> (.43 + (.49			+ 0.25
	1200,						<0.				-11
	1300,			.70 <u>+</u> (.78 + (_	1.14			+ 0.29
	1400,			.74 ± (<0.				$\frac{-}{1}$ 0.37
	1500, 1600,			.61 + 0			2.85 ± 1.96 ±				+ 0.19
	1700,			$\frac{101}{98} + 0$			<0,				+ 0.37
	1800,			03 + 0			<0.				+ 0.34 + 0.37
	1900,			.73 + (<0.				+ 0.28
	2000,			.00 + 0			1.11 +				+ 0.39
	2100,			$\frac{1}{72} + 0$			0.98				+ 0.42
	2200,			98 + 0			1.28				+ 0.53
	2300,			12 + (<0.	_			+ 0.18
	2400,			.97 🗕 (<0.				+ 0.26
	2500,			.76 + (0.94 +				+ 0.29
	2600,			.80 + 08.			<0.				+ 0.18
	2700,			.02 + (1.87	1.50			+ 0.39
	2800,	L	1.	$19 \pm ($	1.21		1.96	1.49			+ 0:57
	2900,	R		05 + 0			<0.			1.16	+ 0.53
	3000,			.54 <u>+</u> (1.41 +			1.41	\pm 0.80
	3100,			.61 + (. 70			+ 0.29
	3200,			.76 I (.55	-		+ 0.27
	3300,			78 + 0			0.59				+ 0.32
	3400,			$.74 \pm ($			<0,				\pm 0.23
	3500,			$\frac{11 + 0}{2}$				1.55			+ 0.57
	3600,			$.98 \pm 0$			$0.93 = \frac{1}{4}$	_			<u>+</u> 0.45
	3700,	L	0.	.86 + (1.19		<0.	.63		0.38	<u>+</u> 0.33

aRefer to Figure 9.

bErrors are 2g based on counting statistics.

TABLE 16 RADIONUCLIDE CONCENTRATIONS IN SURFACE SOIL SAMPLES COLLECTED AT 100 M INTERVALS ALONG ROAD #4 WELDON SPRING, MISSOURI

Locationa	Radionu	Radionuclide Concentrations (pCi/g)			
	Ra-226	U-238	Th-232		
0, L	1.01 + 0.23 ^b	1.24 + 1.42	0.77 + 0.42		
100, 0.5L	0.73 + 0.16	<0.50	<0.13		
200, 0.5R	0.93 ± 0.22	1.97 + 1.45	0.80 + 0.38		
300, 0.5L	0.64 + 0.28	<0.65	0.37 + 0.33		
400, 0.5R	0.64 + 0.15	<0.49	0.30 ± 0.25		
500, 0.5L	0.74 + 0.21	0.75 + 1.27	0.29 + 0.31		
600, R	0.90 ± 0.21	<0.50	0.32 + 0.19		
700, L	0.80 + 0.21	0.89 + 1.13	<0.13		
717, R	0.98 ± 0.21	1.22 + 0.56	0.69 + 0.37		

aRefer to Figure 9. bErrors are 20 based on counting statistics.

TABLE 17 RADIONUCLIDE CONCENTRATIONS IN SURFACE SOIL SAMPLES COLLECTED AT 100 M INTERVALS ALONG ROAD #5 WELDON SPRING, MISSOURI

Locationa	Radionu	clide Concentrati	ons (pCi/g)
	Ra-226	U-238	Th-232
0, 0.5R	0.54 + 0.14 ^b	1.45 + 1.20	<0.17
100, 0.5L	0.39 ± 0.13	<0.63	0.39 + 0.2
200, 0.5R	0.80 ± 0.29	1.29 + 1.57	0.41 + 0.2
300, 0.5L	0.76 + 0.19	1.16 + 1.04	0.38 + 0.38
400, R	0.73 ± 0.17	<0.66	0.34 + 0.28
500, L	0.54 + 0.13	0.54 + 0.67	0.54 ± 0.3
600, R	0.57 ± 0.17	<0.59	0.34 + 0.3
700, L	0.61 + 0.17	<0.61	0.44 + 0.1
800, R	0.62 + 0.18	<0.55	0.37 ± 0.2
900, L	0.67 + 0.21	0.63 + 0.96	0.32 + 0.26
1100, L	0.67 ± 0.31	<0.84	0.48 ± 0.43
1200, R	0.61 + 0.15	0.30 + 0.28	<0.10
1300, L	0.85 ± 0.33	1.54 ± 2.06	0.62 + 0.36
1400, R	0.88 ± 0.23	1.12 + 1.16	0.26 + 0.26
1500, R	0.74 ± 0.15	0.74 ± 0.76	1.12 ± 0.35
1600, L	0.85 ± 0.20 .	1.15 + 1.35	0.24 + 0.32
1700, R	0.65 ± 0.16	<0.56	<0.11
1800, L	0.71 ± 0.16	0.63 ± 0.51	0.33 + 0.20
1900, R	0.61 ± 0.19	0.70 + 1.55	<0.13

aRefer to Figure 8. bErrors are 20 based on counting statistics.

RADIONUCLIDE CONCENTRATIONS IN SURFACE SOIL SAMPLES COLLECTED AT 100 M INTERVALS ALONG ROAD #6 WELDON SPRING, MISSOURI

Locationa	Radionu	clide Concentration	ons (pCi/g)
	Ra-226	U-238	Th-232
0, R	0.40 + 0.16 ^b	1.08 + 0.42	0.79 <u>+</u> 0.26
100, L	0.69 ± 0.21	1.56 + 1.60	0.99 ± 0.44
200, R	0.85 ± 0.24	0.64 ± 2.80	1.36 + 0.42
300, L	0.93 ± 0.18	0.89 ± 0.60	1.15 + 0.37
400, R	0.44 ± 0.12	0.53 ± 0.67	0.42 ± 0.27
500, L 600, R	$\begin{array}{c} 0.56 \pm 0.18 \\ 0.74 \pm 0.27 \end{array}$	<0.54 1.06 + 1.25	$\begin{array}{c} 0.45 \pm 0.20 \\ 1.35 \pm 0.41 \end{array}$
700, L	0.74 ± 0.27 0.36 ± 0.16	0.73 + 0.39	0.52 ± 0.41
800, R	0.73 + 0.10	1.02 + 1.03	0.32 ± 0.20 0.31 ± 0.23
900, L	0.74 + 0.18	$\frac{1.50 + 1.94}{1.94}$	<0.12
1000, R	0.57 + 0.22	1.16 + 1.60	0.63 + 0.24
1100, R	0.63 ± 0.16	2.16 + 0.52	0.27 + 0.24
1200, R	1.16 ± 0.28	3.05 + 1.83	1.54 ± 0.47
1300, L	0.47 ± 0.14	1.44 + 0.96	0.21 + 0.25
1400, R	1.07 ± 0.27	<0.87	0.47 ± 0.29
1500, L	0.72 ± 0.24	1.66 ± 0.90	1.27 ± 0.36
1600, R	0.86 ± 0.25	1.23 ± 1.32	0.32 ± 0.27
1700, L	0.84 ± 0.30	2.20 + 1.87	1.09 ± 0.58
1800, R	0.77 ± 0.16	<0.55	0.41 ± 0.29
1900, L	0.82 ± 0.20	0.84 ± 1.18	0.29 ± 0.51
2000, R	0.77 ± 0.18	<0.74	0.54 ± 0.24
2100, L 2200, R	$\begin{array}{c} 1.06 \pm 0.21 \\ 0.59 \pm 0.23 \end{array}$	1.32 ± 1.03 1.62 ± 3.07	0.97 ± 0.34 < 0.18
2300, L	0.39 ± 0.23 0.48 ± 0.16	1.15 ± 3.07 $1.15 + 1.59$	0.17 + 0.10
2400, E 2400, R	0.48 ± 0.16 0.63 ± 0.15	0.31 ± 0.31	0.17 ± 0.10 0.18 ± 0.20
2500, L	0.69 + 0.22	<0.80	0.18 ± 0.26 0.59 ± 0.26
2600, R	0.87 + 0.20	<0.67	0.46 + 0.38
·			

^aRefer to Figure 9. ^bErrors are 20 based on counting statistics.

RADIONUCLIDE CONCENTRATIONS IN SURFACE SOIL SAMPLES
COLLECTED AT 100 M INTERVALS ALONG ROAD #7
WELDON SPRING, MISSOURI

Locationa	Radionu	clide Concentration	ons (pCi/g)
	Ra-226	U-238	Th-232
U, R	1.00 ± 0.26^{b}	1.67 <u>+</u> 0.69	0.55 ± 0.29
100, L	1.06 + 0.28	2.56 + 2.13	0.62 + 0.44
200, R	0.76 ± 0.16	<0.54	0.53 + 0.2
300, L	0.75 ± 0.17	0.24 + 0.40	0.61 + 0.28
400, R	0.95 ± 0.24	<0.85	0.85 + 0.3
500, L	0.84 + 0.24	0.95 + 0.41	1.92 + 0.9
600, R	0.79 + 0.32	1.35 + 3.73	0.95 + 0.7
700, L	0.67 + 0.25	0.48 + 1.77	0.57 ± 0.36

aRefer to Figure 10.

bErrors are 20 based on counting statistics.

TABLE 20

DIRECT RADIATION LEVELS MEASURED AT SOIL SAMPLE LOCATIONS
DITCH #4
WELDON SPRING, MISSOURI

Location ^a (m)	Gamma Exposure Rates at 1 m Above the Surface $(\mu R/h)$	Gamma Exposure Rates at the Surface $(\mu R/h)$
Ditch #4		
0	8	14
100	10	10
200	10	8
300	10	10
400	7	7
500	7	7
600	7	7
700	7	7
800	7	7
900	6	7
1000	6	6
1100	6	6
1200	6	6
1300	6	7
1400	6	6
1500	6	6
1300	o	O .
Ditch #4A		•
0	6	6
100	7	7
200	6	6
300	7	7
400	6	ż
500	<u>.</u> 6	6
600	7	7
700	, 6	7
800	7	7
877	6	6
	· ·	U
itch Originating		
oc. #1 0	- 8	8
100	7	8
200	7	7
300	6	6
400	6	6
500	7	6
600	7	. 7

aRefer to Figure 4.

TABLE 21

RADIONUCLIDE CONCENTRATIONS IN SEDIMENT

COLLECTED AT 100 M INTERVALS - DITCH #4, #4A AND THE DITCH FROM LOCATION #1

WELDON SPRING, MISSOURI

Locati	ona		clide Concentration	ons (pCi/g)
(m)		Ra-226	U-238	Th-232
Ditch #4				.*
0,	С	$1.02 + 0.32^{b}$	59.2 + 3.6	1.71 + 0.45
100,		$\frac{1.32 + 0.32}{1.22 + 0.25}$	0.52 + 5.67	1.05 + 0.55
200,				
300,	С	1.39 + 0.23	12.1 + 3.6	1.80 + 0.72
400,	С	1.07 ± 0.22	1.91 ± 2.24	1.33 ± 0.66
500,				
600,		1.06 ± 0.26	<1.42	1.13 ± 0.60
700,		1.57 + 0.24	2.24 + 1.24	0.90 ± 0.28
800,		$\begin{array}{c} 1.23 \pm 0.27 \\ 0.92 + 0.23 \end{array}$	$\begin{array}{c} 2.43 \pm 1.89 \\ 7.24 + 3.11 \end{array}$	1.16 ± 0.49
900, 1000,	R L	0.92 + 0.23	7.24 + 3.11	0.86 <u>+</u> 0.54
1100,		0.70 + 0.20	<0.73	0.62 + 0.34
1200,		0.91 ± 0.30	0.75 + 2.58	0.90 + 0.48
1300,		_		-
1400,		0.79 ± 0.21	2.84 <u>+</u> 1.61	1.10 + 0.52
1500,	R	0.93 ± 0.31	<1.03	2.23 ± 0.61
Ditch #4A				
0,	R	1.07 + 0.22	2.08 + 1.43	1.15 ± 0.39
100,		0.86 ± 0.29	1.20 ± 0.56	1.13 ± 0.31
200,		0.53 ± 0.26	<0.88	1.42 ± 0.37
300,		0.72 ± 0.25	<1.11	1.18 ± 0.38
400,		1.36 ± 0.31	2.75 + 2.02	1.66 + 1.04
500, 600,		$\begin{array}{c} 0.75 \pm 0.28 \\ 0.84 \pm 0.22 \end{array}$	$ \begin{array}{r} 1.41 + 1.29 \\ 0.46 + 0.57 \end{array} $	$\begin{array}{c} 1.28 \pm 0.41 \\ 1.53 \pm 0.44 \end{array}$
700,		1.03 + 0.33	$\frac{1}{3.95} + \frac{1}{2.41}$	1.44 + 0.53
800,		1.06 + 0.32	<1.07	0.83 ± 0.39
877,		0.71 ± 0.20	0.95 ± 0.85	1.22 ± 0.37
Location	#1			
OM,	C	$0.90 + 0.25^{b}$	2.19 + 0.68	1.57 + 0.38
100м,	С	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	7.65 + 2.07	1.22 + 0.43
200M,	С	0.73 1 0.34	7.56 ± 2.39	1.52 ± 0.46
300M,	С	0.84 + 0.27	2.77 + 2.18	1.60 ± 0.50
400M,	С	0.63 ± 0.15	1.66 ± 1.00	0.84 ± 0.26
500M,	C	0.80 ± 0.20	2.69 ± 0.94	0.97 ± 0.30
600M,	С	0.89 ± 0.21	6.58 ± 2.64	1.32 ± 0.61

a Refer to Figure 4.

b Errors are 20 based on counting statistics.

TABLE 22

RADIONUCLIDE CONCENTRATIONS IN SURFACE WATER SAMPLES WELDON SPRING, MISSOURI

Location ^{a.}	Radionuclide Conce	ntrations (pCi/l)
	Gross Alpha	Gross Beta
Pond Off Railroad #2 (North Bank)	1.93 <u>+</u> 0.47b	8.09 <u>+</u> 0.78
Pond Off Road #6 (West Bank)	0.74 <u>+</u> 0.57	2.09 <u>+</u> 1.00
Raffinate Ditch, 22M	4.28 + 2.24	3.22 <u>+</u> 2.57
Ditch 4A - Origin	1.38 <u>+</u> 0.74	4.98 <u>+</u> 1.00

a Refer to Figure 4.

b Errors are 20 based on counting statistics.

TABLE 23

DIRECT RADIATION LEVELS MEASURED AT SOIL SAMPLE LOCATIONS
SCHOTE CREEK
WELDON SPRING, MISSOURI

<u>Locati</u> (m)		Gamma Exposure Rates at 1 m Above the Surface (µR/h)	Gamma Exposure Rates at the Surface (µR/h)
0	1R	7	8
100	1L	8	8
200	1R	8	8
300	1L	8	8
400	1R	8	8
500	1L	8	8
600	1 R	7	7
700	1L	7	. 8
727	1R	7	7

aRefer to Figure 4.

TABLE 24 RADIONUCLIDE CONCENTRATIONS IN SEDIMENT SAMPLES COLLECTED AT 100 M INTERVALS ALONG SCHOTE CREEK WELDON SPRING, MISSOURI

Locationa	Radionucl	ide Concentratio	ons (pCi/g)
(m)	Ra-226	U-238	Th-232
0 R 100 L 200, R 300, L 400, R 500, L 600 L 727, R	$\begin{array}{c} 0.93 & \pm & 0.22^{b} \\ 0.83 & \pm & 0.20 \\ 0.60 & \pm & 0.12 \\ 0.77 & \pm & 0.24 \\ 1.20 & \pm & 0.25 \\ 0.58 & \pm & 0.19 \\ 0.45 & \pm & 0.21 \\ 0.44 & \pm & 0.14 \\ \end{array}$	1.30 ± 1.74 <0.88 1.28 ± 0.70 1.20 ± 1.73 1.54 ± 1.51 0.84 ± 0.64 1.16 ± 1.18 0.57 ± 0.59	$ \begin{array}{r} 1.14 & \pm & 0.34 \\ 1.04 & \pm & 0.34 \\ 0.83 & \pm & 0.23 \\ 0.99 & \pm & 0.32 \\ 1.40 & \pm & 0.46 \\ 0.58 & \pm & 0.22 \\ 0.57 & \pm & 0.46 \\ 0.86 & \pm & 0.56 \end{array} $

a Refer to Figures 4. b Errors are 2σ based on counting statistics.

TABLE 25

RADIONUCLIDE CONCENTRATIONS IN SEDIMENT SAMPLES FROM ARMY PROPERTY PONUS WELDON SPRING, MISSOURI

Location ^a	Radionucli	de Concentrati	ons (pCi/g)
•	Ra-226	U-238	Th-232
Pond Off Railroad #2 (North Bank)	0.54 <u>+</u> 0.19 ^b	0.78 <u>+</u> 1.21	0.36 + 0.40
Pond Off Road #6 (West Bank)	1.02 <u>+</u> 0.27	10.3 <u>+</u> 2.2	1.43 <u>+</u> 0.47

a Refer to Figure 4.b Errors are 2σ based on counting statistics.

RADIONUCLIDE CONCENTRATIONS IN SPLIT SPOON SOIL SAMPLES COLLECTED FROM BOREHOLES ON THE ARMY RESERVE PROPERTY WELDON SPRING, MISSOURI

TABLE 26

Boreholea	Depth	Radionucl	lide Concentration	on (pCi/g)
Location	(cm)	Ra-226	U-238	Th-232
63	0 - 60 90 - 120 120 - 150 240 - 270 270 - 300 390 - 420 420 - 450 540 - 550	$ \begin{array}{cccccccccccccccccccccccccccccccccccc$	1.73 ± 1.29 <0.85 3.15 ± 1.70 <0.84 2.34 ± 1.88 <0.87 3.34 ± 1.11 <0.92	$ \begin{array}{cccccccccccccccccccccccccccccccccccc$
64	$ \begin{array}{r} 0 - 30 \\ 30 - 60 \\ 90 - 120 \\ 120 - 150 \\ 240 - 270 \\ 270 - 300 \\ 390 - 420 \\ 420 - 450 \end{array} $	$ \begin{array}{cccccccccccccccccccccccccccccccccccc$	<pre><1.32 1.69 + 2.09 2.79 + 1.36 1.46 + 1.54 1.81 + 1.72 1.68 + 1.54 1.28 + 0.82 <1.12</pre>	$ \begin{array}{c} 1.63 + 0.87 \\ 1.65 + 0.45 \\ 1.95 + 0.70 \\ 1.12 + 0.78 \\ 1.44 + 0.73 \\ 1.37 + 0.52 \\ 1.29 + 0.49 \\ 1.89 + 0.62 \end{array} $
65	0 - 30 30 - 60 90 - 120 120 - 150 240 - 270 270 - 300	$ \begin{array}{c} 1.06 \pm 0.34 \\ 1.17 \pm 0.30 \\ 1.18 \pm 0.29 \\ 1.25 \pm 0.30 \\ 0.90 \pm 0.26 \\ 0.63 \pm 0.30 \end{array} $	$ \begin{array}{r} 1.71 + 1.50 \\ 8.87 + 2.80 \\ 3.45 + 1.28 \\ 1.18 + 1.52 \\ 1.16 + 0.78 \\ 1.62 + 1.64 \end{array} $	$ \begin{array}{c} 1.17 + 0.57 \\ 2.13 + 0.61 \\ 1.48 + 0.52 \\ 1.60 + 0.48 \\ 1.35 + 0.55 \\ 1.25 + 0.65 \end{array} $
	$ 0 - 30 \\ 30 - 60 \\ 90 - 120 \\ 120 - 150 \\ 240 - 270 \\ 270 - 300 \\ 390 - 405 \\ 540 - 555 $	$ \begin{array}{c} 1.21 & + & 0.35 \\ 1.19 & + & 0.35 \\ 1.66 & + & 0.36 \\ 1.86 & + & 0.35 \\ 0.98 & + & 0.28 \\ 0.71 & + & 0.28 \\ 1.24 & + & 0.57 \\ 1.46 & + & 0.99 \end{array} $	$ \begin{array}{cccccccccccccccccccccccccccccccccccc$	$ \begin{array}{cccccccccccccccccccccccccccccccccccc$
67	0 - 30 $30 - 60$ $90 - 120$ $120 - 150$ $240 - 270$ $270 - 300$ $390 - 420$ $420 - 450$	$\begin{array}{c} 0.99 \ \pm \ 0.32 \\ 1.02 \ \pm \ 0.37 \\ 1.08 \ \pm \ 0.34 \\ 0.59 \ \pm \ 0.35 \\ 0.75 \ \pm \ 0.25 \\ 1.13 \ \pm \ 0.33 \\ 0.32 \ \pm \ 0.25 \\ 0.54 \ \pm \ 0.29 \\ \end{array}$	3.32 ± 2.78 4.85 ± 1.70 <0.87 1.60 ± 2.06 <0.81 <0.88 <0.88 0.86 ± 1.76	$\begin{array}{c} <0.21 \\ 1.45 + 0.92 \\ 1.01 + 0.56 \\ 1.20 + 0.72 \\ 1.19 + 0.39 \\ 1.24 + 0.61 \\ 1.09 + 0.48 \\ 1.11 + 0.41 \end{array}$

RADIONUCLIDE CONCENTRATIONS IN SPLIT SPUON SOIL SAMPLES

COLLECTED FROM BOREHOLES ON THE ARMY RESERVE PROPERTY WELDON SPRING, MISSOURI

Borehole Location	Depth (cm)	Radionuc Ra-226	lide Concentratio	on (pCi/g) Th-232
200222011	(Cin)	114 225	U 2 30	111 232
68	0 - 30 30 - 60 90 - 120 120 - 150 240 - 255 390 - 450 540 - 570	$ \begin{array}{cccccccccccccccccccccccccccccccccccc$	3.93 ± 2.66 6.00 ± 2.67 1.16 ± 2.13 < 0.79 < 0.63 1.90 ± 1.29 < 0.71	$ \begin{array}{cccccccccccccccccccccccccccccccccccc$
69	0 - 30 30 - 60 90 - 120 120 - 150 240 - 270 270 - 300 390 - 420 420 - 450 540 - 545	$\begin{array}{c} 0.97 \ \pm 0.33 \\ 0.58 \ \pm 0.27 \\ 0.79 \ \pm 0.26 \\ 0.74 \ \pm 0.26 \\ 1.01 \ \pm 0.30 \\ 1.11 \ \pm 0.34 \\ 0.82 \ \pm 0.28 \\ 0.67 \ \pm 0.36 \\ 1.52 \ \pm 0.85 \end{array}$	$\begin{array}{c} 2.03 & \pm & 1.02 \\ 0.71 & \pm & 1.35 \\ 1.93 & \pm & 1.09 \\ 0.62 & \pm & 0.93 \\ 1.52 & \pm & 0.95 \\ 1.34 & \pm & 1.33 \\ 1.42 & \pm & 1.19 \\ & < 0.65 \\ 2.82 & \pm & 1.98 \end{array}$	$\begin{array}{c} 0.80 \ \pm \ 0.36 \\ 0.98 \ \pm \ 0.39 \\ 1.18 \ \pm \ 0.41 \\ 1.39 \ \pm \ 0.47 \\ 1.74 \ \pm \ 0.52 \\ 1.54 \ \pm \ 0.48 \\ 1.08 \ \pm \ 0.41 \\ 1.09 \ \pm \ 0.44 \\ < 0.72 \end{array}$
70	$ \begin{array}{rrrr} 0 & - & 30 \\ 30 & - & 60 \\ 90 & - & 120 \\ 120 & - & 150 \\ 240 & - & 270 \\ 270 & - & 300 \\ 390 & - & 420 \\ 420 & - & 450 \\ \end{array} $	$ \begin{array}{cccccccccccccccccccccccccccccccccccc$	<0.91 $3.49 + 1.09$ $2.00 + 1.67$ $2.53 + 1.48$ $2.01 + 1.32$ $1.37 + 1.76$ <0.73 $0.34 + 0.78$	$ \begin{array}{c} 1.11 + 0.42 \\ 1.62 + 0.87 \\ 1.47 + 0.63 \\ 1.77 + 0.58 \\ 1.31 + 0.70 \\ 1.38 + 0.78 \\ 0.88 + 0.49 \end{array} $
71	0 - 30 30 - 60 90 - 120 120 - 150 240 - 270 390 - 420 420 - 450 540 - 570 570 - 600 690 - 720 720 - 750 840 - 870 870 - 900 990 - 102 1140 - 1155	$\begin{array}{c} 0.94 & \pm & 0.31 \\ 1.01 & \pm & 0.35 \\ 1.11 & \pm & 0.35 \\ 1.55 & \pm & 0.33 \\ 0.74 & \pm & 0.29 \\ 0.64 & \pm & 0.23 \\ 0.81 & \pm & 0.24 \\ 0.49 & \pm & 0.34 \\ 0.62 & \pm & 0.27 \\ 1.32 & \pm & 0.33 \\ 2.11 & \pm & 0.38 \\ 1.35 & \pm & 0.40 \\ 1.18 & \pm & 0.29 \\ 1.28 & \pm & 0.30 \\ 0.58 & \pm & 0.28 \\ \end{array}$	<0.86 1.06 ± 0.75 1.18 ± 1.83 2.53 ± 1.04 <0.79 1.84 ± 0.77 0.89 ± 1.64 <0.86 1.21 ± 1.62 2.29 ± 1.50 9.88 ± 2.66 2.49 ± 1.46 <0.98 2.09 ± 1.36 0.92 ± 1.40	$\begin{array}{c} 1.03 & + & 0.58 \\ 0.25 & + & 0.37 \\ 1.22 & + & 0.56 \\ 1.34 & + & 0.64 \\ 0.86 & + & 0.34 \\ 1.56 & + & 0.48 \\ 1.02 & + & 0.38 \\ 1.58 & + & 0.43 \\ 0.68 & + & 0.60 \\ 2.11 & + & 0.69 \\ 2.58 & + & 0.67 \\ 2.71 & + & 0.57 \\ 2.10 & + & 0.47 \\ 1.79 & + & 0.47 \\ 1.39 & + & 0.47 \\ \end{array}$

RADIONUCLIDE CONCENTRATIONS IN SPLIT SPOON SOIL SAMPLES

COLLECTED FROM BOREHOLES ON THE ARMY RESERVE PROPERTY WELDON SPRING, MISSOURI

Borehole	Depth	Radionuo	lide Concentrati	on (pCi/g)
Location	(cm)	Ra-226	U-238	Th-232
72	0 - 30	0.88 + 0.27	0.66 + 0.66	0.76 + 0.4
	30 - 60	1.27 ± 0.26	1.91 + 1.34	1.29 + 0.3
	90 - 120	1.16 ± 0.37	2.77 + 1.67	1.60 + 0.78
	120 - 150	1.04 ± 0.27	1.84 ± 1.66	1.52 ± 0.5
	240 - 270	1.03 ± 0.34	1.02 ± 1.57	1.41 + 0.48
	270 - 300	0.89 ± 0.23	1.14 ± 1.63	1.41 ± 0.6
	390 - 420 420 - 450	0.82 ± 0.36 0.66 ± 0.20	1.87 ± 0.85	1.26 + 0.49
	540 - 550	0.63 ± 0.20 $0.63 + 0.56$	<0.70	1.17 ± 0.4
	J40 - JJ0	0.03 ± 0.36	3.51 ± 1.41	1.25 ± 0.63
73	0 - 30	0.63 ± 0.43	<0.78	0.97 ± 0.54
	30 - 60	0.44 ± 0.17	$\frac{1.71 + 0.80}{1.000}$	0.51 ± 0.20
	90 - 120 120 - 150	0.71 + 0.23	3.09 ± 1.76	0.83 ± 0.39
	120 - 150 240 - 270	0.42 ± 0.17 0.61 ± 0.20	<0.79	<0.25
	270 - 300	0.61 ± 0.20 0.40 ± 0.33	3.28 + 1.46 $1.53 + 1.37$	0.67 + 0.63
	390 - 450	0.38 + 0.22	<0.68	$\begin{array}{c} 1.21 \pm 0.49 \\ 0.81 \pm 0.30 \end{array}$
	540 - 570	0.56 ± 0.21	1.70 + 0.89	1.45 ± 0.59
	570 - 600	0.59 + 0.30	<0.78	1.05 + 0.44
	690 - 720	0.69 ± 0.21	1.33 + 1.26	0.86 + 0.5
	720 - 750	0.63 + 0.35	<0.72	1.05 ± 0.51
	840 - 870	0.53 ± 0.27	0.30 ± 0.59	0.89 ± 0.41
	870 - 900	0.64 <u>+</u> 0.33	<0.83	1.44 + 0.47
	990 - 1020	1.50 ± 0.29	2.16 ± 0.80	1.07 \pm 0.3
74	0 - 30	1.37 + 0.53	3.05 + 1.24	1.63 + 0.90
	30 - 60	0.91 ± 0.32	1.97 + 1.47	1.70 ± 0.49
	90 - 120	0.89 ± 0.25	2.06 + 1.19	1.28 + 0.36
	120 - 150	0.83 ± 0.28	0.55 ± 2.13	1.24 ± 0.4
	240 - 270	0.93 ± 0.23	0.93 ± 1.24	1.40 ± 0.46
	270 - 300	0.79 ± 0.26	<0.80	1.66 ± 0.4
	390 - 420 420 - 450	0.78 ± 0.30	0.65 ± 0.73	1.33 ± 0.77
	540 - 570	0.69 ± 0.20 0.71 ± 0.34	1.27 + 1.40	0.88 ± 0.46
	570 - 600	0.75 + 0.21	1.50 ± 1.32 $4.09 + 1.23$	0.93 ± 0.40
	690 - 720	0.75 ± 0.21	<0.89 <0.89	1.59 ± 0.53 1.15 ± 0.48
	720 - 750	0.97 + 0.38	<1.23	$\frac{1.49 + 0.73}{1.49 + 0.73}$
	840 - 870	0.76 + 0.27	1.87 + 1.24	1.07 + 0.56
	870 - 900	0.85 + 0.21	1.87 + 1.19	0.93 + 0.49
	990 - 1020	0.90 ± 0.30	1.24 ± 1.86	1.51 ± 0.51
	1020 - 1050	0.58 ± 0.26	<0.76	1.54 + 0.54
	1140 - 1170	0.61 + 0.22	1.37 + 1.27	1.56 ± 0.54

RADIONUCLIDE CONCENTRATIONS IN SPLIT SPOON SOIL SAMPLES

COLLECTED FROM BOREHOLES ON THE ARMY RESERVE PROPERTY WELDON SPRING, MISSOURI

Borehole	Depth	Radionuc	lide Concentration	on (pCi/g)
Location	(cm)	Ra-226	U−238	Th-232
	1170 - 1200 1290 - 1320 1320 - 1350 1440 - 1470 1470 - 1500 1590 - 1620 1620 - 2650	$\begin{array}{c} 0.63 \pm 0.22 \\ 0.74 \pm 0.30 \\ 0.82 \pm 0.25 \\ 2.18 \pm 0.39 \\ 1.91 \pm 0.37 \\ 1.19 \pm 0.27 \\ 1.02 \pm 0.34 \end{array}$	$\begin{array}{c} 0.94 & \pm & 1.36 \\ 1.42 & \pm & 1.43 \\ 3.37 & \pm & 1.55 \\ 2.10 & \pm & 1.39 \\ 6.41 & \pm & 2.49 \\ 1.51 & \pm & 1.10 \\ < 0.91 \end{array}$	$ \begin{array}{c} 1.42 + 0.41 \\ 0.83 + 0.43 \\ 1.04 + 0.65 \\ 1.07 + 0.66 \\ 1.36 + 0.55 \\ 1.44 + 0.47 \\ 1.79 + 0.43 \end{array} $
75	0 - 30 30 - 60 90 - 120 120 - 150 240 - 270 270 - 300 390 - 420 420 - 450 540 - 570 570 - 600 690 - 720	<0.50 1.15 ± 0.33 1.55 ± 0.43 1.04 ± 0.37 0.82 ± 0.22 0.99 ± 0.32 0.93 ± 0.24 0.79 ± 0.31 0.59 ± 0.36 0.83 ± 0.41 0.52 ± 0.36	$ \begin{array}{cccccccccccccccccccccccccccccccccccc$	$ \begin{array}{cccccccccccccccccccccccccccccccccccc$
76	0 - 30 30 - 60 120 - 150 120 - 150 240 - 270 270 - 300 390 - 420 420 - 450 540 - 570 570 - 600 690 - 720 720 - 750	$\begin{array}{c} 0.96 & \pm & 0.31 \\ 1.01 & \pm & 0.30 \\ 0.85 & \pm & 0.28 \\ 0.94 & \pm & 0.32 \\ 0.94 & \pm & 0.28 \\ 0.78 & \pm & 0.28 \\ 0.63 & \pm & 0.21 \\ 0.60 & \pm & 0.29 \\ 0.89 & \pm & 0.40 \\ 0.92 & \pm & 0.40 \\ 0.48 & \pm & 0.27 \\ 0.33 & \pm & 0.19 \\ \end{array}$	<pre></pre>	$\begin{array}{c} 1.59 \ \pm \ 0.58 \\ 0.51 \ \pm \ 0.96 \\ 1.56 \ \pm \ 0.50 \\ 1.46 \ \pm \ 0.59 \\ 1.37 \ \pm \ 0.56 \\ 1.00 \ \pm \ 0.36 \\ 0.61 \ \pm \ 0.38 \\ 1.29 \ \pm \ 0.41 \\ 1.68 \ \pm \ 0.67 \\ 1.42 \ \pm \ 0.92 \\ 0.78 \ \pm \ 0.41 \\ 0.54 \ \pm \ 0.35 \\ \end{array}$
77	840 - 870 870 - 900 990 - 1020 1020 - 1050 1140 - 1170 1170 - 1200 0 - 30 30 - 60 90 - 120	0.65 + .22 $0.91 + 0.35$ $0.82 + 0.31$ $1.33 + 0.30$ $0.75 + 0.38$ $1.25 + 0.32$ $0.58 + 0.27$ $0.82 + 0.27$ $1.31 + 0.35$	$ \begin{array}{r} $	$ \begin{array}{cccccccccccccccccccccccccccccccccccc$

RADIONUCLIDE CONCENTRATIONS IN SPLIT SPOON SOIL SAMPLES

COLLECTED FROM BOREHOLES ON THE ARMY RESERVE PROPERTY
WELDON SPRING, MISSOURI

Borehole	Depth	Radionu	clide Concentrati	on (pCi/g)
Location	(cm)	Ra-226	Ŭ−238	Th-232
77	120 - 150	0.80 + 0.25	1.04 + 1.80	1.35 + 0.54
	240 - 270	0.70 ± 0.30	$ \begin{array}{c} 1.60 + 1.07 \\ 2.01 + 1.95 \\ 2.77 + 1.02 \end{array} $	0.82 + 0.50
	270 - 300	0.93 ± 0.36	2.01 ± 1.95	1.42 + 0.46
	390 - 420	0.82 ± 0.20	2.77 ± 1.02	1.40 ± 0.4
	420 - 450	0.70 ± 0.50	1.79 + 1.41	0.97 ± 0.5
	540 - 570	1.55 ± 0.28	1.40 ± 1.38	1.64 ± 0.40
	570 - 600	1.04 ± 0.37	1.76 + 1.92	1.83 ± 0.57
	690 - 720	0.94 ± 0.22	1.62 + 1.52	0.97 ± 0.43
	720 - 750	1.07 \pm 0.38	1.23 ± 1.42	1.15 ± 0.79
78	0 - 30	0.86 ± 0.30	1.16 + 2.23	0.50 <u>+</u> 0.25
	30 - 60 90 - 120	0.61 ± 0.20	1.11 ± 0.93	<0.24
	90 - 120 120 - 150	0.94 ± 0.28 1.12 ± 0.26	1.22 + 1.10	0.88 ± 0.39
	240 - 270	$\frac{1.12 + 0.26}{1.15 + 0.22}$	1.34 ± 1.31 1.10 + 1.08	0.56 ± 0.54
	270 - 300	1.32 + 0.33	1.10 ± 1.08 1.57 ± 0.84	1.05 ± 0.36
	390 - 420	0.80 + 0.25	<0.78	$\begin{array}{c} 1.36 \pm 0.59 \\ 0.89 \pm 0.33 \end{array}$
	420 - 450	0.82 + 0.30	<1.12	0.89 ± 0.31 0.23 ± 0.28
	540 - 570	0.70 ± 0.38	0.98 + 2.01	1.11 + 0.37
	570 - 600	0.76 + 0.22	1.66 + 1.42	0.95 + 0.54
	690 - 705	0.19 + 0.15	0.85 + 1.17	<0.16
4	840 - 870	0.41 ± 0.31	0.79 + 1.68	0.77 + 0.33
	870 - 900	<0.15	<0.77	<0.27
	1020 - 1050	1.17 ± 0.26	1.39 ± 1.34	1.74 ± 0.50
79	0 - 30	1.33 ± 0.37	2.34 <u>+</u> 2.18	1.19 + 0.55
	30 - 60	<0.24	<1.04	1.14 ± 0.51
	90 - 120	0.98 ± 0.41	1.03 ± 2.85	1.57 ± 0.50
	120 - 150	1.19 ± 0.39	0.93 ± 1.00	0.98 + 0.66 $0.85 + 0.38$
	240 - 270 270 - 300	$\begin{array}{c} 0.62 \pm 0.37 \\ 0.69 \pm 0.29 \end{array}$	1.66 ± 1.76	0.85 ± 0.38
-	390 - 420	1.04 + 0.44	2.32 + 0.84	1.45 + 0.53
	370 420	1.04 + 0.44	2.08 ± 1.86	1.17 ± 0.53
80	0 - 30	$\frac{1.43 + 0.47}{1.000}$	4.12 + 1.26	1.22 + 0.86
	30 - 60 90 - 120	0.95 ± 0.26 1.09 ± 0.34	<0.80	1.45 + 0.52
	90 - 120 120 - 150	1.09 ± 0.34 1.26 ± 0.39	<1.08	1.74 ± 0.87
	240 - 270	<0.15	2.31 + 1.75	1.52 ± 0.80
	270 - 300	0.74 + 0.22	3.08 + 1.82 $1.28 + 0.75$	$\begin{array}{c} 1.07 \pm 0.40 \\ 1.89 \pm 0.48 \end{array}$
	390 - 420	0.91 + 0.33	<0.98	$\frac{1.89 \pm 0.48}{2.03 \pm 0.51}$
	420 - 450	0.77 + 0.38	<1.13	1.22 + 0.48
	540 - 570	0.67 + 0.22	1.98 + 1.21	0.61 + 0.33

TABLE 26 (Continued)

RADIONUCLIDE CONCENTRATIONS IN SPLIT SPOON SOIL SAMPLES COLLECTED FROM BOREHOLES ON THE ARMY RESERVE PROPERTY WELDON SPRING, MISSOURI

Borehole	Depth	Radionuc	lide Concentration	on (pCi/g)
Location	(cm)	Ra-226	U-238	Th-232
82	0 - 30	1.09 + 0.42	1.65 + 2.36	1.12 + 0.46
	30 - 6 0	1.89 + 0.46	6.11 + 2.66	0.70 ± 0.8
	90 - 120	1.07 + 0.33	<2.67	1.32 + 0.76
	120 - 150	3.07 ± 0.51	<1.22	0.74 ± 0.63
	240 - 270	0.34 + 0.42	1.64 + 0.73	0.69 + 0.53
	270 - 300	0.46 + 0.24	<0.65	<0.26
	370 - 420	0.30 + 0.19	0.97 + 1.06	0.68 + 0.40
	420 - 450	<0.18	<1.01	0.78 + 0.6
	540 - 570	0.64 + 0.32	1.17 + 1.86	<0.38
	570 - 600	<0.28	<1.07	<0.37

aRefer to Figure 3. bErrors are 20 based on counting statistics.

APPENDIX A

Major Sampling and Analytical Equipment

The display or description of a specific product is not to be construed as an endorsement of that product or its manufacturer by the authors or their employer.

A. Direct Radiation Measurements

Eberline RAS CAL Portable Scaler/Ratemeter Model PRS-1 (Eberline, Sante Fe, NM)

Eberline PRM-6 Portable Ratemeter (Eberline, Sante Fe, NM)

Eberline Alpha Scintillation Probe Model AC-3-7 (Eberline, Sante Fe, NM)

Eberline Beta-Gamma Pancake Probe Model HP-260 (Eberline, Sante Fe, NM)

Victoreen NaI Gamma Scintillation Probe Model 489-55 (Victoreen, Cleveland, OH)

Reuter-Stokes Pressurized Ionization Chamber Model RSS-111 (Reuter-Stokes, Cleveland, OH)

Ludlum Portable Scaler Model 2200 (Ludlum, Sweetwater, TX)

B. Laboratory Analysis

Automatic low-background Alpha-Beta Counter Model LB5110-2080 (Tennelec, Inc., Oak Ridge, TN)

Ge(Li) Detectors (2)
Model LGCC2220SD, 23% efficiency
(Princeton Gamma-Tech, Princeton, NJ)

Used in conjunction with: Lead Shield, SPG-16 (Applied Physical Technology Smyrna, GA) High-Purity Germanium Detector Model GMX-23195-S, 23% efficiency (EG&G ORTEC, Oak Ridge, TN)

Used in conjunction with: Lead Shield, G-16 (Gamma Products, Inc., Palos Hills, IL)

ND-66/ND-680 System (Nuclear Data, Inc., Schaumburg, IL)

Alpha Spectrometry System
Tennelec Electronics, EG&G ORTEC
Surface barrier detectors
(Tennelec, Inc., EG&G, Oak Ridge, TN)

Radon Emanation System
Counter/Timer, Model 2071
Single Channel Analyzer, Model 2031
High Voltage Power Supply, Model 3102
(Canberra Industries, Meriden, CT)

Tennelec Linear Amplifier
Model TC 202BLR
(Tennelec, Inc., Oak Ridge, TN)

Radon Bubblers and Lucas Cells (Rocky Mountain Scientific Glass Blowing, Co., Aurora, CO)

APPENDIX B MEASUREMENT AND ANALYTICAL PROCEDURES

APPENDIX B

Measurement and Analytical Procedures

Gamma Scintillation Measurement

Walkover surface scans and measurements of gamma exposure rates were performed using Eberline Model PRM-6 portable ratemeters with Victoreen Model 489-55 gamma scintillation probes containing 3.2 cm x 3.8 cm NaI(Tl) scintillation crystals. Count rates were converted to exposure rates (μ R/h) using factors determined by comparing the response of the scintillation detector with that of a Reuter Stokes model RSS-111 pressurized ionization chamber at locations on the Busch Wildlife and Weldon Spring Wildlife areas.

Alpha and Beta-Gamma Measurements

Measurements of total alpha radiation levels were performed using Eberline Model PRS-1 portable scaler/ratemeters with Model AC-3-7 alpha scintillation probes. Measurements of direct beta-gamma radiation levels were performed using Eberline Model PRS-1 portable scaler/ratemeters with Model HP-260 thin-window pancake G-M probes. Count rates (cpm) were converted to disintegration rates (dpm/100 cm²) by dividing the net rate by the 4π efficiency and correcting for active area of the detector. The effective window area is 59 cm^2 for the ZnS detectors and 15 cm^2 for the G-M detectors. The average background count rate was 40 cpm for the G-M probes and approximately 2 cpm for the ZnS alpha probes.

Beta and gamma dose rates were calculated individually and the results summed for a combined beta-gamma dose rate. Beta dose rates were calculated by applying the conversion factor of 1400 cpm/mrad/h to the net beta count rate.

Borehole Logging

Borehole gamma radiation measurements were performed using a Victoreen Model 489-55 gamma scintillation probe connected to a Ludlum Model 2200 portable scaler. The scintillation probe was shielded by a 1.25 cm thick lead shield with four 2.5 cm x 7 mm holes evenly spaced around the region of the scintillation crystal. The probe was lowered into each hole using a tripod

holder with a small winch. Measurements were performed at 30 cm intervals in all holes. The logging data were used to identify regions of possible residues and guide the selection of subsurface soil sampling locations.

Soil and Sediment Sample Analysis

Gamma Spectrometry

Soil and sediment samples were dried, mixed, and a portion placed in a 0.5 L Marinelli beaker. The quantity placed in each beaker was chosen to reproduce the calibrated counting geometry and ranged from 600 to 800 g of soil. Net soil weights were determined and the samples counted using intrinsic germanium and Ge(Li) detectors coupled to a Nuclear Data Model ND-680 pulse height analyzer system. Background and Compton stripping, peak search, peak identification, and concentration calculations were performed using the computer capabilities inherent in the analyzer system. Energy peaks used for determination of radionuclides of concern were:

Ra-226 - 0.609 MeV from Bi-214 (secular equilibrium assumed)

Th-232 - 0.911 MeV from Ac-228 (secular equilibrium assumed)

U-238 - 0.094 MeV from Th-234 (secular equilibrium assumed)

or 1.001 MeV from Pa-234 (secular equilibrium assumed)

Alpha Spectrometry: Th-230

Thorium was separated by a process of high temperature fusion, acid dissolution, precipitation, redissolution, and solvent extraction. The thorium was then precipitated with cerium fluoride onto counting discs. Surface barrier detectors coupled to a Nuclear Data Model ND 680 pulse height analyzer enabled identification of the characteristic Th-230 energy peak (4.68 MeV) and activity determination.

Water Sample Analysis

Water samples were rough-filtered through Whatman No. 2 filter paper.

Remaining suspended solids were removed by subsequent filtration through

0.45 µm membrane filters. The filtrate was acidified by addition of 10 ml of concentrated nitric acid. A known volume of each sample was evaporated to dryness and counted for gross alpha and gross beta using a Tennelec Model IB-5110 low-background proportional counter.

Analysis for Ra-226 and Ra-228 was performed using the standard technique EPA 600/4-80-032.

Uranium and thorium isotopic analyses were performed by taking aliquots of liquid, then acidifying and evaporating to dryness. The residue was dissolved by pyrosulfate fusion and precipitated with barium sulfate. The barium sulfate precipitate was redissolved and the uranium and thorium separated by liquid - liquid extraction. The uranium and thorium were then precipitated with a cerium fluoride carrier and counted using surface barrier detectors (ORTEC), alpha spectrometers (Tennelec), and an ND-66 Multichannel Analyzer (Nuclear Data).

Removable Contamination Measurements

Smear measurements were performed on numbered filter paper disks, 47 mm in diameter. Each smear was sealed in a labeled envelope with the location and other pertinent information recorded. A low-background alpha-beta counting system was used to count individual smears.

Errors and Detection Limits

The uncertainties associated with the analytical data, presented in the tables of this report, represent the 95% (20) confidence levels based only on counting statistics. Other sources of error associated with the sampling and analyses introduce an additional uncertainty of \pm 6 to 10% in the results.

Calibration and Quality Assurance

Laboratory and field survey procedures are documented in manuals developed specifically for the Oak Ridge Associated Universities Radiological Site Assessment Program.

With the exception of the measurements conducted with portable gamma scintillation survey meters, instruments were calibrated with NBS-traceable standards. The calibration procedures for the portable gamma instruments are performed by comparison with an NBS calibrated pressurized ionization chamber.

Quality control procedures on all instruments included daily background and check-source measurements to confirm equipment operation within acceptable statistical fluctuations. The ORAU laboratory participates in the EPA and EML Quality Assurance Programs.

APPENDIX C

SUMMARY OF RADIATION GUIDELINES
APPLICABLE TO VICINITY PROPERTIES AT THE WELDON SPRING CHEMICAL PLANT SITE

U.S. DEPARTMENT OF ENERGY GUIDELINES FOR RESIDUAL RADIOACTIVITY AT FORMERLY UTILIZED SITES REMEDIAL ACTION PROGRAM AND REMOTE SURPLUS FACILITIES MANAGEMENT PROGRAM SITES

(Rev. 1, July 1985)

A. INTRODUCTION

This document presents U.S. Department of Energy (DOE) radiological protection guidelines for cleanup of residual radioactive materials and management of the resulting wastes and residues. It is applicable to sites identified by the Formerly Utilized Sites Remedial Action Program (FUSRAP) and remote sites identified by the Surplus Facilities Management Program (SFMP).* The topics covered are basic dose limits, guidelines and authorized limits for allowable levels of residual radioactivity, and requirements for control of the radioactive wastes and residues.

Protocols for identification, characterization, and designation of FUSRAP sites for remedial action; for implementation of the remedial action; and for certification of a FUSRAP site for release for unrestricted use are given in a separate document (U.S. Dept. Energy 1984). More detailed information on applications of the guidelines presented herein, including procedures for deriving site-specific guidelines for allowable levels of residual radio-activity from basic dose limits, is contained in a supplementary document-referred to herein as the "supplement" (U.S. Dept. Energy 1985).

"Residual radioactivity" includes: (1) residual concentrations of radionuclides in soil material, (2) concentrations of airborne radon decay products, (3) external gamma radiation level, and (4) surface contamination. A "basic dose limit" is a prescribed standard from which limits for quantities that can be monitored and controlled are derived; it is specified in terms of the effective dose equivalent as defined by the International Commission on Radiological Protection (ICRP 1977, 1978). Basic dose limits are used explicitly for deriving guidelines for residual concentrations of radionuclides in soil material, except for thorium and radium. Guidelines for

^{*}A remote SFMP site is one that is excess to DOE programmatic needs and is located outside a major operating DOE research and development or production area.

The term "soil material" refers to all material below grade level after remedial action is completed.

residual concentrations of thorium and radium and for the other three quantities (airborne radon decay products, external gamma radiation level, and surface contamination) are based on existing radiological protection standards (U.S. Environ. Prot. Agency 1983; U.S. Nucl. Reg. Comm. 1982). These standards are assumed to be consistent with basic dose limits within the uncertainty of derivations of levels of residual radioactivity from basic limits.

A "guideline" for residual radioactivity is a level of residual radioactivity that is acceptable if the use of the site is to be unrestricted. Guidelines for residual radioactivity presented herein are of two kinds: (1) generic, site-independent guidelines taken from existing radiation protection standards, and (2) site-specific guidelines derived from basic dose limits using site-specific models and data. Generic guideline values are presented in this document. Procedures and data for deriving site-specific guideline values are given in the supplement.

An "authorized limit" is a level of residual radioactivity that must not be exceeded if the remedial action is to be considered completed. Under normal circumstances, expected to occur at most sites, authorized limits for residual radioactivity are set equal to guideline values. Exceptional conditions for which authorized limits might differ from guideline values are specified in Sections D and F. A site may be released for unrestricted use only if the residual radioactivity does not exceed guideline values at the time remedial action is completed. Restrictions and controls on use of the site must be established and enforced if the residual radioactivity exceeds guideline values. The applicable controls and restrictions are specified in Section E.

DOE policy requires that all exposures to radiation be limited to levels that are as low as reasonably achievable (ALARA). Implementation of ALARA policy is specified as procedures to be applied after authorized limits have been set. For sites to be released for unrestricted use, the intent is to reduce residual radioactivity to levels that are as far below authorized limits as reasonable considering technical, economic, and social factors. At sites where the residual radioactivity is not reduced to levels that permit release for unrestricted use, ALARA policy is implemented by establishing controls to reduce exposure to levels that are as low as is reasonably achievable. Procedures for implementing ALARA policy are described in the supplement. ALARA policies, procedures, and actions must be documented and filed as a permanent record upon completion of remedial action at a site.

B. BASIC DOSE LIMITS

The basic limit for the annual radiation dose received by an individual member of the general public is 500 mrem/yr for a period of exposure not to exceed 5 years and an average of 100 mrem/yr over a lifetime. The committed effective dose equivalent, as defined in ICRP Publication 26 (ICRP 1977) and calculated by dosimetry models described in ICRP Publication 30 (ICRP 1978), shall be used for determining the dose.

C. GUIDELINES FOR RESIDUAL RADIOACTIVITY

C.1 Residual Radionuclides in Soil Material

Residual concentrations of radionuclides in soil material shall be specified as above-background concentrations averaged over an area of $100~\text{m}^2$. If the concentration in any area is found to exceed the average by a factor greater than 3, guidelines for local concentrations shall also be applicable. These "hot spot" guidelines depend on the extent of the elevated local concentrations and are given in the supplement.

The generic guidelines for residual concentrations of Th-232, Th-230, Ra-228, and Ra-226 are:

- 5 pCi/g, averaged over the first 15 cm of soil below the surface
- 15 pCi/g, averaged over 15-cm-thick layers of soil more than 15 cm below the surface

These guidelines take into account ingrowth of Ra-226 from Th-230 and of Ra-228 from Th-232, and assume secular equilibrium. If either Th-230 and Ra-226 or Th-232 and Ra-228 are both present, not in secular equilibrium, the guidelines apply to the higher concentration. If other mixtures of radio-nuclides occur, the concentrations of individual radionuclides shall be reduced so that the dose for the mixtures will not exceed the basic dose limit. Explicit formulas for calculating residual concentration guidelines for mixtures are given in the supplement.

The guidelines for residual concentrations in soil material of all other radionuclides shall be derived from basic dose limits by means of an environmental pathway analysis using site-specific data. Procedures for deriving these guidelines are given in the supplement.

C.2 Airborne Radon Decay Products

Generic guidelines for concentrations of airborne radon decay products shall apply to existing occupied or habitable structures on private property that are intended for unrestricted use; structures that will be demolished or buried are excluded. The applicable generic guideline (40 CFR 192) is: In any occupied or habitable building, the objective of remedial action shall be, and reasonable effort shall be made to achieve, an annual average (or equivalent) radon decay product concentration (including background) not to exceed 0.02 WL.* In any case, the radon decay product concentration (including background) shall not exceed 0.03 WL. Remedial actions are not required in order to comply with this guideline when there is reasonable assurance that residual radioactive materials are not the cause.

C.3 External Gamma Radiation

The average level of gamma radiation inside a building or habitable structure on a site to be released for unrestricted use shall not exceed the background level by more than 20 $\mu R/h$.

 $^{^{\}pm}A$ working level (WL) is any combination of short-lived radon decay products in one liter of air that will result in the ultimate emission of 1.3 \times 10^5 MeV of potential alpha energy.

C.4 Surface Contamination

The following generic guidelines, adapted from standards of the U.S. Nuclear Regulatory Commission (1982), are applicable only to existing structures and equipment that will not be demolished and buried. They apply to both interior and exterior surfaces. If a building is demolished and buried, the guidelines in Section C.1 are applicable to the resulting contamination in the ground.

	Allowable Total Residual Surface Contamination (dpm/100 cm ²); 1			
Radionuclides† ²	Average ^{†3} , † ⁴ Maximum ^{†4} , † ⁵		Removable;4,†6	
Transuranics, Ra-226, Ra-228, Th-230, Th-228, Pa-231, Ac-227, I-125, I-129	100	300	20	
Th-Natural, Th-232, Sr-90, Ra-223, Ra-224, U-232, I-126, I-131, I-133	1,000	3,000	200	
U-Natural, U-235, U-238, and associated decay products	5,000α	15,000α	1,000α	
Beta-gamma emitters (radionuclides with decay modes other than alpha emission or spontaneous fission) except Sr-90 and others noted above	5,000β-γ	15,000β-γ	1,000β-γ	

[†] As used in this table, dpm (disintegrations per minute) means the rate of emission by radioactive material as determined by correcting the counts per minute measured by an appropriate detector for background, efficiency, and geometric factors associated with the instrumentation.

^{†&}lt;sup>2</sup> Where surface contamination by both alpha- and beta-gamma-emitting radionuclides exists, the limits established for alpha- and beta-gamma-emitting radionuclides should apply independently.

 $[\]dot{\tau}^3$ Measurements of average contamination should not be averaged over an area of more than 1 m². For objects of less surface area, the average should be derived for each such object.

^{†4} The average and maximum dose rates associated with surface contamination resulting from beta-gamma emitters should not exceed 0.2 mrad/h and 1.0 mrad/h, respectively, at 1 cm.

^{†5} The maximum contamination level applies to an area of not more than 100 cm².

The amount of removable radioactive material per 100 cm² of surface area should be determined by wiping that area with dry filter or soft absorbent paper, applying moderate pressure, and measuring the amount of radioactive material on the wipe with an appropriate instrument of known efficiency. When removable contamination on objects of surface area less than 100 cm² is determined, the activity per unit area should be based on the actual area and the entire surface should be wiped. The numbers in this column are maximum amounts.

D. AUTHORIZED LIMITS FOR RESIDUAL RADIOACTIVITY

The remedial action shall not be considered complete unless the residual radioactivity is below authorized limits. Authorized limits shall be set equal to guidelines for residual radioactivity unless: (1) exceptions specified in Section F of this document are applicable, in which case an authorized limit may be set above the guideline value for the specific location or condition to which the exception is applicable; or (2) on the basis of site-specific data not used in establishing the guidelines, it can be clearly established that limits below the guidelines are reasonable and can be achieved without appreciable increase in cost of the remedial action. Authorized limits that differ from guidelines must be justified and established on a site-specific basis, with documentation that must be filed as a permanent record upon completion of remedial action at a site. Authorized limits differing from the guidelines must be approved by the Director, Oak Ridge Technical Services Division, for FUSRAP and by the Director, Richland Surplus Facilities Management Program Office, for remote SFMP--with concurrence by the Director of Remedial Action Projects for both programs.

E. CONTROL OF RESIDUAL RADIOACTIVITY AT FUSRAP AND REMOTE SFMP SITES

Residual radioactivity above the guidelines at FUSRAP and remote SFMP sites must be managed in accordance with applicable DOE Orders. The DOE Order 5480.1A requires compliance with applicable federal, state, and local environmental protection standards.

The operational and control requirements specified in the following DOE Orders shall apply to interim storage, interim management, and long-term management.

- a. 5440.1B, Implementation of the National Environmental Policy Act
- b. 5480.1A, Environmental Protection, Safety, and Health Protection Program for DOE Operations
- c. 5480.2, Hazardous and Radioactive Mixed Waste Management
- d. 5480.4, Environmental Protection, Safety, and Health Protection Standards
- e. 5482.1A, Environmental, Safety, and Health Appraisal Program
- f. 5483.1, Occupational Safety and Health Program for Government-Owned Contractor-Operated Facilities
- g. 5484.1, Environmental Protection, Safety, and Health Protection Information Reporting Requirements
- h. 5484.2, Unusual Occurrence Reporting System
- i. 5820.2, Radioactive Waste Management

E.1 Interim Storage

a. Control and stabilization features shall be designed to ensure, to the extent reasonably achievable, an effective life of 50 years and, in any case, at least 25 years.

- b. Above-background Rn-222 concentrations in the atmosphere above facility surfaces or openings shall not exceed: (1) 100 pCi/L at any given point, (2) an annual average concentration of 30 pCi/L over the facility site, and (3) an annual average concentration of 3 pCi/L at or above any location outside the facility site (DOE Order 5480.1A, Attachment XI-1).
- c. Concentrations of radionuclides in the groundwater or quantities of residual radioactive materials shall not exceed existing federal, state, or local standards.
- d. Access to a site shall be controlled and misuse of onsite material contaminated by residual radioactivity shall be prevented through appropriate administrative controls and physical barriers—active and passive controls as described by the U.S. Environmental Protection Agency (1983—p. 595). These control features should be designed to ensure, to the extent reasonable, an effective life of at least 25 years. The federal government shall have title to the property.

E.2 Interim Management

- a. A site may be released under interim management when the residual radioactivity exceeds guideline values if the residual radioactivity is in inaccessible locations and would be unreasonably costly to remove, provided that administrative controls are established to ensure that no member of the public shall receive a radiation dose exceeding the basic dose limit.
- b. The administrative controls, as approved by DOE, shall include but not be limited to periodic monitoring, appropriate shielding, physical barriers to prevent access, and appropriate radiological safety measures during maintenance, renovation, demolition, or other activities that might disturb the residual radioactivity or cause it to migrate.
- c. The owner of the site or appropriate federal, state, or local authorities shall be responsible for enforcing the administrative controls.

E.3 Long-Term Management

Uranium, Thorium, and Their Decay Products

- a. Control and stabilization features shall be designed to ensure, to the extent reasonably achievable, an effective life of 1,000 years and, in any case, at least 200 years.
- b. Control and stabilization features shall be designed to ensure that Rn-222 emanation to the atmosphere from the waste shall not: (1) exceed an annual average release rate of 20 pCi/m²/s, and (2) increase the annual average Rn-222 concentration at or above any location outside the boundary of the contaminated area by more than 0.5 pCi/L. Field verification of emanation rates is not required.

- c. Prior to placement of any potentially biodegradable contaminated wastes in a long-term management facility, such wastes shall be properly conditioned to ensure that (1) the generation and escape of biogenic gases will not cause the requirement in paragraph b of this section (E.3) to be exceeded, and (2) biodegradation within the facility will not result in premature structural failure in violation of the requirements in paragraph a of this section (E.3).
- d. Groundwater shall be protected in accordance with 40 CFR 192.20(a)(2) and 192.20(a)(3), as applicable to FUSRAP and remote SFMP sites.
- e. Access to a site should be controlled and misuse of onsite material contaminated by residual radioactivity should be prevented through appropriate administrative controls and physical barriers—active and passive controls as described by the U.S. Environmental Protection Agency (1983—p. 595). These controls should be designed to be effective to the extent reasonable for at least 200 years. The federal government shall have title to the property.

Other Radionuclides

f. Long-term management of other radionuclides shall be in accordance with Chapters 2, 3, and 5 of DOE Order 5820.2, as applicable.

F. EXCEPTIONS

Exceptions to the requirement that authorized limits be set equal to the guidelines may be made on the basis of an analysis of site-specific aspects of a designated site that were not taken into account in deriving the guidelines. Exceptions require approvals as stated in Section D. Specific situations that warrant exceptions are:

- a. Where remedial actions would pose a clear and present risk of injury to workers or members of the general public, notwithstanding reasonable measures to avoid or reduce risk.
- b. Where remedial actions—even after all reasonable mitigative measures have been taken—would produce environmental harm that is clearly excessive compared to the health benefits to persons living on or near affected sites, now or in the future. A clear excess of environmental harm is harm that is long-term, manifest, and grossly disproportionate to health benefits that may reasonably be anticipated.
- c. Where the cost of remedial actions for contaminated soil is unreasonably high relative to long-term benefits and where the residual radioactive materials do not pose a clear present or future risk after taking necessary control measures. The likelihood that buildings will be erected or that people will spend long periods of time at such a site should be considered in evaluating this risk. Remedial actions will generally not

be necessary where only minor quantities of residual radioactive materials are involved or where residual radioactive materials occur in an inaccessible location at which sitespecific factors limit their hazard and from the they are costly or difficult to remove. Examples are residual radioactive materials under hard-surface public roads and sidewalks, around public sewer lines, or in fence-post foundations. In order to invoke this exception, a site-specific analysis must be provided to establish that it would not cause an individual to receive a radiation dose in excess of the basic dose limits stated in Section B, and a statement specifying the residual radioactivity must be included in the appropriate state and local records.

- d. Where the cost of cleanup of a contaminated building is clearly unreasonably high relative to the benefits. Factors that shall be included in this judgment are the anticipated period of occupancy, the incremental radiation level that would be effected by remedial action, the residual useful lifetime of the building, the potential for future construction at the site, and the applicability of remedial actions that would be less costly than removal of the residual radioactive materials. A statement specifying the residual radioactivity must be included in the appropriate state and local records.
- e. Where there is no feasible remedial action.

G. SOURCES

Long-Term Management

Limit or Guideline	Source
Basic Dose Limits	
Dosimetry Model and Dose Limits	International Commission on Radiological Protection (1977, 1978)
Generic Guidelines for Res	idual Radioactivity
Residual Concentrations of Radium and Thorium in Soil Material	40 CFR 192
Airborne Radon Decay Products	40 CFR 192
External Gamma Radiation	40 CFR 192
Surface Contamination	Adapted from U.S. Nuclear Regulatory Commission (1982)
Control of Radioactive Wast	es and Residues
Interim Storage	DOE Order 5480.1A

DOE Order 5480.1A; 40 CFR 192

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URANIUM SITE SPECIFIC SOIL GUIDELINES FOR THE U.S. ARMY RESERVE PROPERTY

developed by

T.L. Gilbert

Environmental Research Division Argonne National Laboratory

Uranium site-specific soil guidelines where derived for the U.S. Army Reserve property adjacent to the Weldon Spring Chemical Plant. These derived soil guidelines (Gilbert 1986) are based on the requirement that the average annual committed effective dose equivalent to an individual should not exceed a basic dose limit of 100 mrem/yr above background (U.S. Dept. Energy 1985). Procedures specified in the DOE manual for implementing residual radioactivity guidelines were used to carry out the derivation (Gilbert et al. 1985).

It was calculated that the basic dose limit would not be exceeded if the average concentration of U-238 within contaminated zones does not exceed 60 pCi/g. This guideline value applies to the activity concentration of U-238 when no principal radionuclides other than U-238 and U-234 are present in above-background concentrations and when U-238 and U-234 are both present in secular equilibrium. In those locations, where other radionuclides are present in above-background concentrations, the mixture sum formula (Gilbert et al. 1985---Section 5.4.2) would be used to determine if guidelines are met.

For small, isolated areas on this property, soil concentrations of U-238 in excess of the above guideline levels are allowable, provided it can be small established that the basic dose limit is not exceeded.

The foregoing guidelines are based solely on the requirement that the estimated potential dose to an onsite resident should not exceed 100 mrem/yr. An additional DOE requirement is that the radiation dose to individuals and groups should be kept as low as reasonably achievable (ALARA), economic and social factors being taken into account (Gilbert et al. 1985—Section 6).

REFERENCES

- Gilbert, T.L. 1986. Derivation of Site-Specific Soil Guidelines for Weldon Spring Vicinity Properties I. U.S. Army Reserve Property. Prepared by Environmental Research Division, Argonne National Laboratory, for Division of Facility and Site Decommissioning, U.S. Department of Energy. Draft Report, January 1986.
- Gilbert, T.L., K.F. Eckerman, W.R. Hansen, J.W. Healy, W.E. Kennedy, Jr., B.A. Napier, and J.K. Soldat. 1985. A Manual for Implementing Residual Radioactivity Guidelines: A Supplement to U.S. Department of Energy Guidelines for Residual Radioactivity at Formerly Utilized Sites Remedial Action Program and Remote Surplus Facilities Management Program Sites. Prepared by Argonne National Laboratory, Oak Ridge National Laboratory, Los Alamos National Laboratory, and Battelle Pacific Northwest Laboratory for the U.S. Department of Energy. September 1985.
- U.S. Department of Energy. 1985. Guidelines for Residual Radioactivity at FUSRAP and Remote SFMP Sites (Revision 1, July 1985). Attachment to memo from J.E. Baublitz (Director, Division of Remedial Action Projects) to E.L. Keller (OR) and C.E. Miller (RL). (Reproduced in Appendix B of preceding reference.)

IRA-300-303

cc: D. R. Lewis

R. A. Nelson

A. J. Stewart

UNITED STATES ENVIRONMENTAL PROTECTION AGENCY 3589-87-I. EPA-010

•

RECEIVED SONTRACT 358

NOV 5 287

Mr. Rodney R. Nelson
U.S. Department of Energy
Weldon Spring Site Remedial
Action Project/Office
Route 2, Highway 94, South
St. Charles, Missouri 63303

Dear Mr. Neison:

We have reviewed the Department of Energy's (DOE) proposals for the following four interim response actions:

- Electric Power and Pole Removal,
- Overhead Piping/Asbestos Removal,
- * Cleanup of Vicinity Property No. 7 on the Army Reserve Area, and
- Disposal of Containerized Chemicals.

Our comments on these proposals were sent to you earlier. You were also provided comments by the Missouri Department of Natural Resources (MDNR). No comments from the public were directed to the Environmental Protection Agency (EPA) and according to our records, there has been no public comment directed to MDNR or DOE.

We are in agreement these actions should proceed to ensure worker safety and reduce the further release of contaminants from this site. The EPA hereby approves these actions under the condition that the comments earlier provided by EPA and MDNR are adequately addressed. The MDNR has notified me they also concur with these actions. Please provide copies of any summary reports for these actions to EPA and MDNR.

We also received copies of the following four interim response actions:

- Dismantling of Building 401,
- Dismantling of Building 409,
- Removal of PCB Transformers, and
- Debris Consolidation.

Mac.a.

We will provide any comments on these within the agreed upon 21-day comment period. We are most pleased to see that activities are underway to stabilize the site and reduce contaminant release.

Sincerely yours,

Morris Kay Regional Administrator

cc: Dr. Fred Brunner, MDNR

bc: Robert Morby

Dan Shiel

Rowena Michaels

Ron Ritter

NOV 1 0 1987

Ms. B. Katherine Biggs
United States Environmental
Protection Agency
Region VII
726 Minnesota Avenue
Kansas City, Kansas 66101

Dear Ms. Biggs:

USEPA COMMENTS ON INTERIM RESPONSE ACTIONS (IRA'S)

Enclosed is our response to the comments contained in your letter of October 8, 1987, regarding the following interim response actions:

- 1. Electric Power Line and Pole Removal
- 2. Overhead Piping/Asbestos Removal
- 3. Army Reserve Area Vicinity Property No. 7
- 4. Disposal of Containerized Chemicals

We anticipate that this will adequately resolve the issues raised. We intend to proceed with action on these items in accordance with the enclosure.

If you have any questions, please give me a call.

Sincerely,

ORIGINAL SIGNED BY:

R. R. Nelson Project Manager Weldon Spring Site Remedial Action Project

Enclosure:
As stated

cc: D. Bedan, MDNR

E. Brown, FLW
w/enclosure

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RESPONSES TO USEPA REVIEW COMMENTS ON IRA PACKAGES

Cleanup of Vicinity Property No. 7, Army Reserve Area

Comment: The proposal to cleanup this vicinity property calls for excavating the contaminated area to a depth of six (6) inches or where the radium concentration is below 15 pCi/g and then backfilling with clean material. The EPA suggests that while its criteria for residual radium in soil is satisfied by this approach, that we consider removing additional soil to reduce the concentration to below 5 pCi/g. As the area is small, little additional excavation would be required.

Response: Subsequent to the preparation of the IRA package for Army Reserve Vicinity Property #7, the Deparment of Army requested that the area not be backfilled upon completion of the cleanup as proposed by the DOE. The DOE will leave the excavation area open and apply the suface criteria of 5 pCi/g to this particular vicinity property.

Disposal of Containerized Chemicals

Comment: It is suggested that the specifications for this work might be strengthened by adding waste characterization procedures into Section 2.0 (Scope) of the document. The procedures are those which may be required under 40 CFR 260-268, or others required by the permit held by the Treatment, Storage and/or Disposal Facility.

Response: Procedures required in 40 CFR 260-268 will be referenced in Section 2.0 of the Request for Proposal. In addition, it will be emphasized that all waste characterization procedures which are required by the successful bidder's treatment, storage and/or disposal facilities permit must be satisfied. It will be required that these procedures (if applicable) be presented in the subcontractor's work plan.

Comment: EPA recommends that the specific subcontractor qualifications and experience in handling known and unknown potentially hazardous wastes be defined in the document.

Response: We are in agreement with the EPA that the

Request for Proposal should contain subcontractor

qualifications and experience clauses. The

appropriate clauses will be added to the document.

Comment: The EPA recommends that the specification require

the successful bidder to identify the specific waste disposal facilities which will accept the containerized chemical waste, in the work plan

phase.

Response: The specification will be modified to include

provision for certification by the subcontractor

that the waste disposal facilities meet the requirements when hazardous wastes are involved. The land disposal ban provision of RCRA will also be addressed as part of the subcontractor's work

plan.

General

Comment: The EPA review states that there is one(1)

deficiency common to the four proposals and that is that plans for onsite handling and storage of radioactive contaminated materials should be

developed.

Response: Plans for onsite handling and storage of

radioactive contaminated materials are currently

being finalized and will be provided under

separate cover.

NOV 10 1987

Mr. David E. Bedan
Missouri Department of
Natural Resources
Post Office Box 176
Jefferson City, Missouri 65102

Dear Mr. Bedan:

MISSOURI DNR COMMENTS ON INTERIM RESPONSE ACTIONS (IRA'S)

Enclosed is our response to the comments contained in Dr. Frederick A. Brunner's letter of October 26, 1987, regarding the following interim response actions:

- 1. Electric Power Line and Pole Removal
- 2. Overhead Piping/Asbestos Removal
- 3. Army Reserve Area Vicinity Property No. 7
- 4. Disposal of Containerized Chemicals

We anticipate that this will adequately resolve the issues raised and we intend to proceed with these actions in accordance with the enclosure.

If you have any questions, please give me a call.

Sincerely,
CRIGINAL SIGNED BY:
R. R. NELSON

R. R. Nelson Project Manager Weldon Spring Site Remedial Action Project

Enclosure: As stated

cc: B. K. Biggs, USEPA
E. Brown, FLW
W/enclosure

FILE NUMBER:

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RESPONSES TO MDNR COMMENTS ON INITIAL FOUR (4) IRA PROPOSAL PACKAGES

I. Removal of Overhead Piping and Asbestos Removal

Comment: The DNR states that Missouri has adopted the Federal Clean Air Act standards for asbestos handling and has been delegated responsibility for implementing these standards and that asbestos and piping removal activities are subject to both the Missouri Air Conservation Law and the Missouri Solid Waste Management Law. DNR recommends that we maintain close contact with the Air Pollution Control Program to insure compliance with these standards.

Response: DNR Air Pollution Control Program office will be kept apprised of plans for asbestos removal work at the Weldon Spring Site. The WSSRAP will comply with requirements for disposal of asbestos and other demolition wastes in accordance with the Missouri Solid Waste Management Act.

Comment: The DNR has determined that the overhead piping and asbestos should be handled as a "special waste".

Response: We are proceeding to include the Special Waste Disposal Request form in the Request for Proposal for this work.

Comment: The DNR states that onsite handling of asbestos and other demolition waste may also be subject to Missouri Solid Waste Management Law requirements and requests that we furnish information on the size, design, location of the staging area and the amounts and methods of handling for the materials to be handled in the materials staging area.

Response: Information on handling and staging of the materials will be furnished to the DNR prior to issuing requests for proposals for this work.

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II. Disposal of Containerized Chemicals

Comment: Define specific levels at which the containerized wastes are considered radioactive.

Response: WSSRAP is developing concentration levels for wastes containing natural uranium for review and acceptance by concerned federal agencies. We will advise the DNR of this determination as it comes available. Until this determination is made we will retain on site containerized chemical materials which contain detectable levels of radioactive materials as determined by our onsite instruments.

Comment: What are removal plans for underground storage tanks on site?

Response: The underground tanks at the WSS have been sampled and found to contain only rainwater with trace amounts of motor fuel. The drainage and removal of the underground tanks is not part of the containerized chemical inventory and removal IRA Scope of Work. They may be removed as part of a subsequent IRA.

Comment: The document appears to be a generic outline for removal of waste. Items such as disposal facilities, transporters, waste characterization procedures, waste treatment procedures, etc. are not detailed.

Response: The IRA documentation represents a request for proposal to be sent to potential removal subcontractors. The responsibility for developing a detailed work plan addressing such items as disposal facilities, transporters, waste characterization procedures, waste treatment procedures, etc. rests with the successful bidder. A requirement of the subcontract specification is the development of the subcontractor's work plan which must be approved by the WSSRAP before the work may begin. The DNR will be provided a copy of the subcontractor work plan when it becomes available for review. The WSSRAP office requests that the State provide a timely review (14 calendar days) to avoid delaying the subcontractor's

efforts. The State will be given 2-weeks notice of the interval at which time the work plan will be available for review.

III. Remedial Action on Army Vicinity Property

Comment: DNR states that the interim measure suggested is lacking in detail and should contain information on:

- How the removal is to be conducted?
- How and where excavated material will be contained?
- Health and safety plans for the work.
- Why is DOE recommending only remedial action for this vicinity property?
- When does BOE plan to remove the additional contamination in the other six locations?

Response: The technical requirements, i.e. specifications, drawings, special conditions, etc., did not accompany this IRA package for review as they were incomplete at the time of package submittal. Requirements for removal, containment and storage, and health and safety plans will be included in the Request for Proposal for this work.

The very small quantity (less than two cubic yards) of contaminated material can be removed manually and will not require any significant mechanical equipment. The technical specification developed for this work will provide required direction to the subcontractor.

We plan to excavate and place the contaminated soil in 55 gallon drums. The drums will be sealed, placed and stored in a dry, concrete floored building at the Weldon Spring Site awaiting final disposition. Total volume is anticipated to be less than 5 drums.

No unusual hazards are anticipated for this activity. The contract will require workers to abide by the WSSRAP Environmental, Safety and Health Plan.

DOE proposes performing remedial action of Army Vicinity Property No. 7 to remove contamination from an area where the Army has imminent construction plans. Cleanup of other vicinity properties, containing larger volumes of contaminated material, will be recommended based upon urgency of cleanup needs and development of storage facilities on the WSS.

Remediation of the remaining Army Vicinity Properties is tentatively scheduled for the first and second quarters of fiscal 1989.

IV. Power Line/Pole Removal

Response: The plan should address PCB contamination in regard to pole removal if transformers/capacitors containing PCB's were mounted on them.

Comment: The power poles to be removed in the IRA were specifically not associated with transformers containing PCBs. Those poles which have oil-cooled transformers mounted on them will be surveyed and removed at a later date.

IRA-300-304

ALMANTINE & C. ILAH. CLASESP.

3589-87-I-DOE-176



Department of Energy

Oak Ridge Operations
Weldon Spring Site
Remedial Action Project Office
Route 2, Highway 94 South
St. Charles, Missouri 63303

August 25, 1987

Ms. Katherine Biggs
United States Environmental
Protection Agency
Region VII
726 Minnesota Avenue
Kansas City, Kansas 66101



Dear Ms. Biggs:

As we have discussed previously, the Department of the Army has requested the Weldon Spring Site Remedial Action Project to conduct remedial action on several vicinity properties in the Army Reserve training area adjacent to the Weldon Spring Site. These vicinity properties are areas of low level radiological contamination. Since the Army plans to do road construction in the training area beginning this fall, it is necessary that the contamination in the area of this activity be removed.

Enclosed is our plan for action on vicinity property #7 on the Army Reserve area. This is a very small area of contamination, consisting of a total of about 0.7 cubic yard of contaminated soil. Enclosure 01 to this letter is a copy of the report, Radiological Survey U.S. Army Reserve Property, Weldon Spring Site, St. Charles County, Missouri, prepared by Oak Ridge Associated Universities (ORAU) January 1986. This report defines the level and extent of the contamination of the vicinity properties on the Army Reserve area. Enclosure 02 is a copy of the DOE guidelines for residual radioactive material. The version contained in the ORAU report has been superceded. These documents will be applicable to future vicinity properties, so please retain them for reference.

Enclosure 03 is a recent radiological characterization performed by project personnel. A brief Action Description Memorandum containing the plan for action is contained in Attachment 04.

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The DOE will document the action in a memorandum setting forth the rationale for the action. A draft of such memo is Enclosure 05. The Action Description Memorandum will be attached to the memo. We would appreciate your prompt attention to this matter so that we may proceed with the remedial action and allow the Army Reserve to proceed with the construction work.

If there are any questions, please give me a call.

١

Sincerely,

R. R. Nelson

Project Manager Weldon Spring Site

Remedial Action Project

CE-541:Nelson

Enclosures:
As stated

cc: Dave Bedan, MDNR w/enc.
 Emily Brown, FLW wo/enc.

RADIOLOGICAL SURVEY
U.S. ARMY RESERVE PROPERTY
WELDON SPRING SITE
ST. CHARLES COUNTY, MISSOURI

Prepared by

E. J. DEMING

Radiological Site Assessment Program
Manpower Education, Research, and Training
Oak Ridge Associated Universities
Oak Ridge, Tennessee 37831-0117

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Prepared for

U.S. Department of Energy
as part of the
Formerly Utilized Sites - Remedial Action Program

FINAL REPORT

January 1986

This report is based on work performed under contract number DE-AC05-760R00033 with the U.S. Department of Energy.

*Currently with EG&G Idaho, Idaho Falls, Idaho.

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ENCLOSURE 04

DESCRIPTION OF PROPOSED INTERIM REMEDIAL ACTION AT THE WELDON SPRING SITE

As part of its Surplus Facilities Management Program (SFMP), the U.S. Department of Energy (DOE) proposes to decontaminate a small area of contaminated soil on the U.S. Army Reserve Property located to the west of DOE's Weldon Spring site and transport the resultant wastes to the Weldon Spring site for interim storage. This action is being taken at the request of the U.S. Department of the Army, Fort Leonard Wood, to allow the Army to proceed with a road improvement project during the fall of 1987. The activities associated with the remedial action are:

- Removing about 0.4 m³ (0.5 yd³) of soil that is radioactively contaminated above current guidelines from Vicinity Property 7, which is located about 1 m (1 yd) north of Army Road No. 1 and 300 m (330 yd) west of a road intersection (see Figs. 1 and 2). The principal soil contaminants are radium-226 and thorium-230, with maximum concentrations of 215 pCi/g and 53 pCi/g, respectively.
- Placing the soil in appropriate containers (55-gal drums are currently planned) for ease of handling and for controlling the possible spread of radioactive contamination.
- Loading the containers on the back of a pickup truck.
- Transporting the material about 1.0 km (0.5 mi) through the Army Reserve Property to the Weldon Spring site (solely on Army- and DOE-owned land).
- Placing the contaminated material in the raffinate pits area at a controlled location for interim storage.
- Monitoring the environment during remedial actions to ensure compliance with all applicable radiation protection requirements.

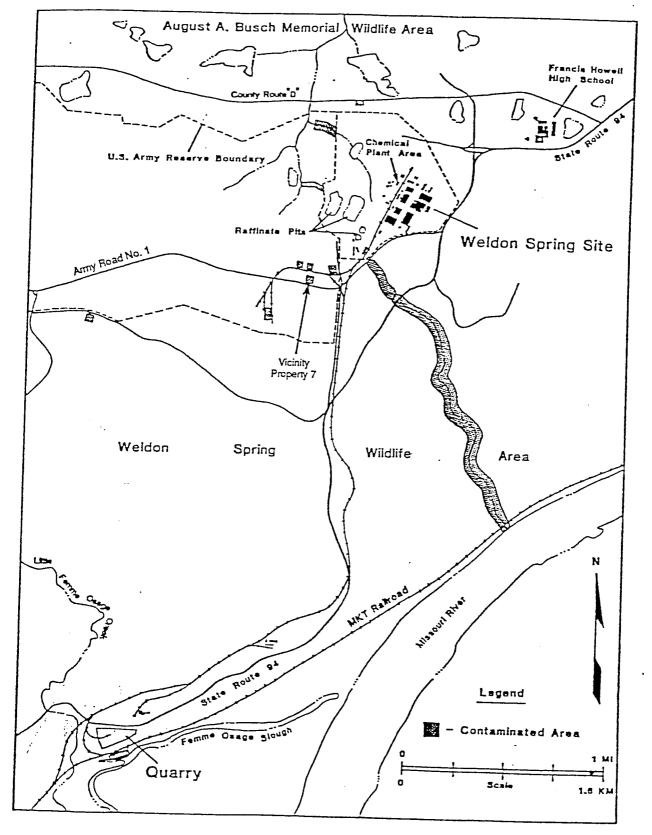


FIGURE 1 Location of Contaminated Vicinity Properties in the Area of the Raffinate Pits, Chemical Plant, and Quarry

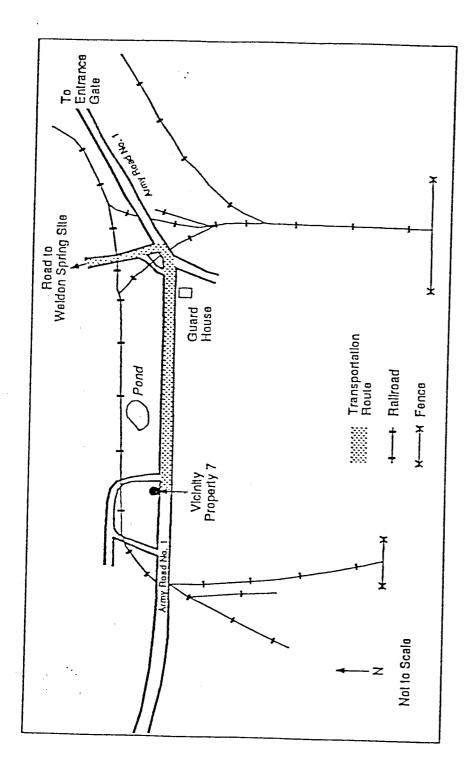


FIGURE 2 Location of Violnity Property 7

INTERIM RESPONSE ACTION (IRA) ADMINISTRATIVE RECORD FILE ARFS FILE # IR-0400

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401		Sampling and A	Analysis Plans
	IR-0	400-401-1.01	SAMPLING OF INSULATION ON INTER-BUILDING OVERHEAD UTILITY PIPES FOR ASBESTOS CONTENT - NOV. 1986
402		Sampling and A	Analysis Data/Chain of Custody Forms
	IR-0	400-402-1.01	REMOVAL OF OVERHEAD YARD PIPING & ASBESTOS REMOVAL
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IRA-400-401

Introduction

This plan describes the sampling effort to be conducted in November of 1986 for determining the amount of asbestos in insulation on pipes on overhead utilities at the WSCP.

Objective

This sampling effort will be performed to determine in a statistically valid manner whether insulation on individual sections of the inter-building overhead utility pipes at WSSRAP contain asbestos. The data collected in this sampling effort will be used by the Engineering Department in the preparation of bid specifications for the removal of the overhead inter-building utilities. Pipes in or on buildings will not be sampled during this sampling effort. The samples will be collected by the Environmental Safety and Health Department (ES&H).

Sampling Schedule

Samples will be collected on Monday and Tuesday November 17 and 18, 1986. Sampling may continue into the remainder of the week if necessary. Should differences in insulation composition be noted visually during the planned sampling effort, additional samples will be collected as necessary to further define the composition of the insulation.

Sampling Locations

Sampling locations have been selected to obtain samples from each

1

FILE	NUMBER:	

size and type of pipe. The rationale for selection of the sampling locations is based on a review of existing WSCP construction blue prints and a site survey of the WSCP facilities conducted on November 7, 1986. Construction drawings numbered 7500-5 to 7500-28 prepared by Blaw-Knox Company in 1955 and 1956 shown locations of the pipes and the pipe support numbers for these overhead utilities. Samples will be collected from each size and type of pipe (steam, ethylene glycol, raffinate, etc.) and type of insulation. Insulation will be sampled in straight sections and at joints in the pipe. The sampling locations are described in Table 1. Figure 1 shows the planned sampling locations. These sampling locations have been selected to maximize the number of samples which can be collected at each point where the manlift will be positioned.

Equipment

The following equipment will be available for use during sample collection.

Item

Knife
Cork Borer
Hack Saw
Screwdriver
Hammer
Sample Bottles
Ziploc Bags
Water Sprayer
Bucket of Water
Duct Tape
Camera
Sample Labels
Field Data Sheets
Clipboard
Paint

Usage

Cutting Insulation and Sheathing
Cutting Insulation
Cutting Sheathing
Sheathing Removal
Bending Metal Sheathing
Sample Shipping
Sample Shipping
Tool Cleaning and Dust Control
Tool Cleaning
Dust Control
Documentation of Sampling Location

Flagging Tape Tape Measure Respirator with HEPA Filters Personal Protection Disposable Coveralls Rubber Gloves Rubber Boots Safety Glasses Hard Hat Eve Wash Radiation Monitoring Equip. Carrying Case Manlift

Documentation of Sampling Location Documentation of Sampling Location Personal Protection Transportation of Equipment Access to Sampling Location

Training

Personnel involved in collection of asbestos samples shall receive training in the health and safety aspects of asbestos. also be trained in the operation of any special equipment (i.e. hydraulic manlifts).

Sample Collection

Prior to collecting samples, a safety meeting will be conducted to familiarize personnel with equipment and procedures. A respirator is recommended, but is not required as sampling will not generate airborne concentrations above allowable levels. Once the sampling location is verified and access is established, samples will be collected as follows:

- Start a field data sheet for the sample to be collected. Record the date, time, location, pipe type if known, pipe size, and any other relevant information on the field data sheet. An example field data sheet is presented in Figure 2.
- Select two corresponding pre-labeled sample bottles for the sample to be collected at that location. Example labels are shown in Figure 3.
- 3. Open the sheathing using a knife, hack saw or screwdriver as

appropriate.

- 4. If the sheathing is not metal, place pieces of the removed sheathing in the sample containers.
- 5. Loosen a piece of insulation using a cork borer, knife, or other appropriate tool.
- 6. Split the piece of insulation in half. Ensure that both halves contain a complete cross section of the insulation.
- 7. Place the pieces of insulation in the sample bottles.
- 7. Close the sample bottle.
- 8. Complete and sign the field data sheet.

Radiation Monitoring

A member of the Health Physics Group shall measure the radiation levels of the individual samples prior to shipment of the samples off site. Sample collection personnel shall use radiation measurement equipment calibrated by the Health Physics Group to monitor levels in the work area during sample collection.

Chain of Custody

4.

All samples will be kept in the custody of the sample collection personnel from the time of collection shipment. A chain of custody log sheet which lists each sample will be completed and signed by the sample collection personnel. The original of the chain of custody log sheet will be shipped with the samples to the laboratory. A copy of the chain of custody will be retained at WSSRAP by the ES&H Department. An example chain of custody form is presented in Figure

Sample Analysis

Samples will be analyzed by polarized light microscopy. The selected laboratory shall be a participant in the EPA laboratory evaluation program.

Quality Control

Sample collection will be observed on a random basis by a representative of the WSSRAP ES&H Department. One in twenty of the split samples will be submitted for analysis. The remainder of the splits will be archived at the WSSRAP facility for verification by a separate laboratory if necessary. Once QC checks of the lab data are complete, the archived samples will be used in training WSSRAP personnel and subcontractors for future asbestos removal activities.

Reporting of Visual Observations

The sample collection crew will discuss with the Engineering

Department observations regarding visible differences and/or

similarities of the insulation at the various sampling locations.

Reporting of Analytical Results

The laboratory will report the concentration of asbestos in the sample. The lab data, field data and drawings showing the sampling locations will be incorporated in a final report by ES&H for submittal to the Engineering Department. This report will be completed within 7 days of receipt of the lab data.

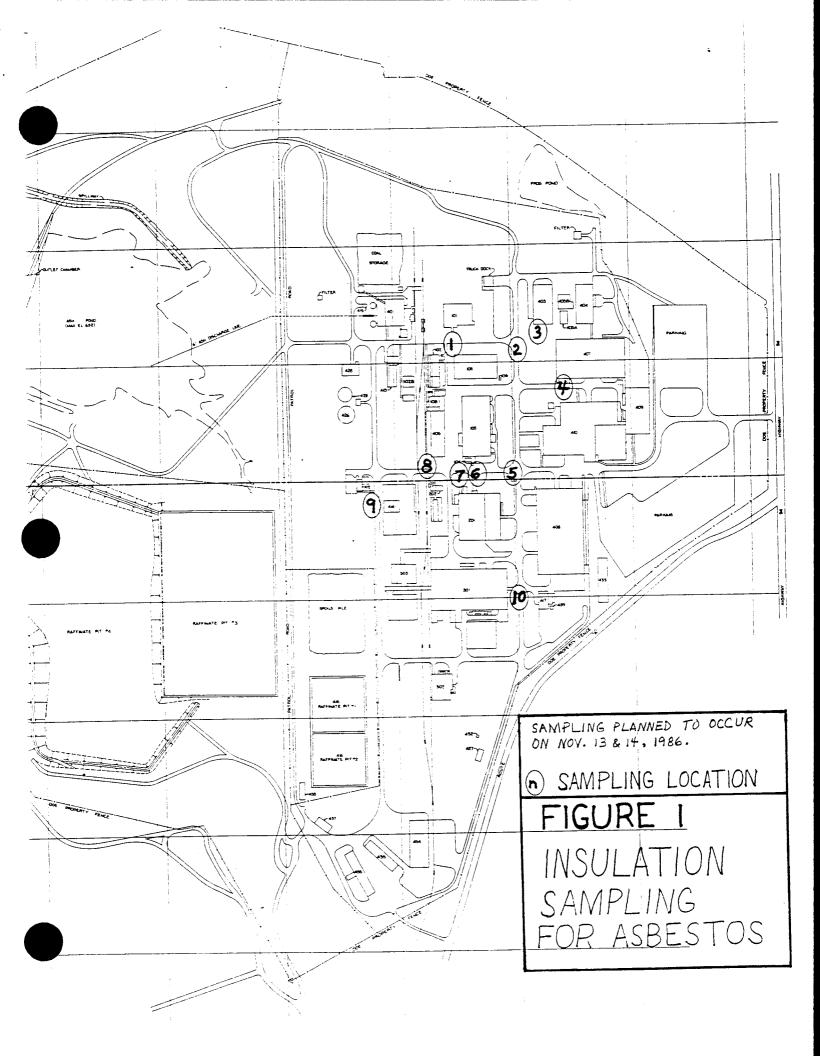


FIGURE 2

EXAMPLE FIELD DATA SHEET

WELDON SPRING SITE REMEDIAL ACTION PROJECT (WSSRAP) Route 2, Highway 94, St. Charles, Missouri 63303

Phone (314) 441-8086 Telex (314) 447-0803

BULK MATERIAL SAMPLING FOR ASBESTOS - FIELD DATA

SAMPLE NUMBER:	Date:
SAMPLING LOCATION	
Building Number	
Floor	
Room	·
Construction Drawing	
Other	
MATERIAL TYPE	
Fipe Insulation Fipe Diameter:	inches
	Steam, Ethylene Glycol,Raffinate, Dther
Ceiling Floor Wall Boiler Insulation	
COVER OVER MATERIAL	
Fiber, Metal, Flaster, None	, Other
DESCRIPTION OF MATERIAL	
Color:	
Hardness: Fibrous (Fri	able), Granular (Soft), or Hard (Concrete-Like)
Thickness:in	ches
COMMENTS	
Endintion Lovel	
Radiation Level Background Only Above Background	cpm Instrument:
Sample Collection Personnel:	Company:

FIGURE 3

EXAMPLE SAMPLE LABEL

Weldon Spring Site Remedial Action Project (WSSRAF)

MK-FERGUSON COMPANY (PMC)

Rt. 2, Hwy 94, St. Charles, MO 63303

Phone (314) 441-8086

Sample Number IN-2000- 1

Location: WSCP (Pipe Support No.:_____)
Matrix: Pipe Insulation - Above Ground Utilities
Collected By: Kirk Meyer
Date: November _____ 1786

Table 1 -- Asbestos Sampling Locations, WSSRAP, Nov. 17-18, 1986

							, ,
	Location Number	Support Number	Description (Drawing No)	Insul Quant.	ation Samples To Pipe Type	Be Colle Sheath S	126
	1	508	NW Corner of Building 105 (7500-16)	1 2 2 1	Steam Steam Ethylene Glycol Unknown Elbow (Steam) Elbow (Steam)	Fiber Fiber Fiber Fiber Fiber	18 inches 10 6 ? 18 10
	2	522	NE Corner of Building 105 (7500-17)	1 2 1 1 1	Steam Ethylene Glycol Unknown Unknown Unknown Unknown	Fiber Fiber Fiber Fiber Fiber	8 6 4 2 2 1
	3	71	NW Corner of Building 403 (7500-23)	1 2 1 1 1	Steam Ethylene Glycol D. S. Steam Unknown Unknown	Fiber Fiber Fiber Fiber Fiber	6 2 2 8 2 2
	4	731	From Building 407 to 410 (7500-28)	g 2 1	Ethylene Glycol Unknown	Fiber Fiber	4 2
•	5	40	NE Corner of Building 201 (7500-19)	1 1 1 1	Steam Raffinate Elbow Raffinate Elbow	Fiber Fiber Fiber Metal Metal	18 3 3 3 3
	6	562	North Center Building 201 (7500-24)		Steam Raffinate Raffinate	Fiber Fiber Metal	6 3 3
	7	564	NE Corner of Building 201 (7500-24)		Raffinate	Unknown	3
	8	Ground	SW Corner of Building 406 (N. A.)		Steam?	Fiber	1
	9	764	West of Building 406 (7500-25)	1 1 1	Raffinate Elbow Raffinate Elbow	Fiber Fiber Metal Metal	3 3 3 3
)	10	84	East Center Building 303 (7500-20)	1 1 1	Steam Steam Elbow (Steam)	Fiber Fiber Fiber	10 3 1
				4.0			

WELDON SPRING SITE REMEDIAL ACTION PROJECT (WSSRAP)
Route 2, Highway 94, St. Charles, Missouri 63303
Fhone (314) 441-8086 Telex (314) 447-0803

ENVIRONMENTAL PROTECTION GROUP -- DOCUMENT REVIEW

1	PLAN FOR SAMPLING OF INTER-BUILDING OVERHEAD ASBESTOS CONTENT - NOVEMB	OTILITY THE TO
	NOVEMBER 13, 1986 Author: KIR	
For As Ret	Ansmittal: - Approval For Review - Your Information Review and Requested Review and turn of Loan to Us Returned for	Correct File
Table Reviewed Title Others	N. Ed. HODSON YUSUF G. NOCHANI	Date 101. 12 1986 Nov. 12 1986
Environmental E S & H Manage Guality Manage Engineering Ma	anager	NOV. 12. 1986 NOV. 12 1986

IRA-400-402



Department of Energy

Oak Ridge Operations
Weldon Spring Site
Remedial Action Project Office
Route 2, Highway 94 South
St. Charles, Missouri 63303

September 03, 1987

Ms. Katherine Biggs
United States Environmental
Protection Agency, Region VII
726 Minnesota Avenue
Kansas City, Kansas 66101

Dear Ms. Biggs:

Enclosed is the information regarding the removal of overhead yard piping and asbestos insulation from the Weldon Spring Site, which we agreed upon in our telephone conference on July 24, 1987.

The site contains insulated overhead piping in the yard areas. The insulation is deteriorating and falling to the ground. The material is subject to blowing about and presents a potential hazard to exposed personnel. We propose to remove the material.

Radiologically contaminated material will be retained on site in a secure condition. Non-radiologically contaminated materials will be disposed of offsite in approved landfills.

The entire task involves removing about 33,000 linear feet of pipe, 13,00 of which is asbestos insulated, and 500 pipe supports. The work will be accomplished by a subcontractor to MK-Ferguson Company, our Project Management Contractor.

The enclosed index lists five (5) attachments including sampling data and plans and specifications for the work. If you have any questions, please contact Jim Coyne of PEER Consultants, our support services contractor at (314) 441-8472.

Sincerely,

(d. 61. 1 () so

R. R. Nelson Project Manager Weldon Spring Site Remedial Action Project

Enclosure:
As stated

cc: Dave Bedan, MDNR

FILE NUMBER: ____

INTERIM MEASURE

OVERHEAD PIPING/ASBESTOS REMOVAL

SUMMARY

This task consists of removing all abandoned outside overhead yard piping and associated insulation and pipe supports at the Weldon Spring Site. Approximately 33,000 lf of pipe are involved, 13,000 lf of which is asbestos insulated pipe. About 500 structural steel pipe support columns and a number of bridging structures are also included in this task.

The asbestos containing material (ACM) is deteriorating and falling to the ground. It is accessible to the elements and subject to blowing about the site. Its presence and state of deterioration constitute a significant threat to the health and safety of on-site workers and could under dry, high wind conditions pose a threat to off-site personnel.

We propose this work to place the ACM under control and mitigate worker and public exposure to the hazard posed by the deteriorating pipe insulation.

The plan calls for removal, survey, segregation, and protection of materials in on-site staging areas. Radiologically contaminated ACM will be retained on-site. Non-radiologically contaminated materials will be released for disposal off-site in approved ascessos landfills. We expect much of the utility pipe, structural supports, and asbestos to be releasable for disposal off-site.

3589-SC-WP017

TECHNICAL SPECIFICATIONS

WSSRAP OVERHEAD PIPING REMOVAL .



LIST OF TECHNICAL SPECIFICATIONS 3589-SC-WP017

Spe	cification Section No.	<u>Title</u>
1)	Section 02051	Demolition Of Overhead Piping And Supports
2)	Section 02080	Asbestos Removal On Overhead Piping

WSSRA PROJECT REVIEWS AND APPROVALS

			•
Subject:	WSSRA PROJECT - CP	•	5121-C:SP-S-01-0192-02
•	Specification Section 02051		. (DOCUMENT NO.)
•	Demolition of Overhead Piping	•	
	and Supports		•
			•
	•		
•			·
		•	•
Prepared: Reviewed	•		Signature Date Signature 5-19-87 Date 5-19-87
Engi	, MKE: Design Engineer ineering & Design Manager ef Engineer, ESC Division		Klu Rippord 5-19-87 Thilip & Shuener 5-19-87 L. Sada 22 May 87
– PMC	Engineering Manager		QA Compliance <u>Pdr 5-20-87</u> Skoturi 6-17-87

O.A. REVIEW
ENTRY NO. _\(\nu/\text{P}\)

A MORRISSH-KNUDSEN COMPANY

SECTION 02051

DEMOLITION OF OVERHEAD PIPING AND SUP

PART 1 - GENERAL

1.1 SCOPE

- A. This Specification Section describes the the demolition and placing into storage of facilities:
 - 1. All outdoor above ground piping, pipe pipe bridges at a uranium feed mate plant, including steel piping (insul sulated), steel support structures and extending up to building lines. Also moval of some piping which is polyvinyl pine and some timber pole, pipe support was primarily for supplying plant uti water. Piping to raffinate pits and st generally insulated with asbestos contains.
 - 2. Asbestos containing insulation on three haust flue ducts to chimney stacks on steam plant, Building No. 401. Recof flue duct.
 - 3. Asbestos-insulated pipe section with strame support (approximately 50 feet loground beside road east of Building Notopsoil within 5 feet of each end of pipe
 - 4. Sections of asbestos insulated piping ground about 60 feet north of Raffinate proximately 79 joints at 30 ft. long equations strip topsoil within 5 feet of stacked pip
- B. Locations of demolition and material stag: shown on project drawings.
- C. Refer to Special Conditions, Articles SC-9 a restrictions applicable to working in areas w

Document No. 5121-C:SP-S-01-0192-02 Re-issued for Construction-Revision 2 Demolition of OH Piping and Supports 02051 - 1 D. Refer to Section 02080 for requirements applicable to removal, handling, and clean-up of asbestos-containing materials.

1.2 WORK NOT INCLUDED

Foundations and underground structures are excluded.

1.3 SUBMITTALS

A. Dismantling and Asbestos Containment Plan:

The Subcontractor shall submit a detailed plan of the work schedule and procedures to be used in dismantling the overhead piping and supports. The plan shall include details concerning sequencing, method for asbestos containment during pipe cutting, and removal methods for support removal, air monitoring, transportation and storage. The Subcontractor shall meet with the Contractor prior to beginning work to discuss and obtain acceptance of the Plan.

PART 2 - PRODUCTS

(Not Used)

PART 3 - EXECUTION

3.1 DEMOLITION

- A. Pollution Controls:
 - 1. Water sprinkling and temporary enclosures shall be used to limit the amount of airborne dust and dirt to the lowest practical level. Other methods of limiting the amount of airborne dust and dirt must be approved by Contractor.
 - 2. Water shall not be used if it is likely to create hazardous or objectionable conditions such as ice, flooding, or pollution. An approved water-based biodegradable wetting agent (surfactant) may be used to reduce the quantity of water required.

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3454S 051587 2

- B. Cutting and Capping:
 - Pipes shall be cut as close as practicable to the ground level and the faces of buildings.
 - 2. Pipe supports shall be disconnected from their concrete footings where possible; otherwise they shall be cut off as close to the footing as possible. Footings shall be left in place.
- C. Holes due to removal of timber poles shall be backfilled.
- D. Areas disturbed by demolition operations, including backfilling and soil stripping, shall be seeded as specified in Section 02930.
- 3.2 DISPOSAL OF DEMOLITION DEBRIS
 - A. All materials shall be transported to and stacked at the on-site material staging area.
 - B. Within the staging area, materials shall be stacked in separate areas as directed by the Contractor. Materials shall be segregated into stacks of similar shapes.
 - C. Before placement in the material staging area, all materials shall be cut or broken up in sizes not greater than 10 feet in any dimension and not greater than 27 cubic feet in volume.
 - D. Unless otherwise approved by the Contractor, the demolition debris shall be stored in layers or piles not to exceed 10 feet in height.
 - E. All piping is considered empty except for short runs which may contain small amounts of liquids. For piping known to have contained ethylene glycol or piping suspected to contain 'unknown' liquids, Subcontractor shall provide drum containers for collecting liquids without liquids contacting ground surface.

END OF SECTION 02051

Document No. 5121-C:SP-S-01-0192-02
Re-issued for Construction-Revision 2
Demolition of OH Piping and Supports
02051 - 3

A MORRISON KNUDSEN COMPANY

WSSRA PROJECT REVIEWS AND APPROVALS

Subject:	WSSRA PROJECT - CP	5121-C:SP-S-01-0122-03
332 , 323	Specification Section 02080	(DOCUMENT NO.)
	Asbestos Removal on Overhead	
	Piping	
		·
•		<u>Signature</u> <u>Date</u>
Prepared:		JEnes 5-29-87
Reviewed	la	D. Report 5-29-57
	•	
		·
Approved	, MKE:	1100
	Design Engineer	W. Kennon 5-29-87
	ineering & Design Manager	4 July 1 Junearer 5-29-87
- Chi	ef Engineer, ESC Division	6. 5. Smith 29. V. 87
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		QA Compliance Pda 5-39-8
PM0	C Engineering Manager	DX Surs 6/1/10/
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		O.A. REVIEW
		EMTRY NO. WA
		INTERNAL PROPERTY.

SECTION 02080

ASBESTOS REMOVAL ON OVERHEAD PIPING

PART 1 - GENERAL

1.1 SCOPE

- A. This Specification Section describes the requirements for removal and disposal of asbestos-containing material on outdoor aboveground utility and process piping and valves between buildings at the Chemical Plant Site. All steam pipe insulation contains asbestos. Ethylene glycol piping insulation contains asbestos at bends only. Pipe is to be removed with insulation intact where possible. Clean-up of some asbestos contaminated soil is included.
- B. The Subcontractor shall furnish all labor, supervision, materials, services, insurance, and equipment necessary to carry out the removal operation in accordance with the EPA and OSHA regulations and the State of Missouri Department of Natural Resources.
- C. The Subcontractor shall be responsible for disposal of all asbestos containing materials at a designated location shown on the Subcontract Drawings.
- D. The requirements specified in this Section are supplemental to the requirements specified in General Provisions, General Conditions and Special Conditions.

1.2 CODES AND REGULATIONS

- A. All work on this Subcontract shall comply with applicable codes and regulations including, but not limited to, the following:
 - U.S. Department of Labor, Occupational Safety and Health Administration ("OSHA"): 29CFR1926.58
 - 2. Environmental Protection Agency ("EPA"):

National Emmissions Standard for Hazardous Air Pollutants (NESHAPS) 40CFR61, Subparts A and M

B. Posting Regulations: Copies of applicable OSHA and EPA regulations shall be obtained by the Subcontractor. One copy of each shall be posted at the job site. The EPA notification of asbestos removal, including the "Plan of procedures to be employed for compliance with EPA regulations" shall also be posted at the job site.

1.3 DEFINITIONS

- A. Asbestos Controlled Area: The asbestos contaminated work area or any other area with either an airborne asbestos level equal to or above 0.01 fiber/cc air or visible uncontained deposits of asbestos-contaminated material.
- B. Asbestos-Containing Material: Materials either containing more than 1% by weight of asbestos, excluding substrate materials, or contaminated with asbestos to a degree that handling the materials may reasonably be expected to give rise to exposure to airborne asbestos fibers.
- C. Contaminated Materials: Materials having radioactive contamination levels greater than specified in the applicable US Environmental Protection Agency Standards. These standards are presented in the Federal Register, January 5, 1983, Section 192.12: "Standards for Remedial Actions at Inactive Uranium Processing Sites". The contamination levels will be determined by the Contractor.
- D. HEPA Filter: A High Efficiency Particulate Absolute filter capable of trapping and retaining 99.97% of particles with aerodynamic equivalent diameters greater than or equal to 0.3 micrometer.
- E. Clearance Air Samples: Air samples taken following asbestos removal and visual inspection by the Contractor or his authorized representative to verify completion of work specified under this Subcontract.
- F. Amended Water: Water to which a surfactant has been added.
- G. Encapsulant: A liquid material which can be applied to asbestos material which controls the possible release of asbestos fibers from the material by penetrating into the material and by binding its components together.

At least 20 days prior to beginning work on the asbestoscontaining materials and 20 days prior to completion of Work on this Subcontract, the Subcontractor shall submit written notifications to the State of Missouri Department of Natural Resources. The specific information shall be provided as detailed in 40CFR61.146, Subpart M. The notice shall be sent to:

> Department of Natural Resources Division of Environmental Quality P. O. Box 176 Jefferson City, Missouri 65102

Attn: Bob Craig (314-751-8328)

1.5 EXPOSURE MONITORING

- A. The airborne asbestos exposure limit outside of asbestos controlled areas is 0.01 fiber/cc air. Where control of airborne asbestos levels to below 0.01 fiber/cc air is impractical, temporary asbestos control areas shall be established. Asbestos removal work areas with higher concentrations, or where concentrations might reasonably be expected to exceed the allowable limit, must be separated from other work areas by air-tight barriers. Airtight barriers for piping shall generally consist of glove bag or glove box.
- B. All air monitoring shall be performed under the supervision of the Subcontractor's industrial hygienist, who must be certified by the American Board of Industrial Hygiene.
- C. Documentation of each air sample shall be as specified by the Contractor and shall include at least the date and time, sample number, exact sampling location, name of individual performing sampling, sampling rate, sampling volume, analytical method, analytical results and limits of quantification and detection per National Institute of Occupational Safety and Health (NIOSE) analytical methods.
- D. Analysis of air samples shall be as specified in 29 CFR 1926.58 Appendix A or equivalent. Samples shall be analyzed onsite. Test results of samples taken outside of asbestos control areas and one sample per work shift from each asbestos control area shall be reported to the Contractor within 4 hours of collection. Samples exceeding the asbestos exposure limit shall be reported to the Con-

Document No. 5121-C:SP-S-01-0122-03
Re-Issued for Construction-Revision 3
Asbestos Removal on Overhead Piping
02080 - 3

tractor immediately following the analysis. The analysis quality assurance program shall include recounting of a portion of the samples by a laboratory accredited by the American Industrial Hygiene Association (AIHA) for asbestos analysis and currently judged proficient in asbestos counting by successful participation in the NIOSH Proficiency Analytical Testing (PAT) program. The portion of samples recounted shall include the following: (1) all clearance air samples, (2) all samples taken outside asbestos control areas exceeding the exposure limit, and (3) 10% of the remaining samples taken from both inside and outside asbestos control areas not exceeding the exposure limit. All quality assurance sample results shall be reported to the Contractor within two weeks of collection.

- E. Preliminary air sampling shall be sufficient to establish the perimeters of asbestos control areas to the satisfaction of the Contractor.
- F. Air samples shall be taken in the breathing zones of workers removing asbestos in sufficient numbers to permit selection of appropriate respirators and Time Weighted Average (TWA) Exposures. At a minimum, for TWA exposures, one personal sample on the worker with the highest probable exposure is required in each asbestos control area, per work shift.

PART 2 - PRODUCTS

2.1 MATERIALS

- A. Wetting Materials: For wetting asbestos-containing materials prior to disturbance, amended water or a removal encapsulant shall be used. Wetting materials shall be tested at the job site on the material to be removed to ensure that the use of the material will result in the thorough wetting of the asbestos containing material and retardation of fiber release during disturbance of the material. Testing of wetting materials shall be done to the satisfaction of the Contractor.
- B. Polyethylene Sheet: A single polyethylene film in the largest sheet size possible to minimize seams, 6.0 mils thick, clear or opaque.

- c. Polyethylene Tubing: Seamless tubing of a single polyethylene film, 6 mil (minimum) thickness, and of a diameter sufficient to enclose one or more insulated pipes.
- D. Disposal Bags: Six mil (minimum) thickess leak-tight polyethylene bags shall be provided by the Subcontractor. Bags shall be labeled in accordance with EPA and OSHA regulations.
- E. Glove Bags: Six mil (minimum) thickness polyethylene glove bags.

PART 3 - EXECUTION

3.1 ESTIMATED AMOUNTS OF ASBESTOS-CONTAINING MATERIALS TO BE REMOVED

The estimated amount of friable asbestos is approximately one hundred eighty-five (185) cubic yards on 11600 feet of pipe and 6 cubic yards on flue ducts. This asbestos is in piping insulation material located from ground level to a height of approximately 50 feet on various exterior pipe racks between the various structures, to the limits of the building faces, including vertical risers.

- 3.2 REMOVAL METHOD OPTIONS AND EXECUTION
 - A. Removal of Asbestos-Contaminated Soil:
 - 1. Soil shall be saturated with amended water or removal encapsulant. The surface of the soil shall be kept continuously wet throughout its removal and decontamination. Areas designated on plan drawings, where asbestos insulation has fallen to ground, shall have the top soil stripped.
 - The limits of stripping under piperacks and pipeways shall include that area directly under the pipeway bounded by the outermost dimensions of the included pipe support plus 5 feet beyond. The top 3 inches of soils shall be removed and placed in disposal bags along with any visible asbestos. The limits of stripping for other designated items is the area immediately below and within 5 feet horizontally.

- 3. Personnel protective equipment shall be worn during initial soil decontamination activities. Traffic shall not be permitted onto the fresh soil surface. After the entire first layer of soil is removed, coveralls and boot covers shall be completely changed. Decontaminate excavation equipment by washing with water. Collect all water and filter as specified in 3.2.G. The remaining asbestos removal and decontamination work shall be carried out as specified.
- B. Piping shall be removed in units or sections with insulation in place. The maximum allowable unit size is ten (10) linear feet.
- C. Completely seal straight runs of piping between supporting columns with polyethylene sheeting or tubing. Duct tape shall be used to seal edges and spiral stripe wrap surface as reinforcement. Areas where asbestos is to be removed from the piping prior to dismantling and cutting, or areas where insulation and pipe are to be cut simultaneously, shall be enclosed in a glove bag or glove box. Glove boxes, if designed for multiple use, shall be decontaminated prior to being moved.
- D. Isolation of Work Area: Areas where asbestos-containing material is to be removed, disengaged from pipe racks, or disturbed in any way shall be isolated and enclosed with glove bags or glove boxes. If glove boxes are to be used, construction details of the temporary enclosures shall be submitted as part of the Asbestos Removal Plan. The glove box shall be maintained under minimum negative operating pressure of at least 0.02 inch of water.
- E. When glove boxes are inadequate, areas where asbestos-containing material is to be removed, in any way, shall be isolated and enclosed in temporary enclosures maintained under a minimum negative pressure of 0.02 inch of water. The minimum enclosure shall have a solid floor; walls shall be framed with wood or metal. The interior surface of the walls, ceiling and floor shall be covered with two layers of polyethylene sheeting such that the inner layer can be removed without disturbing the integrity of the outer layer. The sequence of applying polyethylene sheeting to the inner surfaces, overlaps, sealing procedure and decontamination facilities shall be as described in 29 CFR 1926.58 Appendix F. Decontamination facilities shall be of adequate number and size to allow effective decontamination of both workers and equipment.

- F. Maintenance of the Enclosure System: Damage and defects in the enclosure system shall be repaired immediately upon discovery. If at any time during the project after removal work has started, visible debris or elevated levels of asbestos in air are observed outside the enclosure or if damage occurs to barriers, work shall immediately stop, the Contractor shall be notified, and repairs shall be made to the enclosure and debris cleaned up as specified in Paragraph A above.
- G. Wastewater Collection: All wastewater shall be collected and filtered through a dual filtration system. Provide a first filter that removes all fibers 20 microns or longer and a second filter that removes all fibers 5 microns or longer. The filtered wastewater shall be collected, transported, and discharged at a location near Raffinate pit No. 4 where directed by Contractor.
- H. Where piping is cut at building lines, the exposed cut ends of asbestos containing insulation which will remain shall be sealed in polethylene secured by duct tape.

3.3 COMMENCEMENT OF REMOVAL WORK

- A. Removal work shall not commence until:
 - 1. After first enclosure system (glove bag and glove box), which shall be typical for all subsequent work, has been constructed (with glove box, also, brought under negative pressure), Subcontractor shall notify Contractor that the enclosure is available for inspection. Subcontractor shall receive written approval from Contractor to proceed with asbestos removal for glove bag and, separately, for glove box.
 - 2. All submissions, notifications, postings and permits have been provided and approved by the Contractor.
 - 3. All equipment is on hand.
 - 4. All worker training and certification is completed.

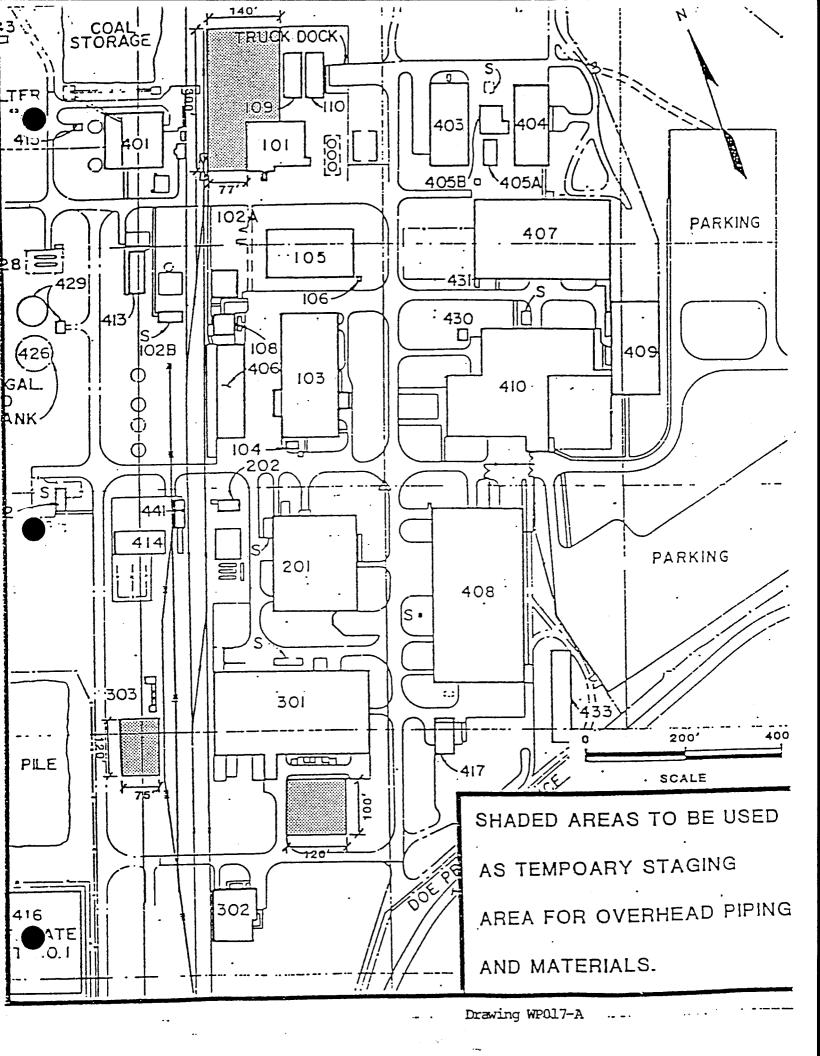
3.4 TRANSPORT AND STORAGE

A. All materials removed shall be transported to and placed in the on-site Material Staging Area shown on the Subcontract Drawings. The area shall be distinguished by installing flags, or by boundary markers approved by the

Contractor, to prevent disturbance of the area by other subcontractors. The flags or markers shall be installed at each corner of the area and at a spacing not exceeding 40 feet.

- B. The Subcontractor shall comply with EPA air emissions regulations, 40CFR61.152 and 40CFR61.153 during all transport and handling operations.
- C. All bags containing asbestos shall be labeled in accordance with EPA and OSHA regulations.
- D. All bags shall be placed in "fiberdrums" and covered with polyethylene film at material staging area.
- 3.5 CLEANING WORK AREAS AFTER ASBESTOS REMOVAL
 - A. Prior to removal of enclosures, work areas shall be cleaned to assure there are no asbestos fibers which could become airborne.
 - B. Construction equipment shall be cleaned and free of asbestos fibers before removal from site-controlled work areas.
 - C. The Subcontractor shall notify the Contractor when work is complete for a final visual inspection.
 - D. After Contractor's final visual inspection, the enclosure shall be removed and all asbestos containing waste disposed of per the above.

END OF SECTION 02080



Weldon Spring Site Remedial Action Project (WSSRAP)
Route 2, Highway 94 South, St. Charles, Missouri 63303
Phone (314) 441-8086 Telecopy (314) 447-0803

FINAL

Asbestos Content of Pipe Insulation On Outdoor Overhead Utilities At The WSCP

Prepared for:

U.S. Department of Energy Weldon Spring, Missouri

Prepared by:

Weldon Spring Site Remedial Action Project
MK-Ferguson Company -- Project Management Contractor
Weldon Spring, Missouri

ES&H Kirk Meyer February 3, 1987

MKF Project No. 3589

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1.0 INTRODUCTION

This report presents the results of a survey to determine the type of insulation on overhead utilities at the Weldon Spring Chemical Plant (WSCP). In particular, this survey determined which pipes were insulated with asbestos containing material as defined by the EPA (greater than 1 percent asbestos by volume). Results of this survey are intended to be used in the preparation of an engineering plan for removal of the overhead utilities from the site. This report does not include a determination of the actual quantity of asbestos insulation, rather only the types of pipes which have asbestos insulation are presented. Asbestos quantities can be calculated by applying the information contained herein in conjunction with the piping inventory.

Sampling for this survey was conducted by the PMC during November 1986. Samples were analyzed by Particle Data Laboratories of Elmhurst, Illinois. A total of 42 samples were collected from six different types of insulation.

Section 2.0 of this report describes the overhead piping and its insulation. Section 3.0 presents the sampling methods. Test results are presented in Section 4.0. The laboratory report is presented in Appendix A. Copies of the field data sheets are in Appendix B. Results of the radiological analysis of these samples are in Appendix C.

2.0 DESCRIPTION OF OVERHEAD PIPING AND SAMPLING LOCATIONS

Overhead utilities pipelines are positioned between buildings at the WSCP as shown in Figure 2-1. Insulated and non-insulated process pipelines present had the following contents or functions:

Insulated Pipes

Steam
Ethylene Glycol
Raffinate
Miscellaneous Process Lines (near Building 403)

Non-Insulated Pipes

Fuel Gas Plant Air Instrument Air

Locations of overhead utilities, pipes and supports can be found on construction drawings numbered 7500-5 to 7500-28 prepared by Blaw-Knox Company in 1955 and 1956. Bechtel National Incorporated drawings 201-SK22-C-01, 201-SK22-C-02 and 201-SK22-P-01 also show locations of insulated pipes by type of pipe.

The nearest support numbers and construction drawing numbers reported for each sampling location in Appendix B refers to the Blaw-Knox Drawings.

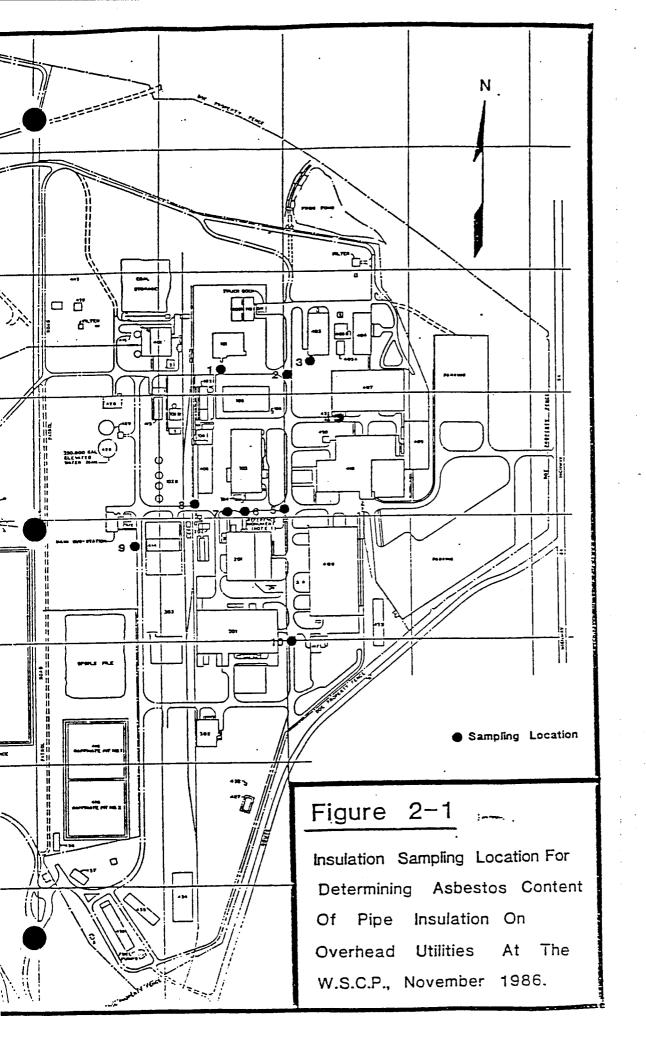


Figure 2-1 Insulation Sampling Locations for Determining Asbestos Content of Pipe Insulation on Overhead Utilities at the WSCP, November 1986.

3.0 SAMPLING AND ANALYSIS METHODS

Samples were collected from insulation on each type of insulated pipe and from each visibly different type of insulation. Samples were collected from the 42 locations listed in Table 3-1 and shown in Figure 2-1. The sampling locations were selected in such a way that insulation on all types of pipe (steam, raffinate, ethylene glycol and process) and all sizes of pipes of each type could be sampled. Ten sampling zones were chosen. All pipes accessible from a manlift in each of these zones were sampled.

Field data sheets are presented in Appendix B of this report. The pipe diameters shown on the field sheets are estimated for most locations. The thicknesses of insulation are actual measured values for all locations. Support and construction drawing numbers refers to drawings prepared by Blaw-Knox around 1956.

Those samples suspected of containing asbestos were submitted for analysis. One sample from each insulation type (fiberglass and cork) not suspected to contain asbestos was also analyzed.

Each sample was a complete cross-section of the pipe insulation including sheathing. These samples were collected with a hole saw operated with a battery-powered drill. Samples were also collected by hand when the insulation could be easily broken loose without tools. The hole saw and the sampling technicians' gloves were thoroughly decontaminated with a clean water spray between samples. The samples were transferred directly from the hole saw into a Zip-Lok plastic bag and placed in pre-labeled jars.

The samples were divided in half in the laboratory. A clean, dust-free table was used during this operation. Only one sample was open at any one time. The forceps, gloves, and the table top were cleaned with a clean water spray and were dried with clean paper towels between samples.

After all samples were split, planchettes were filled for a radiological survey of the insulation. These planchettes were analyzed for alpha radiation prior to shipping the samples off site. The radiological analysis data is presented in Appendix C.

The samples were analyzed for asbestos content by Particle Data Laboratories of Elmhurst, Illinois. This laboratory is a successful participant in the EPA laboratory evaluation program for asbestos sample analysis. The presence of asbestos in the samples was determined by a polarized light microscopy analytical method and the concentration of asbestos in the sample was determined by volume. Appendix A describes the analysis method in more detail.

Table 3-1 Insulation Sampling Locations for Determining Asbestos Content of Pipe Insulation on Overhead Utilities at the WSCP, November 1986.

SAMPLING	NEAREST	ZONE		ULATION SAMPLES CO	DLLECIED	
ZONE NUMBER	SUPPORT NUMBER		SAMPLE NUMBER	PIPE TYPE	SHEATH	SIZE
1	508	NW Corner of Building 105 (7500-16)	1 2 3 4 5 6 7	Steam Ethylene Glycol Ethylene Glycol Ethylene Glycol Elbow (E.G.) Ethylene Glycol Ethylene Glycol	Fiber Fiber Fiber Fiber Fiber Fiber	18" 2" 6" 6 18" 6" 24"
2	522	NE Corner of Buildirg 105 (7500-17)	8 9 10 11 12 13 14 15	Steam Process Process Process Process Process Process Ethylene Glycol Ethylene Glycol	Fiber Fiber Fiber Fiber Fiber Fiber Fiber Fiber	8" 2" 2" 2" 2" 12"
3	71	NW Corner of Building 403 (7500-23)	16 17 13 19 20 21	Steam Ethylene Glycol Ethylene Glycol Process Process Process Process	Fiber Fiber Fiber Fiber Fiber Fiber Fiber	6" 2" 2" 2" 2" 1.5"
4	731	From Building 407 to 410 (7500-28)	23 24 25	Ethylene Glycol Ethylene Glycol Process	Fiber Fiber Metal	4" 4" 2"
5	40	NE Corner of Building 201 (7500-19)		Raffinate Raffinate Steam	Fiber Fiber Fiber	3" 3" 18"
6	562	North Center Building 201 (7500-24)	29 30 31 32	Process Raffinate Raffinate Steam Elbow (Steam)	Fiber Fiber Metal Fiber Fiber	4" 6" 4" 4"

SAMPLING	NEAREST	ZONE DESCRIPTION	INS	ULATION SAMPLES TO	BE COLLECT	ED
ZONE NO.	SUPPORT NUMBER		QUANT.	PIPE TYPE	SHEATH	SIZE
7	564	NE Corner of Building 201 (7500-24)	34	Raffinate	Metal	3"
8	Ground	SW Corner of Building 406 (N.A.)	35 -	Steam	Fiber	יינ
9	764	West of Building 406 (7500-25)	36 37 38 39	Raffinate Raffinate Raffinate Elbow (Raffinate)	Fiber Fiber Fiber Metal	3" 3" 3" 3"
10	84	Fast Center Building 301 (7500-20)	40 41 42	Steam Steam Steam	Fiber Fiber Fiber	3" 3" 18"

Scurce: WSSRAP, 1987

4.0 TEST RESULTS

Asbestos concentrations greater than 1.0 percent were found in all insulation samples from steam and raffinate pipes. Asbestos was found on all of the new (non-original) process piping near Building 403. Asbestos was also found on all elbows and all repair sections on all insulated pipes including ethylene glycol pipes. Straight sections of ethylene glycol piping were found to not have asbestos insulation. Table 4-1 presents the test results for each location.

All insulation sampled during this survey was friable. Friable materials are defined by EPA to be asbestos containing when they are greater than 1.0 percent asbestos (40 CFR 61). 40 CFR 61 sets engineering controls for preventing the release of asbestos fibers into the air apply during demolition of any structure which contains friable asbestos.

Radiological contamination was found only in a sample from a location where the exterior sheathing was no longer intact (Appendix C).

Table 4-1 Asbestos Content of Insulation on Overhead Utilities Pipes at the WSCP, November 1986.

Pipe Type	Insulation Type	Friable Asbestos Containing Material		
Steam	Plaster	Yes		
Raffinate	Plaster	Yes		
Elbows on All Insulated Pipes	Plaster over Fiberglass or Animal Hair	Yes		
Repair Sections on All Insulated Pipes	Plaster over Fiberglass or Animal Hair	Yes		
"New" Process Lines. Near Building 403	Plaster or Foam	Yes		
Ethylene Glycol	Fiberglass or Cork	No		

Source: WSSRAP, 1987.

Appendix A

Laboratory Report on the
Asbestos Content of Pipe Insulation at the WSCP



115 Hahn Street

Elmhurst, Illinois 60126

(312) 832-5658

December 17, 1986

Mr. Ken Lee MK Ferguson Company Route 2, Highway 94 South St. Charles, Missouri 63303

RE: Examination of Bulk Samples for Asbestos

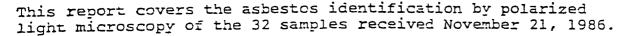
P.O. 3589-1002-1256

File No. ES-19-03-01-01

PDL Project: I-11121

EPA Lab I.D. Number 5118

Dear Mr. Lee:



The attached information tabulates the quantities of fibrous material found in each sample; the numbers will not necessarily add up to 100%, with the balance being filler and binder materials. When a sample is labeled as inhomogeneous, there is the possibility of significantly higher local concentrations than the averaged value reported. This could result in local high airborne asbestos fiber levels if the material is disturbed and appropriate safety precautions are indicated. Also, the symbol (-) indicates not detected.

Identification and quantifications were performed in accordance with Appendix A - Interim Method for the Determination of Asbestos in Bulk Insulation Samples of EPA Asbestos in Schools Regulations, Federal Register, Vol. 47, No. 103, Thursday, May 27, 1982. Analysis was initiated by a gross examination of the sample as received. Any obvious fractions were noted and samples of each fraction were mounted for polarized light microscopy in a 1.515 index liquid. When mounting samples any fiberous material is thoroughly separated for examination. Preliminary evaluation to determine the possible species of asbestos present is performed by morphology, birefringence and refractive index relative to the mounting fluid. Concurrently the relative abundance of any asbestos material, other fibers, fillers and binders is determined. Quantities are based on areal coverage and thickness of the various species present. The term trace means 0.1% or less. Identification of non-asbestos material is not as rigorous as these are not the species of interest.



PARTICLE DATA LABORATORIES, LTD.

When asbestos type fibers are seen morphologically, they are additionally characterized by immersion matching in refractive index liquid using both white light and sodium d-line. A numeric determination of birefringence is available based on the index measurments. A sample has to fit into the accepted ranges of indices, birefringence and morphological features to be classed as asbestos.

The features of the various forms of asbestos are as follows:

- Amosite: Straight thin single fibers and bundles of such fibers usually with cleanly broken ends on individual fibers; refractive indices of 1.700 and 1.695, birefringence 0.020-0.033 and parallel extinctions.
- Chrysotile: Thin fibers and fiber bundles with both straight and wavy sections. The ends of bundles tend to be frayed.

 Indices are 1.529-1.559 and 1.537-1.576, birefringence of 0.004-0.016 and the fibers exhibit parallel extinction.
- Anthophylite: Similar in morphology to amosite but indices of 1.60-1.64, birefrigence of 0.013 -0.025 and extinction varying from parallel to 15 degrees oblique.
- Crocidolite: Similar in morphology to amosite but is distinguished by blue to blue-green pleochroic coloration and indices of 1.680-1.698 and 1.685-1.706. It is commonly referred to as blue aspestos.
- Tremolite-Actinolite Series: Transparent, elongated furrowed prisms, usually with uneven, jagged ends and smooth sides, with oblique extinction and positive elongation; indices are 1.559-1.612 and 1.625-1.637. The two minerals are very similar optically and grade into each other.

Attached are representative photomicrographs of each sample and a compendium of the materials found. The micrographs are taken with crossed polars and a first order red compensator which results in the pink background and shows birefringence as bright colors other than the background and isotropic transparent material as the same color as the background.

Samples will be retained for six months unless otherwise instructed.

It has been a pleasure serving you, and we look forward to serving you again in the near future.

Respectfully submitted,

PARTICLE DATA LABORATORIES, LTD.

Olga Kist

Research Microscopist

BULK SAMPLE ANALYSIS FOR ASBESTOS CONTENT

CLIENT: MK-Ferguson Company

PDL PROJECT: 1- 11121

NON-ASBESTOS FIBERS	PAPER OTHER FIBERS (PERCENTS-BY-WEIGHT)	1-2	3-5	;	trace animal hair/90	;	1-2	. 5	trace animal hair/1-2		1-2	10 animal hair/trace -0.5	-2 cotton/5-10 animal hair/1-2	.2 animal hair/1-2	trace	
NON-ASBE	PAPER (PERCENTS-BY	ਜ	ю́	!		ł	ن م		tr.	;	<u></u>	•	1-2	1-2	. 13	10-20
•	FIBERGLASS	1-2	09	20	1-2	80	trace	1	trace		i	70-80	!	!	. 3–5	. !
	OTHER	1	1	}	! !	1	1	!	;	!	i	-	<u> </u>	!	!	!
FORMS	CROCIDOLITE IY-WEIGHT)	Į į	1	!		!	l i		{	!		1	!	!	! 1	
ASBESTOS FORMS	AMOSITE CROCIDO (PERCENTS-BY-WEIGHT)	10-20	:	i	1	<u>!</u> !	40	30	trace-0.5	30	30	:	20-30	: :	ł	10-20
	CHRYSOTILE	30	trace	20-30	trace	5-10	5-10	10	20-30	i i	5-10	1-2	20	10-20	1-2	š 1
-CMCH	GENOUS	No	No	No	Yes	No	No	No	ON	Yes	No	No	CN C	CN	CN	No
	SAMPLE 1.D.	1. IN 2000-1	2. TN 2000-3	3. IN 2000-5A	4. IN 2000-5B (Dark)	5. TN 2006-6	6. IN 2006-8	7. IN 2000-9	8. IN 2000-10	9. IN 2000-12	10. TN 2000-16	11. IN 2000-18	12. IN 2000-19	13. TN 2000-20	14. TN 2000-21	15. IN 2000-22

BULK SAMPLE ANALYSIS FOR ASBESTOS CONTENT

CLIENT: MK-Ferguson

PDL PROJECT: 1- 11121

BERS	OTHER FIBERS	1	cotton/10		1	1	! .	1	*synthetic/trace	Hair & Feathers/ 1-2	î i		ŀ	;	1		1
NON-ASBESTOS FIBERS	PAPER (PERCENTS-BY-WEIGHT)	trace	5-10	3-5	'n	5-10	!	3-5	!		1	1	χ	1	1-2	trace	trace
3	FIBERGLASS	l	!	1	[]	1	ļ	1	06		l 1	!	1	i l	1	10	80
	OTHER	!	i	i i	!	1	! !	! !	I I		ļ	;	1	l	i I	!	l i
FORMS	SITE CROCIDOLITE	!	1-2	. !	!	1	Į į	1	}		1	1	ļ i	!	!	i 1	1
ASBESTOS FORMS	AMOSITE (PERCENTS-B	20-30	20	10	20-30	20	30-40	10-20	Ľrace*		20-30	20-30	20	30-40	20-30	20-30	1-2
	CHRYSOTILE	1-2	5-10	10	20-30	5-10	10-20	1.0	!		20-30	1-2	5-10	20	1-2	10-20	٧.
	HOMO- GENOUS	Yes	No	No	ON	N _O	N _o	C	N _O		No	Yes	No	Yes	NO	No	Yes
Page 2	SAMPLE I.D.	16. IN 2000-25	17. IN 2000-26	18. IN 2000-27	19. IN 2000-28	20. IN 2000-29	21. IN 2000-30	22. IN 2000-31	23. TN 2000-32		24. IN 2000-33	25. IN 2000-34	26. IN 2000-35	27. IN 2000-36	28, IN 2000-37	29. IN 2000-38	30, TN 2000-40

BULK SAMPLE ANALYSIS FOR ASBESTOS CONTENT

CLIENT: MK-Ferguson Page 3

PDL PROJECT: 1- 11121

EBS	OTHER FIBERS (HT)	1 1	
NON-ASBESTOS FIBERS	PAPER (PERCENTS-BY-WEIGHT	10	5-10
	FIBERGLASS	;	!
	ОТНЕВ	;	;
FORMS	AMOSITE CROCIDOLITE PERCENTS-BY-WEIGHT)	-	1 1
ASBESTOS FORMS	AMOSITE (PERCENTS-E	017	30
	CHRYSOTILE	5-10	10-20
	GENOUS	No	No
)	SAMPLE I.D.	31. IN 2000-41	32. IN 2000-42

^{* =} Sample IN 2000-32 has inhomogeneous portion (possible contamination) consisting of animal hair feathers and synthetic fibers. this material does burn, indicating it not to be asbestos material.

Appendix B

Field Data Sheets for Bulk Material Sampling for Asbestos Content of Pipe Insulation at the WSCP

WELDON SPRING SITE REMEDIAL ACTION PROJECT (WSSRAP) Route 2, Highway 94, St. Charles, Missouri 63303 Fhone (314) 441-8086 Telex (314) 447-0803

BULK MATERIAL SAMPLING FOR ASBESTOS - FIELD DATA

EAMPLING LOCATION Suilding Number: INTEX-BUILDING OVERHEAD UTILITIES Building Number: INTEX-BUILDING OVERHEAD UTILITIES Suilding Number: INTEX-BUILDING OVERHEAD UTILITIES Ficon NEAREST SUPPORT No.: 508 See: SAMILING Z.NE Construction Drawing Other Construction Drawing Other Toolot Rack MATERIAL TYPE A Pipe Insulation Fipe Diameter: Steam, inches Fipe Type: Steam, Ethylene Glycol, Raffinate, Other Color: Wall Fiber Material Fiber Material Color: WHITE Hardness: Sibrows (Fribble), Fraulty (Soft), or Nard (Concrete-like) Thickness: Sibrows (Fribble), Fraulty (Soft), or Nard (Concrete-like) COMMENTS COMMENTS Fadiation Level O.4 DPM Background Only Above Sackground Comment: Comment: And Comment:	SAMPLE NUMBER: IN-2000- Date: NOVEMBER 17, 1986
Construction Drawing Other Construction Drawing Other Construction Drawing Other To P RACK MATERIAL TYPE A Fipe Insulation Fipe Diameter: Fipe Type: Ceiling Floor Wall Soller Insulation Other Cover Over Material Fiber Matal, Plaster, None, Cther Filed Hardness: Hardness: Thickness: Comments Comments Fibrus (Friedle), Frauler (Soft), or Hard (Concreta-like) Comments Comments Factor Level	SAMPLE NUMBER:
### ATERIAL TYPE A	SAMPLING LOCATION Building Number: INTER-BUILDING OVERHEAD UTILITIES FIG. NEAREST SUPPORT NO.: 508 SAMPLING Z.NE
Floor Wall Sciler Insulation Other COVER OVER MATERIAL Fiber Matal, Plaster, None, Cther Fiber Matal, Plaster, None, Cther Fiber Color: Hardness: Thickness: Thickness: COMMENTS Radiation Level O.4 DPM Fackground Only Above Fackground	MATERIAL TYPE X Fipe Insulation Fipe Diameter: Steam, Ethylene Glycol, Raffinate, Other
DESCRIPTION OF MATERIAL Color: Hardness: Thickness: COMMENTS Radiation Level — 0.4 DPM Background Only Above Background Cpm Instrument:	Floor Wall Esilar Insulation
Color: Hardness: Thickness: Thickness: COMMENTS Radiation Level — 0.4 DPM Background Only Above Fackground cpm Instrument:	Fiber Matal, Flaster, None, Cther Fiber Fiber Fiber
Radiation Level — 0.4 DPM Background Only Above Background com Instrument:	Color: Hardness: Fibrous (Friable), Erapular (Soft), or Hard (Concrete-Like)
Radiation Level — 0.4 DPM Background Only Above Background cpm Instrument:	COMMENTS
Simple Collection rensemble:	Radiation Level — 0.4 DPM Background Unly Above Background com Instrument: Sample Collection Personnel:

WELDON SPRING SITE REMEDIAL ACTION PROJECT (WSSRAP) Route 2, Highway 94, St. Charles, Missouri 63303 Fhone (314) 441-8086 Telex (314) 447-0803

BULK MATERIAL SAMPLING FOR ASBESTOS - FIELD DATA

SAMPLE NUMBER: IN-2000-2 Date: NOVEMBER 17, 1986
TANDLE THE LECATION
Fuilding Number: INTER-BUILDING OVERHEAD UTILITIES FIG. NEAREST SUPPORT NO.: 508 FROM SAMPLING ZONE Construction Drawing Other 2nd Pipe from South on Topkack
MATERIAL TYPE Z Fipe Insulation Fipe Diameter: Steam, Z Ethylene Glycol,Raffinate, Cailing
Floor Wall Epiler Insulation Other
Fiber, Matal, Flaster, None, Other Tiper (National)
DESCRIPTION OF MATERIAL
Color:
Handness: X Fibrous (Frishle), Eranular (Soft), or Hand (Concrete-Like)
Thickness: <u>3</u> inches
COMMENTS FIBERGLASS
Radiation Level Background Only Above Background com Instrument:
Sample Collection Personnel: 11.50 1/2000 1/17/02 Company: Tuesta Engineering
Jacobi 1

WELDON SPRING SITE REMEDIAL ACTION PROJECT (WSSRAP) Route 2, Highway 94, St. Charles, Missouri &3303 Fhone (314) 441-8086 Telex (314) 447-0803

BULK MATERIAL SAMPLING FOR ASBESTOS - FIELD DATA

SAMPLE NUMBER: IN-2000-3 Date: NOVEMBER 17, 1986
ENTER-BUILDING OVERHEAD UTILITIES
NEAREST SUPPORT NO.: 508
SAMPLING ZONE
Construction Drawing Other T500-16 35-4
Other Strand
MATERIAL TYPE
Fipe Insulation
Fipe Insulation Fipe Diameter: Fipe Type: Steam, Ethylene Glycol, Raffinate, Other
Ceiling
Floor Wall
Eciler Insulation
Other
COVER OVER MATERIAL
Fiber Metal, Flaster, None, Other
DESCRIPTION OF MATERIAL
Color:
Hardness: Fibrous (Frishla), Branular (Soft), or Hard (Concreta-Like)
Thickness: <u>3</u> inches
COMMENTS Fiberaless
COMMENTS 7 (Serates)
Radiation Level Background Only
Above Background com Instrument:
Sanoie Collection Personnel: ALER HOLE THE Company: Ties to the sense
J. Gozawi

SAMPLE NUMBER: IN-2000-4 Date: NOVEMBER 17. 1986
SAMPLE NUMBER:
Building Number: INTER-BUILDING OVERHEAD UTILITIES
NEAREST SUPPORT NO.: 368
SAMPLING ZONE
Construction Drawing Other SAMPLING ZONE 7500-16 2 Ad From North on Top Rack
Other
· · · · · · · · · · · · · · · · · · ·
MATERIAL TYPE X Fipe Insulation
Fipe Insulation Fipe Diameter: Fipe Type: Steam, Ethylene Glycol, Raffinate,
Pipe Type: Steam, Ethylene Glycol,
Cailing
Floor Wall
Boiler Insulation
Otner
Fiber, Metal, Flaster, None, Other Francisco Tarrapper
Fiber, Metal, Flaster, None, Other Francisco Tarraps
DESCRIPTION OF MATERIAL
Color: Ward (Contrained its)
Hardness: Fibrous (Friable), Eranular (Soft), or Hard (Concrete-Like)
Thickness:inches
COMMENTE Fiberalus
COMMENTS + 186 AND
Radiation Level
- ABBVE BACKGI COMB - Junki
Sample Callection Personnel: The Aller The Third Campany: The Aller The Third Campany: The Aller The Third
المستراق الم

		D NOVEMP	ER 17, 1986
SAMPLE NUMBE	IN-2000-5		
SAMPLING LOC	CATION Lilding Number: INTER-	RUII DING OVERHEAD	UTILITIES
E.	tilding Number: Tir Zi	719	
	NEAREST SUPPORT NO .:		
چنگ	SAMPLING ZONE	7500-16	
Co	SAMPLING ZONE Instruction Drawing ther	=11=12 = - 1 7 1 2	Pros from
Ot	iher 3	1 4	C 1 - 1-
	\wedge	wrin - New	Construction
MATERIAL TYP	<u>=</u>	-1 R 01N	
F:	ipe Insulation ≈ 2	ELBOW	
	Fipe Type: Steam,	Ethylene Glycol,	Raffinats,
	Crie: _		
F:	eiling loor		
F:	all ciler Insulation		•
=:	ciler insdiacien ther		
COVER OVER	MATERIAL	- > - 1- >	
Fiber, M	<u>MAtenimo</u> stal, Flaster, None, Othe	F TIBER (BREIN)	
DESCRIPTION	OF MATERIAL	ster/brown inner r	naterial
Color:	a) hite ala	sty promy when	
Hardne	ess: XFibrous (Friable), _	Eramuiar (Soft), or Hard (Con	ucreca-fixs)
Thickn	ness: <u>R</u> inches		
		of Etylene Clurch lim	(clkm)
COMMENTS	Comment on tot	of Spilore Circul IIm	18100
•	4 1		
•			
	evel 0.4 DPM	•	
<u>Radiation l</u>	<u>level</u> Background Only		
	Above Background	cpm Instrument:	
Simple Collection	Personnel: The Men a 11/1/2	Caroliny: Sucial	ma English Com
Sample Collection	The Day of the last of the las		
	(/)		

SAMPLE NUMBER: T	N-2000-6	Date: NoVE	MBER 17, 1986
SAMPLE NUMBER:	113		·
SAMPLINE LOCATION	INTE	R-BUILDING OVERHEAD	D UTILITIES
Building Floor No	EAREST SUPPORT NO	: 508,	
Seem S	AMPLING ZONE		
	ction Drawing	7500-16	-11
Other		Coming down from	2 Fipe Tron
. •		7500-16 Coming down from North on top Leads to "Ton	rack-
MATERIAL TYPE		Leads to "Tan	k" on lower
X Fine In:	sulation Diameter: \$\frac{1}{2} \begin{align*} 2	1-2 / 2 /	. •
Fip	E Diameter: & 6	inches n, Ethylene Glyco	n Raffinata.
Fip	Type: Stein Other		
_ Ceiling			
Floor	•		
Wall Eciler	Insulation		
Otner _			
COVER OVER MATERI	<u>4L</u>	T-0 - 0	f as-
Fiber, Metal,	Flaster, None, Ct	TET FIBER ?	1 1
DESCRIPTION OF MA	TERIAL		
Color:	BROWN		
Hardness:	Fibrous (Frisble),	Eranular (Soft), or Hard	(Concrete-Like)
Thickness:	inches		
			~ F.: 1
COMMENTS	noes not Lour 1	ik, straj film alas	S. Maprice Onknow
Stra	At -section Co	mine down from JE	. C. line Horm = angl
	, CORK		
	•		
Radiation Level	1.1	DPM	
Backgro	und Unly Hackaround	D F / / I cpm Instrument:	
FUC VE	Ve Ale de	Coopeny: 1 s	aulas Enginerie
Samoie Callection Personnel:	1,000 1,000 10 111	Faddauli Kilan	
	() Signal !		

SAMPLE NUMBER: IN-2000-7 Date: NOVEMBER 17, 1986
SAMPLINE LOCATION
. Building Number: INTER-BUILDING OVERHEAD UTILITIES
NEAREST SUPPORT NO .: JUR
SAMPLING ZONE
Other Yant on lower tact.
NATEDIAL TYPE
MATERIAL TYFE
Fipe Insulation
Fipe Insulation Fipe Diameter: Steam, Ethylene Glycol, Raffinate, Other
Cailing
Floor
Wall Eciler Insulation
COVER OVER MATERIAL
Fiber, Metal, Flaster, None, Other Taroace-
DESCRIPTION OF MATERIAL _
Color: BRown
Handness: Yeibrous (Friable), Granular (Soft), or Hand (Concrete-Like)
Thickness: Sinches
ECMMENTS CORK INSULATION
Radiation Level
Background Only
Above Backgroundcpm Instrument:
Samoia Collection Personnel: And Min. C. Wirth Company: Vaca E. Engineer in
() ()

TN-2000-8 Data: NOVEMBER 17. 1986
SAMPLE NUMBER:
SAMPLING LOCATION Building Number: INTER-BUILDING OVER HEAD UTILITIES
Building Number: 177 27 50725776
NEAREST SUPPORT NO.: 500
SAMPLING ZONE Z
Other Too Rack, Farthest South
Other Jos Rack, Partnes, 30014
MATERIAL TYPE
X Fipe Insulation
Pipe Diameter:inches
Fipe Insulation Pipe Diameter: Steam, Sthylene Glycol, Raffinate, Other
Ceiling
Floor Wall
Poiler Insulation
Other
COVER OVER MATERIAL
riber) Metal, Flaster, None, Other Tar Papar
Fiber Metal, Flaster, None, Other
Fiber Metal, Flaster, None, Other
DESCRIPTION OF MATERIAL Color:
DESCRIPTION OF MATERIAL Color: Color:
DESCRIPTION OF MATERIAL Color: Hardness: Tot 1000 To
DESCRIPTION OF MATERIAL Color: Hardness: Thickness: A part of the property
DESCRIPTION OF MATERIAL Color: Hardness: Thickness: Thickness:
DESCRIPTION OF MATERIAL Color: Hardness: Tot 1000 To
DESCRIPTION OF MATERIAL Color: Hardness: Thickness: Thickness:
DESCRIPTION OF MATERIAL Color: Hardness: Thickness: Thickness:
DESCRIPTION OF MATERIAL Color: Hardness: Thickness: COMMENTS This was a large of the contract of the cont
DESCRIPTION OF MATERIAL Color: Hardness: Thickness: COMMENTS Radiation Level Background Only
DESCRIPTION OF MATERIAL Color: Hardness: Thickness: COMMENTS Radiation Level Background Only Above Eackground
DESCRIPTION OF MATERIAL Color: Hardness: Thickness: COMMENTS Radiation Level Background Only

SAMPLE NUMBER: IN-2000-9 Date: NOVEMBER 17, 1986
SAMPLE NUMBER:
SAMPLING LOCATION Building Number: INTER-BUILDING OVERHEAD UTILITIES
Building Number: INTEX-BUILDING OVER HELL
NEAREST SUPPORT NO .: 500
SAMPLING ZONE 2
Ezem Shirt Bills
Construction Drawing 7500-17 And level Forthest South
Other
•
MATERIAL TYPE
Fipe Insulation Fipe Diameter:inches Fipe Type: Steam, Ethylene Glycol, Raffinate, Other Trocess
Fine Diameter:Inches Flycol Raffinate,
Pipe Type: Process ?
Cailing
Floor Wall
Eciler Insulation
Ciner
·
COVER OVER MATERIAL
Fiber, Matal. Plaster, None, Other 100000
Fiber, Metal, Plaster, None, Other Tornand
Fiber, Matal, Plaster, None, Other 100000
Fiber, Matal, Plaster, None, Other 100000
Piber, Matal, Plaster, None, Other 1000000000000000000000000000000000000
DESCRIPTION OF MATERIAL Color: WHITE Color: West (Contrated by the color)
DESCRIPTION OF MATERIAL Color: Hardness: Fibrous (Friable), Eranular (Soft), or Hard (Concrete-Like)
DESCRIPTION OF MATERIAL Color: WHITE Color: West (Contrated by the color)
DESCRIPTION OF MATERIAL Color: Hardness: Thickness: Inches DESCRIPTION OF MATERIAL Fibrous (Friable), Eranular (Soft), or Kard (Concrete-Like) Inches
DESCRIPTION OF MATERIAL Color: Hardness: Thickness: Inches DESCRIPTION OF MATERIAL Fibrous (Friable), Eranular (Soft), or Kard (Concrete-Like) Inches
DESCRIPTION OF MATERIAL Color: Hardness: Fibrous (Friable), Eranular (Soft), or Hard (Concrete-Like)
DESCRIPTION OF MATERIAL Color: Hardness: Thickness: Inches DESCRIPTION OF MATERIAL Fibrous (Friable), Eranular (Soft), or Kard (Concrete-Like) Inches
DESCRIPTION OF MATERIAL Color: Hardness: Thickness: Inches DESCRIPTION OF MATERIAL Fibrous (Friable), Eranular (Soft), or Kard (Concrete-Like) Inches
DESCRIPTION OF MATERIAL Color: Hardness: Thickness: COMMENTS Stanless Stall Diffe
DESCRIPTION OF MATERIAL Color: Hardness: Thickness: COMMENTS Stanless Stall Diffe
DESCRIPTION OF MATERIAL Color: Hardness: Thickness: COMMENTS Stanless Stall Dite Comments Description of Material Fibrous (Friable),
DESCRIPTION OF MATERIAL Color: Hardness: Thickness: COMMENTS Stanless Stall Fibrous (Friable), Eranular (Soft), or Kard (Concrete-Like) Thickness: COMMENTS Stanless Stall Faction Level Factorium Coly Fact
DESCRIPTION OF MATERIAL Color: Hardness: Thickness: COMMENTS Stanless Stall Fibrous (Friable), Eranular (Soft), or Kard (Concrete-Like) Thickness: COMMENTS Stanless Stall Faction Level Factorium Coly Fact
DESCRIPTION OF MATERIAL Color: Hardness: Thickness: Comments Comments

BULK MATERIAL SAMPLING FOR ASSESTED 5
17 1001
CAMELE NUMBER:
SAMPLING LOCATION Building Number: INTER-BUILDING OVER HEAD UTILITIES
Building Number: TNIEX-BUILDING OVER HER
NEAREST SUPPORT NO.: 535
SAMPLING ZONE
$\frac{7500-17}{7500-17}$
Construction Drawing 7500-17 3ºl level down Farthest South
Other
MATERIAL TYPE
X Fipe Insulation o' inches
Fipe Diameter: Ethylene Glycol, Raffinate,
MATERIAL TYPE X Fipe Insulation Fipe Diameter: Steam, Ethylene Glycol, Raffinate, Other Other
Ceiling
Floor Wall
Wall Foiler Insulation
Other Instituted
Fiber, Metal, Flaster, None, Other 101 Dubel Colet
Fiber, Metal, riaster, Money Demo-
DESCRIPTION OF MATERIAL
Salari WhiG
Hardness: Fibrous (Friable), Granular (Soft), or Hard (Concrete-Like)
Thickness:
COMMENTS 2" Stanles Stell Pipe
COMMENTS & Stanles Stanles
$\sim \sim DOM$
Radiation Level O.O DPM Background Only Communication Level
Above Eackground cpm Instrument:
Simple Collection Personnel: Var Man, Market Community C
Sample Collection Personnel: Xun (160) . (17)

SAMPLE NUMBER: IN-2000-11 Date: NOVEMBER 17, 1986
\cdot
Building Number: INTER-BUILDING OVERHEAD UTILITIES
NEAREST SUPPORT NO .: 527
SAMPLING ZONE
Other Drawing 7500-17 Other Dyest An at the Section
1 Aug bi All control
Other. Util at the service
MATERIAL TYPE
X Fine Insulation /
Fipe Diameter: O inches
Fipe Type: Steam, Ethylene Glycol, Raffinate,
Fipe Insulation Fipe Diameter: Fipe Type: Steam, Ethylene Glycol, Raffinate, Other Process?
Floor
——— Wall Boiler Insulation
Other
COVER OVER MATERIAL
Fiber, Metal, Flaster, None, Other Tor Paper Cover
DESCRIPTION OF MATERIAL
Color: Yellow
Color:
Hardness: Fibrous (Friable), Eranular (Soft), or Hard (Concrete-Like)
Thickness: 2_inches
COMMENTS 2" dia Steel fife
COMMENTS COMMENTS
Fiber glass Trisulation
Shorthing Torn at Samolina Location
\sim
Faciation Level 7 2 DPM - do not ship
Background Only Offsite
Background Only cpm Instrument: Off3it2
Background Only — Background — com Instrument: — // // // // // // // // // // // // //
Background Only Offsite

SAMPLE NUMBER: IN-2000-12 Date: NOVEMBER 17, 1986
SAMPLE NUMBER:
EAMPLING LOCATION Building Number: INTER-BUILDING OVERHEAD UTILITIES ELECTION NEAREST SUPPORT NO.: 522 Seem SAMPLING ZONE Construction Drawing Other Mills Other Mills
MATERIAL TYPE X Fipe Insulation Pipe Diameter: Steam. Fipe Type: X Other Process 7 Ceiling Floor Wall Epiler Insulation
COVER OVER MATERIAL Fiber, Metal, Flaster, None, Other Torioger
Color: White Light insulation
Hardness:
COMMENTS Stanley Stal
Radiation Level Eackground Only Above Eackground Community Above Eackground Community Community Campany: Taken 25 English Community Campany: Taken 25 English Community

BULK MATERIAL SAMPLING FUR ASSESTED DATA
SAMPLE NUMBER: TN-2000-13 Date: NOVEMBER 17, 1986
SAMPLING LOCATION Building Number: INTER-BUILDING OVERHEAD UTILITIES
NEAREST SUPPORT NO .: 5 0
Construction Drawing 7500-17
Other 2nd Level Pown, Forthest North
MATERIAL TYPE
Fipe Insulation Fipe Diameter: inches Fipe Type: Steam, Ethylene Glycol, Raffinate, Other free < 55 }
Ceiling Floor Wall
Eciler Insulation Other
COVER OVER MATERIAL Fiber, Metal, Flaster, None, Other Word . Nottina
Fiber, Metal. Flaster, McNe, Cone
Color: Grey
Hardness: Fibrous (Frizble), Granular (Soft), or Hard (Concrete-Like) Thickness:inches
COMMENTS Like Stukotom (Grew)
Stanless Steel construction pie
Fadiation Level — Background Only Above Background com Instrument:
Simple Collection Personnel: Kut Albana III 17/61 Company: Vecubo Environment
The grant of the same of the s

SAMPLE NUMBER: IN-2000-14 Date: NOVEMBER 17, 1986
SAMPLING LOCATION Building Number: INTER-BUILDING OVERHEAD UTILITIES
SECON SAMPLING ZONE
Construction Drawing 7500-17 Other Top Level, 31° From North
Other Top Level. 5 From North
MATERIAL TYPE Z Fipe Insulation Fipe Diameter: inches Fipe Type: Steam, Z Ethylene Glycol,
Ceiling Floor Wall Epiler Insulation Other
Fiber, Matal, Plaster, None, Other (TAR Pales Shet)
DESCRIPTION OF MATERIAL Vellow
Color: Yextow Hardness: X Fibrous (Friable), Granular (Soft), or Hard (Concrete-Like) Thickness:inches
COMMENTS FIBER GLASS Insulation
Radiation Level Background Only Above Background cpm Instrument:
Sample Collection Personnel: Kill Clay. 19/17/5/. Company: Takent Englager in

SAMPLE NUMBER: IN-2000-15	November 17
	Date: NOVEMBER 17, 1986
SAMPLING LOCATION Building Number: INTER-BUILDING	C ALPONEDD LITHITIES
NEAREST SUPPORT NO .: CAR	S OVER HEAD OF TELLITES
SAMPLING ZONE 2	
	7
Construction Drawing $\frac{7500-1}{1-0.1}$	vel, 27 from North
Jen Le	vel, 2 - from North
MATERIAL TYPE	
Fine Insulation 7	
Fipe Diameter:	· •
Fipe Insulation Fipe Diameter: Steam, Y Ethy Other	viene Glyccl,Raffinate,
Cailing	·
Floor	
Wall Eciler Insulation	
Ctner	
,	
COVER OVER MATERIAL	
Fiber, Metal, Flaster, None, Other	TAR Dupos short
DESCRIPTION OF MATERIAL	
Color: Yellow	
Hardness: Fibrous (Friable), Granular (Soft), or Hard (Concrete-Like)
Thickness:inches	
COMMENTS FIRE GLASS I ASULATION	
The relation	
Radiation Level	
Background Only Above Background com Instr-	
Saaole Collection Personnel: Virt Man. 11/1/21;	Japany: Jacobs Eleiner 19

BOLK THICKS IN .	. , —
SAMPLE NUMBER: TN-2000-16 Date: NOVEMBER	1/, 1986
Building Number: INTER-BUILDING OVER HEAD UT	TILITIES
SAMPLING ZONE 3	•
Construction Drawing 7500-83 Other Tob Texas Red from East	-
Other	<u></u>
MATERIAL TYPE Z Fipe Insulation Fipe Diameter: inches Fipe Type: Steam, Ethylene Glycol,	Raffinate,
Ceiling Floor Wall Eoiler Insulation Other	•
Fiber, Metal, Flaster, None, Other Remaining (m)	
Color: White in Insulation	
Hardness: Fibrous (Friable), Granular (Soft), or Hard (Concrete Thickness:inches	-Like)
COMMENTS Or missing from the blue phint	
Radiation Level Background Only Above Background com Instrument:	
Saaole Collection Personnel: Not Way 111-171 Company: Jacobs E.	Since ins

SAMPLE NUMBER: IN-2000-17 Date: NOVEMBER 17, 1986
EMPLING LOCATION Fuilding Number: INTER-BUILDING OVERHEAD UTILITIES
Building Number: INTEXTBUILDING OVER THE TOTAL OF THE PROPERTY
NEAREST SUPPORT NO .: 71
SAMPLING ZONE 3
Construction Drawing 7500-33 Other 709 100 From East
Other 210 Fram Cast
MATERIAL TYPE
Fipe Diameter: C P inches
Fipe Insulation Pipe Diameter: inches Pipe Type: Steam, Ethylene Glycol, Raffinate, Other
Ceiling
Floor
Wall
Eciler Insulation Other
Fiber, Metal, Flaster, None, Other TAR Pulses (but)
Finer, Netal, Flasts, None, Tone,
DESCRIPTION OF MATERIAL
Color: Yellow
Handness:Fibrous (Friable), Eranular (Soft), or Hard (Concrete-Like)
Thickness:inches
COMMENTS FIBER 9/455
COMMENTS FIRM 91435
Radiation Level Background Only
Chave Fackground com Instrument:
Sample Callection Personnel: King Min en 11/1-1/21. Gaegany: 15 July 1 1/2 - 12
الذجوم

SAMPLE NUMBER: IN-2000-18 Date: NOVEMBER 17, 1986
·
Building Number: INTER-BUILDING OVERHEAD UTILITIES
NEAREST SUPPORT No.: 71
SAMPLING ZONE 3
Construction Drawing 7500-23 Other Top, Forthest East
Other Top, forthest tast
MATERIAL TYPE
- V Sing Topulation
Fipe Diameter: 2" inches
Fipe Diameter: 2" P inches Fipe Type: Steam, Ethylene Glycol, Faffinate, Other
Ceiling Other
Floor Wall
Boiler Insulation Other
Othe:
COVER OVER MATERIAL
Fiber, Metal, Flaster, None, Other TAR Echal (W2:
\
DESCRIPTION OF MATERIAL AC
Color: 1ellow
Hardness: Fibrous (Frisble), Eranular (Soft), or Hard (Concrete-Like)
~ 11
Thickness:inches
F10 =0 =1 > 4
COMMENTS FIBERCLASS
<u> </u>
Radiation Level
Background Only Above Background cpm Instrument:
Sample Collection Personnel: Kuch lines 1/15/60 Coapany: January: January: January:
The state of the s

SAMPLE NUMBER: IN-2000-19 Date: NOVEMBER 17, 1986
SAMPLING LOCATION THE COUNTY OF THE CONTROL OF THE
Fuilding Number: INTER-BUILDING OVERHEAD UTILITIES
NEAREST SUPPORT NO .: 71
SECON SAMPLING ZONE Q
Construction Drawing 7500-33
other 2 level from Tib farthert East
MATERIAL TYPE
Fipe Diameter: C inches
Fipe Insulation Fipe Diameter: inches Fipe Type: Steam, Ethylene Glycol, Raffinate, Other
Ceiling
Floor
Wall Foiler Insulation
Folie: Instiscion
POURD RURE MATERIAL
Fiber, Metal, Plaster, None, Other TAR Palber (Wes
Fiber, Metal, Flaster, None, Cone:
DESCRIPTION OF MATERIAL
Color: White
Hardness: Fibrous (Friable), Eranular (Soft), or Hard (Concrete-Like)
Thickness: 2" inches
COMMENTS Stain LCSS Stall Fibe
CONTRACTO
Radiation Level Background Only
Above Backgroundcpm Instrument:
Single Collection Personnel: Kutillen - Illitike Company: Jacob's Engineering
- Comment of the comm

BUCK PRITERIAL CONTROL
SAMPLE NUMBER: TN-2000-20 Date: NOVEMBER 17, 1986
SAMPLE NUMBER:
SAMPLING LOCATION Building Number: INTER-BUILDING OVERHEAD UTILITIES
Building Number: INTEX-BUILDING OVER HELL
NEAREST SUPPORT NO .: 71
SAMPLING ZONE 3
7500-23
Construction Drawing 7500-23
Other Botton lend furthest East
MATERIAL TYPE
X Fipe Insulation inches
Fipe Diameter:
Fipe Insulation Fipe Diameter: inches Fipe Type: Steam, Ethylene Glycol,Raffinate, Other Other Other
Ceiling
Floor
Wal 1
Eciler Insulation
Other
COVER GUER MATERIAL
Fiber, Metal, Plaster, None, Other TAR Pulps (000)
Piper, Metal, Finance, Finance
DESCRIPTION OF MATERIAL
White I am I want
formular (Each) are: Ward (Concrata-like)
Thickness: inches
, , , , , , , , , , , , , , , , , , ,
COMMENTS Stanley Stall
<u>Connerve</u>
Radiation Level
Background Only cpm Instrument:
Above sackground
Sample Collection Personnel: With Millian 11/1/1/1/1/1/1/1/1/1/1/1/1/1/1/1/1/1/1
SINDIZ CONTECTION FOR SOME

BULK MATERIAL SHIPLING FOR HEBESTOS 12225 5
SAMPLE NUMBER: IN-2000-21 Date: NOVEMBER 17, 1986
Building Number: INTER-BUILDING OVERHEAD UTILITIES
Building Number: Entantal Over 1997
NEAREST SUPPORT NO .: 7/
SAMPLING ZONE 3
Construction Drawing 7500- 22
and less truthest West.
Other
MATERIAL TYPE
Fipe Diameter: 1/2 inches
Fipe Insulation Fipe Diameter:inches Fipe Type:Steam,Ethylene Glycol,Raffinate, OtherOther
X Uther Was 255
Ceiling Floor
Eciler Insulation
Ciner
COVER OVER MATERIAL
Fiber, Matal, Flaster, Mone, Other
Fiber, Neter, France, Jones
DESCRIFTION OF MATERIAL
Color: <u>GREY</u>
Hardness:Fibrous (Ariable), Granular (Soft), or Hard (Concrete-Like)
Thickness:inches
COMMENTE Stud Form !! (Black); Stain less stall file
COMMENTS STURY FORM "IL OTACK" ; JUNE TICK
Raciation Lavel
Background Only
Above Background cpm Instrument:
Sample Collection Personnel: Vict Man = 11/17/1/ Coopeny: D. Tricoles Election
Sadole Collection reladiner.

SAMPLE NUMBER: TN-2000 - 22 Date: NOVEMBER 17, 1986
EMPLING LOCATION Fuilding Number: INTER-BUILDING OVERHEAD UTILITIES Fice. NEAREST SUPPORT NO.: 71 Seem SAMPLING Zone Construction Drawing Other Bottom level futher West.
MATERIAL TYFE X Fipe Insulation Fipe Diameter: Steam, inches Fipe Type: Steam, Ethylene Glycol, Raffinate, Other PROCESS ?
Cailing Floor Wall Ecilar Insulation Coner
Fiber, Metal, Flaster, None, Other TAR Cova:
Color: Hardness: Thickness: DESCRIPTION OF MATERIAL Whit Fibrous (Frigidal), Eranular (Soft), or Hard (Concrete-Like) Thickness: a inches
COMMENTS Stamles Steel 1910.
Radiation Level Background Only Above Background cpm Instrument:
Sample Collection Personnel: Virtilla Mortiela Company: Jacobe Engineering

BUEN PINTERTAL DAME ETHO . C. T.
SAMPLE NUMBER: IN-2000- 23 Date: NOVEMBER 17, 1986
SAMPLE NUMBER:
Building Number: INTER-BUILDING OVERHEAD UTILITIES
Building Number: LNI EX-BUILDING OVER HEAD
NEAREST SUPPORT NO : 731
SAMPLING ZONE 4
Construction Drawing 7500-28
Other Extrost upst
· · · · · · · · · · · · · · · · · · ·
MATERIAL TYPE
Fipe Insulation
Fipe Insulation Fipe Diameter:inches Fipe Type: Steam,X Ethylene Glycol,
Other
Ceiling
Ficor
₩=11
Boilar Insulation
Other
COVER OVER MATERIAL
Fiber, Matal, Flaster, None, Other TRR matel (ove)
DESCRIPTION OF MATERIAL
Color: \ellow
Hardness: Yelbrous (Friable), Granular (Soft), or Hard (Concrete-Like)
2 1
Thickness:inches
COMMENTS FUREL GLASS
(Sample label Start & bottle base)
Radiation Level Background Only
Delicie only
Above Background com Instrument:
Above Backgroundcpm Instrument:
Simple Collection Personnel: \(\frac{1}{\infty} \) \(\frac{1}{\inf

BULK THE LITTLE STATE OF THE LOCAL STATE OF THE LOC
SAMPLE NUMBER: IN-2000 - 24 Date: NOVEMBER 17, 1986
EAMPLING LOCATION Building Number: INTER-BUILDING OVER HEAD UTILITIES
Building Number: IN EXPERING OVER HELL
NEAREST SUPPORT NO .: 731
SEET SAMPLING ZONE 4
Construction Drawing 7500-36
Other EAST End.
MATERIAL TYPE
Translation
Fige Diameter: inches
Fipe Institution Fipe Diameter: inches Pipe Type: Steam, Ethylene Glycol,Raffinate,
Other
Ceiling
Floor Wall
Wall Eciler Insulation
Other
COVER OVER MATERIAL
Fiber, Metal, Plaster, None, Other MR Pain (Wal
DESCRIPTION OF MATERIAL WOW
Color:
Hardness: Y Fibrous (Friable), Branular (Soft), or Hard (Concrete-Like)
· · · · · · · · · · · · · · · · · · ·
Thickness:inches
COMMENTS FIBER CLASS .
Radiation Level Eackground Only
Background Only
Abaya Barkarana com Instrument:
Sample Collection Personnel: Val Man Illing Cappany: Jacobs Engineering

TN-2000-26 Date: NOVEMBER 17, 1986
SAMPLE NUMBER: IN-2000-26 Date: NOVEMBER / 1986
SAMPLING LOCATION Building Number: INTER-BUILDING OVERHEAD UTILITIES
Building Number: LNIZX-BUILDING OVER 112.1.
NEAREST SUPPORT NO .: 38
SAMPLING ZONE 5
Construction Drawing 7500-19
Other Doller
MATERIAL TYPE
V - v v v) - b d an
Fipe Diameter: Inches Fipe Type: Stelam, Ethylene Glycol, Raffinate, Other
Cailing
Floor Wall Eoiler Insulation
W=11
Eoiler Insulation Other
0 cnc.
Fiber, Metal, Flaster, None, Other TAR buby Cover
Fiber, Metal, Flaster, None, Other
DESCRIPTION OF MATERIAL
Color: Wit
Hardness: Fibrous (Friable), Eranular (Soft), or Hard (Concrete-Like)
Thickness: 11211 inches
COMMENTS CTOAM
COMMENTS STORM
Radiation Level Background Only
Above Background cpm instrument:
T / Francisco
Sample Collection Personnel: Chile II Louis - 11/1/1-15 Concarry: 120513 200100

SAMPLE NUMBER:
THE LECATION
EMPLINE COCATION Building Number: INTER-BUILDING OVERHEAD UTILITIES Fico: NEAREST SUPPORT NO.: 38 Seem SAMPLING ZONE Construction Drawing 7500-19 Other 2 de Tunthust West
MATERIAL TYPE
Fipe Insulation Fipe Diameter: fipe Diameter: Steam, inches Fipe Type: Steam, Ethylene Glycol, Raffinate, Other
Ceiling Floor Wall Eciler Insulation Ctner
Fiber, Metal, Flaster, None, Other Mctal (00)
DESCRIPTION OF MATERIAL Color: White Hardness: Z fibrous (Friable), Granular (Soft), or Hard (Concrete-Like) Thickness:inches
COMMENTS Stayles Stol 17.15
Radiation Level Background Unly Above Background cpm Instrument:
Sample Collection Personnel: Killo William 11/1/66 Coopeny: Txco: 5 Finance Coopeny: Txco: 5 Fin

SAMPLE NUMBER: IN-2000- 28 Date: NOVEMBER 7. 1986
TOWN THE LECATION
Building Number: INTER-BUILDING OVERHEAD UTILITIES
NEAREST SUPPORT NO.: 38
SAMPLING ZONE
Construction Drawing 7500-19 Other 2nd from west
Other 2nd from west
MATERIAL TYPE
MATERIAL TYFE X Fipe Insulation Fipe Diameter: # inches Fipe Type: X Steam, Ethylene Glycol, Raffinate, Ceiling
Fipe Diameter:inches
Other
Ceiling
Floor Wall
Eciler Insulation
Other
COVER OVER MATERIAL
Fiber, Metal, Flaster, None, Other 129 Dapal Corte
DESCRIPTION OF MATERIAL
Color: White
Handness: Y Fibrous (Friable), Eranular (Soft), or Hand (Concrete-Like)
Thickness:inches
COMMENTS
Radiation Level
Background Only
Above Background com Instrument:
Simple Collection Personnel: 2000/18461 11/1/6/2 Company: Treet 5 E. H. Commander
1/2 miles in the second of the

SAMPLE NUMBER: IN-2000-29 Date: NOVEMBER 17, 1986
Equilding Number: INTER-BUILDING OVERHEAD UTILITIES Fig. NEAREST SUPPORT NO.: 562 Sec SAMPLING ZoNE Construction Drawing Other Chart one
MATERIAL TYFE Yeipe Insulation Fipe Diameter: Fipe Type: Steam, Other Other Figure 19501, Raffinate, Other
Ceiling Floor Wall Eoiler Insulation Other
Fiber, Metal, Flaster, None, Other TAR Pulzi Cole!
DESCRIPTION OF MATERIAL Color: Hardness: Fibrous (Friable), Eranular (Soft), or Hard (Concrete-Like) Thickness: inches
COMMENTS
Radiation Level Background Gnly Above Background cpm Instrument:
Sample Callection Personnel: Via Milina 11/17/26 Casesny: Tacots Engineer no

· · · · · · · · · · · · · · · · · · ·
SAMPLE NUMBER: IN-2000-30 Date: NOVEMBER 17, 1986
TAKE I DESTINA
SAMPLING LOCATION Building Number: INTER-BUILDING OVER HEAD UTILITIES
NEAREST SUPPORT NO.: 562
SHOW SAMPLING ZONE
Social Office Since
Construction Drawing 7500-24
Other Right Africa IV-2000-29 Sund Sout.
MATERIAL TYPE
Fipe Insulation (1)
Fipe Insulation Fipe Diameter:inches
Fipe Insulation Fipe Diameter: Fipe Type: Other Figure Insulation Fipe Type: Other Figure Inches Inches Ethylene Glycol, Eaffinate,
Floor Wall
Wall Boiler Insulation
Other
COVER OVER MATERIAL
Fiber, Metal, Flaster, None, Other TAR Dabu
DESCRIPTION OF MATERIAL
color: white
Hardness: Fibrous (Friable), Granular (Soft), or Hard (Concrete-Like)
Thickness: dinches
COMMENTS
Radiation Level 0.0 DPM
Fadiation Level (). O DPM\ Background Only
Above Background cpm Instrument:
Sanoie Collection Personnel: Vict Meur 1117/91 Capany: Tracks Elarge Elarge
- zimi

SAMPLE NUMBER:
SAMPLING LOCATION
Building Number: INTER-BUILDING OVERHEAD UTILITIES
NEAREST SUPPORT NO .: 562
SAMPLING ZONE 6.
Construction Drawing 7500-24
Construction brawing 7500-51
Other Tob further noth.
MATERIAL TYPE
Fipe Insulation Fipe Diameter: / inches Fipe Type: Steam, Ethylene Glycol, Raffinate,
Pipe Type: Steam, Ethylene Glycol, _^_ Raffinate, Other
Cailing
Floor Wall
Boiler Insulation
Other
COVER OVER MATERIAL
Fiber, Metal, Flaster, None, Other Metal Coul
DESCRIPTION OF MATERIAL
Color: White
Hardness: Fibrous (Frizble), Eranular (Soft), or Hard (Concrete-Like)
Thickness:inches
COMMENTS
CONTIENTS
Radiation Level
Background Only
Above Background cpm Instrument:
Sandle Collection Personnel: Vit Albin. 1117/66 Company: 1 Vacabs Ensincering
222

. ACTION PROJECT (WSSRAP) Charles, Missouri, 63303 Telex (314) 447-0803
OR ASBESTOS - FIELD DATA
Date: NOVEMBER 17, 1986
UILDING OVERHEAD UTILITIES
500-24 11au 32d-from EAST
How 3rd from EAST
υ L
inches Ethylene Glycol, Raffinate,
•
TAR paper (OR)
77.17
Granular (Soft), or Hard (Concrete-Like)
·

em_Instrument:
Cascany: Tagas Engineering

BULK MATERIAL SAMPLING FOR ASSESTED
SAMPLE NUMBER: IN-2000-33 Date: NOVEMBER 17, 1986
SAMPLE NOIDENT
Building Number: INTER-BUILDING OVERHEAD UTILITIES
NEAREST SUPPORT NO.: 56a
Figure NEAREST STITUTE TO STATE C
EDET SAMPLING ZONE C
Construction Drawing 7500-07
Other Ston Drawing Stone 1851-
Dune.
MATERIAL TYPE X Fipe Insulation Fipe Diameter: Steam, Ethylene Glycol, Raffinate,
Fipe Insulation 4. inches
Fipe Diameter: Steam Ethylene Glycol,Raffinate,
U CII C.
Ceiling Floor Wall Eciler Insulation
Floor
Wall Epiler Insulation
Other
COVER OVER MATERIAL
Fiber, Metal, Flaster, None, Other Tar tuta Cabi
DESCRIPTION OF MATERIAL
Franks (Soft) or Hard (Concrete-Like)
• • • • • • • • • • • • • • • • • • •
Thickness: 22 inches
COMMENTS
COMMENTS
Radiation Level Background Only
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Sample Collection Personnel: (1) (1) (1) (1) (1) (1) (2) (2) (2) (2) (2) (2) (2) (2) (2) (2

BULK MATERIAL SAME LINE 1 d. MODEL 1 de
SAMPLE NUMBER: IN-2000-34 Date: NOVEMBER 10, 1986
SAMPLE NUMBER:
Equilding Number: INTER-BUILDING OVERHEAD UTILITIES
Fuilding Number: INTEX-BUILDING OVER HELL
NEAREST SUPPORT NO .: 564
SAMPLING ZONE 7
Sec. JAMICING CO.
Construction Drawing 7500-24
Other Top Level - Farthest North
MATERIAL TYPE
X Fine Insulation U ¹
Fipe Diameter: inches
Fipe Insulation Fipe Diameter: Steam, Ethylene Glycol, Raffinate,
Uther
Ceiling
Floor
Wall Boiler Insulation
Other
Uchie.
COVER OVER MATERIAL
Fiber, Metal, Flaster, None, Other METAL 10047
Fibe: , Metas, Fibers,
DESCRIPTION OF MATERIAL .
Color: Whit
V and the state of
Thickness: 2 inches
COMMENTS Stalles stal pipe
COMMENTS SAN MISS
Radiation Level Background Only
con Instrument:
15-6 MM 15/61 - Finis 12/100
Saacla Collection Personnel: Vacation Company: Nacation Company: N

SAMPLE NUMBER: IN-2000-35 Date: NOVEMBER 18, 1986
SAMPLING LOCATION
Building Number: INTER-BUILDING OVERHEAD UTILITIES
NEAREST SUPPORT NO .: TROUMS (NEAR 406)
SAMPLING ZONE 8
Construction Drawing 7500 14.
MATERIAL TYPE Vertical Pipe at 5 W corner At Rail Loading Dock at 8 vilding -416
at 0
MATERIAL TYPE
X Fine Insulation /
Fipe Diameter:inches Fipe Type: Steam Ethylene Glycol,Raffinate,
Other_
Ceiling
Floor WalI
Wall Poiler Insulation
Other
COVER OVER MATERIAL Fixer, Metal, Plaster, None, Other Topoper
THE PROPERTY OF MATERIAL
DESCRIPTION OF MATERIAL .
Color: White
Hardness: Fibrous (Friable), Granular (Soft), or Hard (Concrete-Like)
Thickness:inches
COMMENTS
Radiation Level
Background Only
Above Background com Instrument:
Simple Collection Personnel: Kitch William 11/15/5/6 Company: Jacch & Engineering

BULK MHIERIAL DAM ZING V CONTRACTOR
TN 2000-36 Pate: NOVEMBER 18, 1986
SAMPLE NUMBER: IN-2000-36 Date: NOVEMBER 18, 1986
SAMPLING LOCATION Building Number: INTER-BUILDING OVER HEAD UTILITIES
Building Number: The Bolton Office
NEAREST SUPPORT NO .: 74
SAMPLING ZONE
Construction Drawing Other SAMPLING 20NL 7500-25 Top, Fartles + Eas+
Top, Farther tast
MATERIAL TYPE
Fipe Insulation ~ 4 inches
Fipe Insulation Fipe Diameter: inches Fipe Type: Steam, Ethylene Glycol, X Raffinate,
Utner
Ceiling Floor Wall Eoiler Insulation Other
Floor W=11
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Fiber, Metal, Flaster, None, Other TAR PARO (0/2)
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DESCRIPTION OF MATERIAL
Hardness: Kibrous (Friable), Granular (Soft), or Hard (Concrete-Like)
Thickness:inches
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COMMENTS
Radiation Level Background Only
Above Background, cpm Instrument:
1/ A N/ C - 11/19/9/ - Jacobs Envince 1/14
Sanole Collection Personnel: Will The Collection Personnel: Wi

SAMPLE NUMBER: IN-2000-37 Date: NOVEMBER 18, 1986
SAMPLE NUMBER:
Building Number: INTER-BUILDING OVER HEAD UTILITIES
Building Number: 211222110
NEAREST SUPPORT NO .: 754
SAMPLING ZONE 9
Construction Drawing 7500-35
Construction Drawing 7500-25 Other Top- 2nd Tom Fast
MATERIAL TYPE
Fipe Insulation ~ 6 % inches
Fipe Insulation Fipe Diameter: ~ 4 inches Fipe Type: Steam, Ethylene Glycol, Y Raffinate, Other
Ceiling
Floor Wall
Boiler Insulation
Ctner
COVER OVER MATERIAL Fiber, Metal, Flaster, None, Other
DESCRIPTION OF MATERIAL
Color: WLD .
Hardness: Fibrous (Friable), Granular (Soft), or Hard (Concrete-Like)
Thickness: _3 inches
COMMENTS Stayless Stel Pile
Radiation Level Background Only
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Sample Collection Personnel: duri // Company: \acons tagin en // Company: \acons tagin en // Company: \acons tagin en // Company:

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iPRING SITE REMEDIAL ACTION PROJECT (WSSRAP) ite 2, Highway 94, St. Charles, Missouri 63303 Fhone (314) 441-8086 Telex (314) 447-0803
ILK MATERIAL SAMPLING FOR ASBESTOS - FIELD DATA
IN-2000-38 Date: NOVEMBER 10, 1986
IN INTER-BUILDING OVERHEAD UTILITIES
SAMPLING ZONE 9
ruction Drawing 7500-35 Elbour Tob level to Hast East
Insulation ipe Diameter: Steam, Ethylene Glycoi, Raffinate, Other Other
ng
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Flaster, None, Other TER HARL (OV2)
, Flaster, Mene, Dene.
MATERIAL
Fibrous (Friable), Eranular (Soft), or Hard (Concrete-Like)
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round Gniy Reground Instrument:
77.610

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SAMPLE NUMBER: IN-2000-39 Date: NOVEMBER 18, 1986
SAMP! ING LOCATION
Building Number: INTER-BUILDING OVERHEAD UTILITIES
NEAREST SUPPORT NO .: 764
SAMPLING ZONE
Construction Drawing 7500-25
Other Show - Smill Section of Rebail
MATERIAL TYPE
Fipe Insulation
Fipe Insulation Fipe Diameter: N L inches Fipe Type: Ethylene Glycol, Kaffinate, Other
Ceiling Other
Floor
Wall Poiler Insulation
Direr insufactor Other
COVER OVER MATERIAL
Fiber, Metal, Plaster, None, Other JAR Para Love
DESCRIPTION OF MATERIAL
Color: Jellon
Hardness: Fibrous (Friable), Granular (Soft), or Hard (Concrete-Like) Thickness:inches
Thickness: 22 inches
COMMENTS FUEL CLASS
Radiation Level Background Only
Above Eackground com _Instrument:
Sample Collection Personnel: Live Moren 1/10/0/ Company: Jacobs Engineering

BULK MATERIAL SAMPLING TOR MODEL TO
SAMPLE NUMBER: IN-2000-40 Date: NOVEMBER 18, 1986
SAMPLING LOCATION Building Number: INTER-BUILDING OVER HEAD UTILITIES
Building Number: 27 20 C21
NEAREST SUPPORT NO .: Soly
SAMPLING ZONE 10
Construction Drawing 7500 © O
1 -118
Other C 1884 14 1 1100
MATERIAL TYPE
X Pipe Insulation
Fipe Insulation Fipe Diameter: inches Fipe Type: Steam, Ethylene Glycol, Raffinate,
Other
Ceiling
Floor
Wall Table 5
Poiler Insulation Other
Octre:
COVER OVER MATERIAL
Fiber, Metal, Plaster, None, Other tak had (ove)
DESCRIPTION OF MATERIAL
11/2
Color: X Fibrous (Friable), Granular (Soft), or Hard (Concrete-Like)
Thickness: 3 inches
COMMENTS Stall Pipe
COMMENTS
Radiation Level Background Only
Above Background/cpm instrument:
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Sample Collection Personnel: (AMC) 11/10/10/10 Company: Vicionia Company: Vicionia Company: Vicionia Vicionia

WELDON SPRING SITE REMEDIAL ACTION PROJECT (WSSRAP)
Route 2, Highway 94, St. Charles, Missouri 63303
Phone (314) 441-8086 Telex (314) 447-0803

BULK MATERIAL SAMPLING FOR ASBESTOS - FIELD DATA

SAMPLE NUMBER: IN-2000-41 Date: NOVEMBER 16, 1986
SHIT LE NO. 12 - 1
EAMPLING LOCATION Building Number: INTER-BUILDING OVERHEAD UTILITIES ELL NEAREST SUPPORT NO.: SL Second SAMPLING ZONE Construction Drawing Other Other T500-20 Steanth Section of Small Pipe
MATERIAL TYPE X Fipe Insulation Fipe Diameter: inches Fipe Type: Steam, Ethylene Glycol,Raffinate, Other
Ceiling Floor Wall Eoiler Insulation Other
COVER OVER MATERIAL Fiber, Metal, Flaster, None, Other Tar Paper (
Color: Hardness: Thickness: Thickness: DESCRIPTION OF MATERIAL Solution Fibrous (Friable), Granular (Soft), or Hard (Concrete-Like) Thickness:
COMMENTS
Radiation Level Background Only Above Background Com Instrument: Above Background Com Instrument: Above Background Com Instrument: Above Background Com Instrument:
Sample Collection Personnel: Row 1/100 m 11/10/10 Concany: JACOM Concany:

WELDON SPRING SITE REMEDIAL ACTION PROJECT (WSSRAP) Route 2, Highway 94, St. Charles, Missouri 63303 Phone (314) 441-8086 Telex (314) 447-0803

BULK MATERIAL SAMPLING FOR ASBESTOS - FIELD DATA /8 SAMPLE NUMBER: IN-2000 - 42 Date: NOVEMBERT, 1986 SAMPLING LOCATION Building Number: INTER-BUILDING OVERHEAD UTILITIES NEAREST SUPPORT NO : 64 SAMPLING ZONE 7500- 20 Construction Drawing Other MATERIAL TYPE X Fipe Insulation Fipe Diameter: 18 inches Fipe Type: ___ Steam, ___ Ethylene Glycol, ___ Raffinate, ____Other__ Ceiling Floor Wall _ Boiler Insulation Other ____ COVER OVER MATERIAL Fiber, Metal, Flaster, None, Other Turpaper DESCRIPTION OF MATERIAL Color: Fibrous (Friable), ____ Granular (Soft), or ____ Hard (Concrete-Like) Hardness: <u>'</u>inches Thickness: COMMENTS Radiation Level ___ Background Only ____cpm Instrument:__ Above Eackground 1/12/3/n company: Jacobs Engineerie Sample Collection Personnel:

Appendix C

Radiological Contamination of Samples Collected to Measure
Asbestos Content of Pipe Insulation at the WSCP

2.06 VEHICLE/EQUIPMENT RELEASE FOR UNNESTRICTED USE CONTAMINATION SURVEY FORM

Property 11_MA	14	Date	Data 11/19/86	36	발	HP C.J. AUSTOW	Ma			•
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DEPARTMENT OF ENERGY	OAK RIDGE, TENNESSEE		CONSTRUCTION DRAWINGS	EXISTING SITE PLAN
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J. S. DEPARTMENT OF ENERGY OAK RIDGE, TENNESSEE

CHEMICAL PLANT	CONSTRUCTION DRAWINGS	OVERHEAD PIPING	NORTH (SHEET 10F2)	DATE GHEF ENG, TO A MOR DATE PING ENGIG MGR DATE DOF PROJ ENG
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cc: D. R. Lewis

R. A. Nelson

A. J. Stewart

UNITED STATES ENVIRONMENTAL PROTECTION AGENCY

3589.87.I.EAS.010

NOVS 1987

Mr. Rodney R. Nelson U.S. Department of Energy Weldon Spring Site Remedial Action Project/Office Route 2, Highway 94, South St. Charles, Missouri 63303





We have reviewed the Department of Energy's (DOE) proposals for the following four interim response actions:

- Electric Power and Pole Removal,
- Overhead Piping/Asbastos Removal,
- Cleanup of Vicinity Property No. 7 on the Army Reserve Area, and
- Disposal of Containerized Chemicals.

Our comments on these proposals were sent to you earlier. You were also provided comments by the Missouri Department of Natural Resources (MDNR). No comments from the public were directed to the Environmental Protection Agency (EPA) and according to our records, there has been no public comment directed to MDNR or DOE.

We are in agreement these actions should proceed to ensure worker safety and reduce the further release of contaminants from this site. The EPA hereby approves these actions under the condition that the comments earlier provided by EPA and MDNR are adequately addressed. The MDNR has notified me they also concur with these actions. Please provide copies of any summary reports for these actions to EPA and MDNR.

FILE NUMBER:

We also received copies of the following four interim response actions:

- Dismantling of Building 401,
- Dismantling of Building 409, Removal of PCB Transformers, and
- Debris Consolidation.

My. Cr. Cr. Cr

We will provide any comments on these within the agreed upon 21-day comment period. We are most pleased to see that activities are underway to stabilize the site and reduce contaminant release.

Sincerely yours,

Morris Kay Regional Administrator

cc: Dr. Fred Brunner, MDNR

bc: Robert Morby

Dan Shiel

Rowena Michaels

Ron Ritter

NOV 1 0 1987

Ms. B. Katherine Biggs
United States Environmental
Protection Agency
Region VII
726 Minnesota Avenue
Kansas City, Kansas 66101

Dear Ms. Biggs:

USEPA COMMENTS ON INTERIM RESPONSE ACTIONS (IRA'S)

Enclosed is our response to the comments contained in your letter of October 8, 1987, regarding the following interim response actions:

- 1. Electric Power Line and Pole Removal
- -2. Overhead Piping/Asbestos Removal
 - 3. Army Reserve Area Vicinity Property No. 7
- 4. Disposal of Containerized Chemicals

We anticipate that this will adequately resolve the issues raised. We intend to proceed with action on these items in accordance with the enclosure.

If you have any questions, please give me a call.

Sincerely,

ORIGINAL SIGNED BY:

R. R. Nelson Project Manager Weldon Spring Site Remedial Action Project

Enclosure:
As stated

Cc: D. Bedan, MDNR
E. Brown, FLW
W/enclosure

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RESPONSES TO USEPA REVIEW COMMENTS ON IRA PACKAGES

Cleanup of Vicinity Property No. 7, Army Reserve Area

Comment: The proposal to cleanup this vicinity property calls for excavating the contaminated area to a depth of six (6) inches or where the radium concentration is below 15 pCi/g and then backfilling with clean material. The EPA suggests that while its criteria for residual radium in soil is satisfied by this approach, that we consider removing additional soil to reduce the concentration to below 5 pCi/g. As the area is small, little additional excavation would be required.

Response: Subsequent to the preparation of the IRA package for Army Reserve Vicinity Property #7, the Department of Army requested that the area not be backfilled upon completion of the cleanup as proposed by the DOE. The DOE will leave the excavation area open and apply the suface criteria of 5 pCi/g to this particular vicinity property.

Disposal of Containerized Chemicals

Comment: It is suggested that the specifications for this work might be strengthened by adding waste characterization procedures into Section 2.0 (Scope) of the document. The procedures are those which may be required under 40 CFR 260-268, or others required by the permit held by the Treatment, Storage and/or Disposal Facility.

Response: Procedures required in 40 CFR 260-268 will be referenced in Section 2.0 of the Request for Proposal. In addition, it will be emphasized that all waste characterization procedures which are required by the successful bidder's treatment, storage and/or disposal facilities permit must be satisfied. It will be required that these procedures (if applicable) be presented in the subcontractor's work plan.

Comment: EPA recommends that the specific subcontractor qualifications and experience in handling known and unknown potentially hazardous wastes be defined in the document.

Response: We are in agreement with the EPA that the

Request for Proposal should contain subcontractor

qualifications and experience clauses. The

appropriate clauses will be added to the document.

Comment: The EPA recommends that the specification require

the successful bidder to identify the specific waste disposal facilities which will accept the containerized chemical waste, in the work plan

phase.

Response: The specification will be modified to include

provision for certification by the subcontractor

that the waste disposal facilities meet the

requirements when hazardous wastes are involved. The land disposal ban provision of RCRA will also be addressed as part of the subcontractor's work

plan.

General

Comment: The EPA review states that there is one(1)

deficiency common to the four proposals and that is that plans for onsite handling and storage of radioactive contaminated materials should be

developed.

Response: Plans for onsite handling and storage of

radioactive contaminated materials are currently

being finalized and will be provided under

separate cover.

NGV 10 1987

Mr. David E. Bedan
Missouri Department of
 Natural Resources
Post Office Box 176
Jefferson City, Missouri 65102

Dear Mr. Bedan:

MISSOURI DNR COMMENTS ON INTERIM RESPONSE ACTIONS (IRA'S)

Enclosed is our response to the comments contained in Dr. Frederick A. Brunner's letter of October 26, 1987, regarding the following interim response actions:

- 1. Electric Power Line and Pole Removal
- 2. Overhead Piping/Asbestos Removal
- 3. Army Reserve Area Vicinity Property No. 7
- 4. Disposal of Containerized Chemicals

We anticipate that this will adequately resolve the issues raised and we intend to proceed with these actions in accordance with the enclosure.

If you have any questions, please give me a call.

Sincerely,
ORIGINAL SIGNED BY:
R. R. NELSON

R. R. Nelson
Project Manager
Weldon Spring Site
Remedial Action Project

Enclosure: As stated

cc: B. K. Biggs, USEPA
E. Brown, FLW
w/enclosure

FILE NUMBER:	
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PEER:JCoyne:x41:mw:11/9/87: (c:DNRCom.Ltr.)

CONCURRENCES

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RESPONSES TO MDNR COMMENTS ON INITIAL FOUR (4) IRA PROPOSAL PACKAGES

I. Removal of Overhead Piping and Asbestos Removal

Comment: The DNR states that Missouri has adopted the Federal Clean Air Act standards for asbestos handling and has been delegated responsibility for implementing these standards and that asbestos and piping removal activities are subject to both the Missouri Air Conservation Law and the Missouri Solid Waste Management Law. DNR recommends that we maintain close contact with the Air Pollution Control Program to insure compliance with these standards.

Response: DNR Air Pollution Control Program office will be kept apprised of plans for asbestos removal work at the Weldon Spring Site. The WSSRAP will comply with requirements for disposal of asbestos and other demolition wastes in accordance with the Missouri Solid Waste Management Act.

Comment: The DNR has determined that the overhead piping and asbestos should be handled as a "special waste".

Response: We are proceeding to include the Special Waste Disposal Request form in the Request for Proposal for this work.

Comment: The DNR states that onsite handling of asbestos and other demolition waste <u>may</u> also be subject to Missouri Solid Waste Management Law requirements and requests that we furnish information on the size, design, location of the staging area and the amounts and methods of handling for the materials to be handled in the materials staging area.

Response: Information on handling and staging of the materials will be furnished to the DNR prior to issuing requests for proposals for this work.

modnrrsp,txtsheil

II. Disposal of Containerized Chemicals

Comment: Define specific levels at which the containerized wastes are considered radioactive.

Response: WSSRAP is developing concentration levels for wastes containing natural uranium for review and acceptance by concerned federal agencies. We will advise the DNR of this determination as it comes available. Until this determination is made we will retain on site containerized chemical materials which contain detectable levels of radioactive materials as determined by our onsite instruments.

Comment: What are removal plans for underground storage tanks on site?

Response: The underground tanks at the WSS have been sampled and found to contain only rainwater with trace amounts of motor fuel. The drainage and removal of the underground tanks is not part of the containerized chemical inventory and removal IRA Scope of Work. They may be removed as part of a subsequent IRA.

Comment: The document appears to be a generic outline for removal of waste. Items such as disposal facilities, transporters, waste characterization procedures, waste treatment procedures, etc. are not detailed.

Response: The IRA documentation represents a request for proposal to be sent to potential removal subcontractors. The responsibility for developing a detailed work plan addressing such items as disposal facilities, transporters, waste characterization procedures, waste treatment procedures, etc. rests with the successful bidder. requirement of the subcontract specification is the development of the subcontractor's work plan which must be approved by the WSSRAP before the work may begin. The DNR will be provided a copy of the subcontractor work plan when it becomes available for review. The WSSRAP office requests that the State provide a timely review (14 calendar days) to avoid delaying the subcontractor's

efforts. The State will be given 2-weeks notice of the interval at which time the work plan will be available for review.

III. Remedial Action on Army Vicinity Property

Comment: DNR states that the interim measure suggested is lacking in detail and should contain information on:

- How the removal is to be conducted?
- How and where excavated material will be contained?
- Health and safety plans for the work.
- Why is DOE recommending only remedial action for this vicinity property?
- When does DOE plan to remove the additional contamination in the other six locations?

Response: The technical requirements, i.e.

specifications, drawings, special

conditions, etc., did not accompany this IRA

package for review as they were incomplete

at the time of package submittal.

Requirements for removal, containment and

storage, and health and safety plans will be
included in the Request for Proposal for
this work.

The very small quantity (less than two cubic yards) of contaminated material can be removed manually and will not require any significant mechanical equipment. The technical specification developed for this work will provide required direction to the subcontractor.

We plan to excavate and place the contaminated soil in 55 gallon drums. The drums will be sealed, placed and stored in a dry, concrete floored building at the Weldon Spring Site awaiting final disposition. Total volume is anticipated to be less than 5 drums.

No unusual hazards are anticipated for this activity. The contract will require workers to abide by the WSSRAP Environmental, Safety and Health Plan.

DOE proposes performing remedial action of Army Vicinity Property No. 7 to remove contamination from an area where the Army has imminent construction plans. Cleanup of other vicinity properties, containing larger volumes of contaminated material, will be recommended based upon urgency of cleanup needs and development of storage facilities on the WSS.

Remediation of the remaining Army Vicinity Properties is tentatively scheduled for the first and second quarters of fiscal 1989.

IV. Power Line/Pole Removal

Response: The plan should address PCB contamination in regard to pole removal if transformers/capacitors containing PCB's were mounted on them.

Comment: The power poles to be removed in the IRA were specifically not associated with transformers containing PCBs. Those poles which have oil-cooled transformers mounted on them will be surveyed and removed at a later date.

INTERIM RESPONSE ACTION (IRA) ADMINISTRATIVE RECORD FILE ARFS FILE # IR-0500

TABLE OF CONTENTS

SECT	ION SECTION TITLE	
0500	IRA #07 CONTA	INERIZED CHEMICALS
	DOCUMENT NUMBERS	DOCUMENT TITLE
501	Sampling and I	Analysis Plans
	IR-0500-501-1.01	DRAFT WSCP CONTAINERIZED WASTE ASSESSMENT AND REMOVAL PLAN
	IR-0500-501-1.02	WASTE ANALYSIS PLAN
502	Sampling and A	Analysis Data/Chain of Custody Forms
503	Engineering Ev	valuations/Cost Analysis
	IR-0500-503-1.03	CONTAINERIZED WASTE HANDLING TRANSPORT & DISPOSAL SUMMARY
504	Engineering E Document	valuations/Cost Analysis Approval or Decision
504	5 -	valuations/Cost Analysis Approval or Decision EPA APPROVAL OF DISPOSAL OF CONTAINERIZED CHEMICALS
504	Document	EPA APPROVAL OF DISPOSAL OF CONTAINERIZED
	Document IR-0500-504-1.01 IR-0500-504-1.02 IR-0500-504-1.03	EPA APPROVAL OF DISPOSAL OF CONTAINERIZED CHEMICALS RESPONSIVENESS SUMMARY TO EPA COMMENT ON CONTAINERIZED CHEMICALS RESPONSIVENESS SUMMARY TO MDNR COMMENT ON CONTAINERIZED CHEMICALS
504	Document IR-0500-504-1.01 IR-0500-504-1.02 IR-0500-504-1.03	EPA APPROVAL OF DISPOSAL OF CONTAINERIZED CHEMICALS RESPONSIVENESS SUMMARY TO EPA COMMENT ON CONTAINERIZED CHEMICALS RESPONSIVENESS SUMMARY TO MDNR COMMENT ON
	Document IR-0500-504-1.01 IR-0500-504-1.02 IR-0500-504-1.03	EPA APPROVAL OF DISPOSAL OF CONTAINERIZED CHEMICALS RESPONSIVENESS SUMMARY TO EPA COMMENT ON CONTAINERIZED CHEMICALS RESPONSIVENESS SUMMARY TO MDNR COMMENT ON CONTAINERIZED CHEMICALS
	Document IR-0500-504-1.01 IR-0500-504-1.02 IR-0500-504-1.03 Amendments to	EPA APPROVAL OF DISPOSAL OF CONTAINERIZED CHEMICALS RESPONSIVENESS SUMMARY TO EPA COMMENT ON CONTAINERIZED CHEMICALS RESPONSIVENESS SUMMARY TO MDNR COMMENT ON CONTAINERIZED CHEMICALS Decision Document MODIFICATION TO THE CONTAINERIZED CHEMICALS EE/CA

IRA-500-501

WELDON SPRING CHEMICAL PLANT

CONTAINERIZED WASTE ASSESSMENT

AND REMOVAL PLAN

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PREPARED FOR

THE DEPARTMENT OF ENERGY

PREPARED BY:

WELDON SPRING SITE REMEDIAL ACTION PROJECT

MK-FERGUSON COMPANY

AND

JACOBS ENGINEERING GROUP INC.

CONTRACT NO. DE AC05-860R21548

File: * Weidon Spring Subfile: F.1/F5/E15 No 20.1

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Appendix A - Chemicals Used at the Weldon Spring Feed Materials Plant

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1.0 INTRODUCTION

The Weldon Spring Site (WSS) is a U.S. Department of Energy (DOE) surplus facility located in St. Charles County, Missouri (Figure 1). The site presently consists of the Weldon Spring Raffinate Pits (WSRP), the Weldon Spring Quarry (WSQ), the Weldon Spring Chemical Plant (WSCP), and properties in the vicinities of these areas that have become contaminated as a result of operations at the WSS.

On October 1, 1986, the MK-Ferguson and Jacobs Engineering Team assumed responsibilities as Project Management Contractor (PMC) for the Weldon Spring Site Remedial Action Program (WSSRAP). The PMC for WSSRAP is also responsible for completion of characterization of the WSCP area.

The WSCP will be characterized to determine its physical properties and to determine the magnitude and distribution of chemical and radioactive contamination at the site. This information will play a key part in DOE's ultimate decision regarding decontamination and decommissioning of the WSS and disposition of its wastes.

1.1 Purpose

The purpose of this containerized waste assessment and removal program is to address the identification and disposal of containerized wastes that currently exists at the WSCP. This program is a necessary prerequisite to the overall implementation of the WSSRAP.

The intent of this program is two fold: 1) to mitigate the potential for sudden or unexpected release to the environment

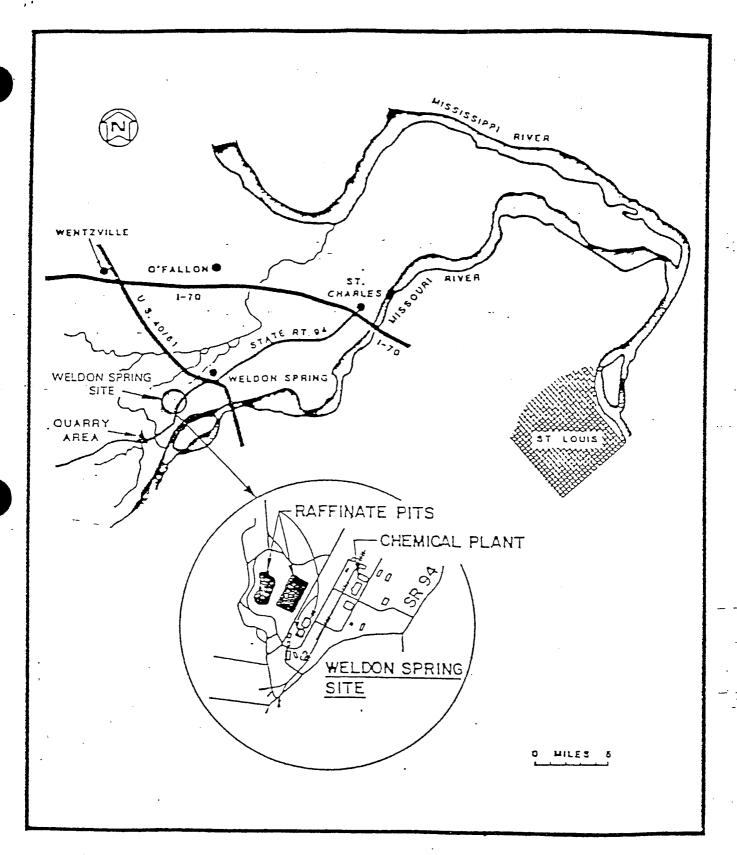


FIGURE 1 LOCATION OF WELDON SPRING CHEMICAL PLANT

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resulting from improper container storage, continued container degradation, or increased site remedial activities which may increase the potential for such releases; and 2) to identify and remove those materials which create potential exposure risks to the health and safety of site personnel during subsequent WSSRAP activities.

1.2 Scope

The containerized waste assessment and removal program is focused on those materials remaining in containers. These materials include: liquids, solids, sediments and sludges currently contained in surface and subsurface storage tanks, process vessels, sumps, drums, fuel tanks, laboratory containers, gas cylinders, fire extinguishers, batteries and other miscellaneous containers.

All materials identified above will be designated as waste materials and will be considered a hazardous substance until such time as the preliminary characterization and/or individual container sampling preclude their classification as a hazardous substance as defined by 40 CFR 300.6 under the Comprehensive Environmental Response, Compensation and Liabilities Act (CERCLA), or a hazardous waste under 40 CFR 261 The Resource Conservation and Recover Act (RCRA).

Furthermore, those containerized wastes which are identified as having radiologic activity above background levels will be segregated and placed in an interim storage area onsite for treatment and disposal with other radiologically contaminated materials from the WSS.



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1.3 Objectives

The objective of this program is to inventory, identify, segregate, package, transport, and dispose of all non-radiologically contaminated hazardous wastes in an environmentally acceptable manner consistent with applicable federal, state and local regulations. This program has the further objective of determining which, if any, of the radiologically contaminated materials are also hazardous wastes under RCRA. The major factors which will be considered in determining the feasibility and effectiveness of various treatment/disposal options will be focused toward permanent alternatives consistent with EPA's proposed alternative treatment/disposal technology guidance for removal and expedited response actions (EPA, September 1986).

2.0 SITE DESCRIPTION

The WSCP includes 13 major buildings and approximately 30 smaller buildings. Of the 13 major buildings, five were used as process buildings, and eight were major support buildings. Most of the buildings are shown and identified in Figure 2. The other two buildings (4-35, 4-3) are just beyond the southern boundary of this Figure.

The WSCP site encompasses a portion of what was previously the Weldon Spring Ordnance Works (WSOW), used to manufacture DNT and TNT between 1941 and 1944. In 1956, the Atomic Energy Commission acquired approximately 220 acres of the original WSOW site from the Department of the Army (DA) for use as a uranium feed materials plant (DA, March 1976).

Following shutdown of the Feed Materials Plant in 1966, the AEC

FIGURE 2 MAJOR STRUCTURES AT WSCP SITE

transferred the facility to the DA for conversion to a defoliant production plant. The DA hauled approximately 6,000 cubic yards of radiologically contaminated materials to the WSQ and also dumped an undetermined amount of contaminated scrap material into Raffinate Pit 4 (DA, March 1976). Work on the defoliant project was canceled in 1969 before renovation of facilities was complete; therefore, herbicide was not produced at this site.

In 1983, the DA containerized the removable, radiologically-contaminated material from most of the buildings and appurtenant structures into 55 gallon drums these drums were placed in rows inside Building 301 and 406. The contaminated materials and machinery inside Building 301 were covered with a thin fabric and sprayed with a 1-inch-thick layer of hard-setting foam to prevent the spread of contamination as a result of rain leaking into the building (SNC, April 1984).

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3.0 TECHNICAL APPROACH

This containerized waste assessment and removal program will be performed in two phases. Phase I will involve two activities, container inventory and chemical characterization. These preliminary activities will be performed by the PMC. Information gathered in this first phase will be utilized to develop subcontractor bid specifications and other documents necessary to award a subcontract to perform Phase II activities. Phase II activities may include: site preparation for container staging, container staging, container staging, waste bulking, manifest preparation and transportation/treatment/disposal of containerized materials.

All work performed under these activities will be subject to and controlled by DOE approved WSSRAP plans including:

Environmental, Safety, and Health Plan for the Weldon Spring Site Remedial Action Project;

Quality Assurance Program Plan for the Weldon Spring Site Remedial Action Project;

Community Relations Plan for the Weldon Spring Site Remedial Actions Project;

WSSRAP Spill Prevention, Control and Countermeasure Plan, and

Applicable DOE Orders.

Activity specific operation plans and standard operating procedures will be developed and implemented by the PMC for the Phase I activities and by the removal subcontractor for Phase II activities to support the unique activities which will be performed during the container assessment and removal operations.

Details of each activity are presented in the following sections.

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3.1 PHASE I

Phase I will consist of container inventory and characterization steps.

3.1.1 <u>Inventory of Containerized Chemical Waste</u>

The container inventory will be the initial activity performed to identify and estimate the number, location, general condition, storage configuration, and sampling method required for WSCP containers. During the inventory, the indoor and surrounding area of each building will be surveyed for the presence of bottles, drums, tanks and other vessels. As containers are identified, their locations will be recorded on base maps. Additionally, each container, or group of containers, will be described on a container inventory form (Figures 3 and 4) as to the type of container(s), capacity, physical condition, markings, etc. and as to the type of material (liquid, solid, gas or sludge). Each container or group of containers will be assigned and labeled with a unique identifying number. When more than one identical container is present at one location, the entire group will be assigned a single number. Like containers will be physically grouped together and suitably labeled.

During the inventory, the potential hazards to on site personnel associated with the waste materials will be assessed using radiologic survey instruments, organic vapor survey meters, and combustible gas/oxygen meters.

3.1.2 <u>Identification of Container Contents</u>

Concurrent with inventory activities, efforts will be undertaken to identify the waste materials. Based on a site survey performed by PMC personnel in April 1987, approximately

FIGURE 3

Weldon Spring Site Remedial Action Project (WSSRAP) Route 2, Highway 94 South, St. Charles, Missouri 63303 Phone (314) 441-8086 Telecopy (314) 447-0803

Containerized Chemical Inventory

Container Number: CC-2 -0587 Date	e:
Number of Containers Included In This Number:	
LOCATION WHERE FOUND: CONTAINER T	YPE:
Building No.: Drum	- Open Top
Floor No.: Drum	- Bung Top -
Room Name/No.:Fiber	Drum
Can -	5 gallon
Drawing No.: Can -	1 gallon :
LOCATION MOVED TO: Tank	- Above Ground
Staging Area No.: Tank	- Below Ground
Staging Area Name: High I	Pressure Tank
Gas C	ylinder
CONTENTS IDENTIFICATION: Bag	
Definitely Known Box	
Tentatively Known Sump	
Unknown Pipe	
QUANTITY OF MATERIAL: Bottle	e
Gallons: Jar	
Pounds: Other	
CONTAINER INTEGRITY:	
Over-Pressurized Leaking R	usted Through
Other	
ORIGINAL LABEL WORDING:	
PERSONNEL HAZARD LEVEL SURVEY DATA:	
PID Reading: ppm calibrated to	
Oxygen Concentration: percent	. .
Combustible Gas Level:% of LEL cal	ibrated to
Radiation Levels: alpha: beta:	gamma:
Other:	
INVENTORIED BY:	-
Signature: Employer:	Jacobs Engineering
Form CC-1	
MK-Fergison Company Project Managem	ent Contractor

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two-thirds of the containers onsite are known to be labeled or the contents can be identified based on process drawings or knowledge of the process. Many of the containers are unopened and have intact labels. Appendix A lists the chemicals associated with processing activities that are potentially still present at the site.

During this identification step, all containers will be screened qualitatively in situ for radiologic activity. This screening will be performed using both alpha and beta/gamma radiation survey meters. The results of this screening will be used only to determine whether the material is grossly contaminated. It will not be used to determine whether materials are releasable offsite.

The contents of all containers determined to be radiologically contaminated during the above screening procedure will be sampled. These samples will be quantitatively analyzed onsite using gamma spectroscopy and offsite for RCRA characteristic testing. The radiologic activity will be determined onsite using one-hour counting times in the High Purity Germanium Gamma Spectroscopy System capable of detecting uranium-238, thorium-230 and radium-226. Other radioactive isotopes will not be measured since these materials are not present on site. The RCRA characteristics test will include EP-toxicity, ignitibility, reactivity, and corrosivity. These tests will be performed to determine whether the radioactive wastes from the site are also RCRA characteristic wastes.

A random sampling of approximately 10 percent of the remaining known and unknown materials which are not radiologically contaminated will be performed to confirm the label markings and physical appearance. The same quantitative tests for RCRA characteristics will be performed on those samples as were

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performed on the radiologically contaminated samples. Collection of samples for these various parameters will be documented on the form shown in Figure 5.

This characterization will allow gross categorization of the types of waste on-site. This information will be used to prepare bid specifications, cost estimates, and to estimate the level of effort for the remaining removal activities. Costs for the removal phase are extremely variable depending on waste types, required overpacking, recontainerization, potential for bulking, distance to an acceptable disposal site, and personnel protective equipment required for safe handling. This sampling will also expand the general knowledge of the WSCP and provide information needed for worker protection.

The appropriate sampling techniques will be determined by the types of material, and any physical constraints or limitations determined during the field assessments. Sampling methods and equipment will vary with the physical form and medium of the sample. In each case, the sampling and collection procedures will be consistent with those identified in EPA guidance and methodology documents (EPA, December 1984, May 1985, and January 1986).

After completion of the container inventory and characterization, the information will be summarized into a graphical, tabular, or matrix formats. This information will be utilized in the development of the subcontractor bid specifications for containerized waste removal and disposal addressed under the following activities.

3.2 Phase II

The second phase will be performed by the subcontractor who is -11-

FIGURE 4

Form CC-4



Weldon Spring Site Remedial Action Project (WSSRAP)
Route 2, Highway 94 South, St. Charles, Missouri 63303
Phone (314) 441-8086 Telecopy (314) 447-0803

Condemned Fire Extinguisher Inventory Building or Area Number Floor Type Of Extinguisher Number (Hash Marks) Total Dry Chemical, Red (AB or ABC) 10 pound 30 pound 350 pound (Wheeled) Dry Chemical, Yellow (for metal fires) 30 pound 150 pound (Wheeled) Carbon Dioxide 5 pound 10 pound 15 pound 20 pound Propellant Cartridge for Dry Chemical Extinguisher Ansul Other Other Types Inventoried By (Signature):_____ Date

MK-Ferguson Company -- Project Management Contractor

FIGURE 5

Weldon Spring Site Remedial Action Project (WSSRAP)
Route 2, Highway 94 South, St. Charles, Missouri 63303
Phone (314) 441-8086 Telecopy (314) 447-0803

JOHUAL	nerized Chemic	al Sampling	
Container Number:	CC-2	-0587	
PHYSICAL DESCRIPTION		•	
Solid	Liquid	Sludge -	Gas
Homogeneous	S	tratified	
Color:			·
Viscosity:			
Other:	_ _ - -		
. Jones .			
Described By: (Si	gnature)		
	gnature)		
Described By: (Si			
Described By: (Si	Compatability		
Described By: (Si Sample Type Date Collected	Compatability	Radiologic Yes or No	Chemical I.I
Described By: (Si Sample Type Date Collected Duplicate Collected Separate Phases to Be Analyzed	Compatability Yes or No	Radiologic Yes or No	Chemical I.I

Form CC-2
MK-Ferguson Company -- Project Management Contractor

the successful bidder on that portion of the project. The bid specifications for Phase II will be prepared as part of Phase I.

3.1.1 Site Preparation

Before container removal actions begin, preparation of support facilities will be required to provide for safe and efficient handling of the containers. Based on the Phase I findings appropriate support facilities will be designated and/or installed by the removal subcontractor. The nature and extent of site preparation will vary depending on the hazard level of wastes, their locations, etc. The number and location of support facilities, special operating areas, or temporary storage areas needed for the removal operations may include:

- o Staging areas,
- o Drum opening area(s),
- o Waste consolidation/loading areas,
- o Interim storage areas,
- Equipment and personnel decontamination areas,
- o Mobile laboratory,
- o Command post and administration area,
- Emergency medical facilities, and
- o Equipment maintenance area.

The site layout will be such that there is a safe travel distance between staging areas, opening/sampling areas, temporary storage areas, etc. Container inventory and characterization data gathered during Phase I will be used to determine the number and size of the various operating areas.

Highly hazardous containers such as those containing explosives, highly radioactive materials, pressurized drums, compressed gas

cylinders, etc. may require separate staging/opening areas located as far as possible from the routine container handling operations and other site operations.

3.1.2 Staging

Any containers which were identified in the inventory and characterization activities which were leaking, badly corroded, or deformed will be overpacked or the contents transferred to a new or reconditioned container as soon as the removal subcontractor is on-site and site preparations are complete.

Once any immediate hazard(s) have been eliminated, removal activities will begin.

A container will not be moved or opened unless it has been ascertained beyond a reasonable doubt that the container is structurally sound, and the contents are not explosive or shock sensitive.

Staging of drums and other movable containers will be considered when the following conditions exist:

- Containers cannot be sampled in place due to physical restrictions,
- O Potentially hazardous materials are stored together (i.e., storage of incompatible or reactive materials next to each other),
- o There are severe access/egress restrictions,
- o Drums can be grouped by waste type to facilitate bulking,
- Containers are pressurized, contain compressed gas, or exhibit unusual conditions.

During staging, the containers will be physically separated into the following categories (when possible): liquids, solids, laboratory materials/packs, gas cylinders, and radioactive wastes.

The use of combined container staging/container opening areas, will be used when possible where container contents are known or suspected, (i.e., similar drum markings, labels, preliminary characterization indicate similar contents, etc.). Containers will be staged in rows of two or in groups of four, with sufficient distance between each row or group to provide easy access for drum opening equipment and adequate space for emergency evacuation.

3.1.3 Sampling for Compatibility and Disposal

A detailed work plan will be developed by the selected subcontractor and will be reviewed and approved in writing by the PMC prior to the start of on-site sampling activities. Generally, a typical plan for tank and drum sampling will address the following topics:

- Goals of the sampling effort,
- O Use of sampling data (to select treatment or disposal options),
- Types of sampling to be performed (detailed drums laboratory container, gas cylinder sampling, detailed tank, vessel sampling, etc.),
- o Sample analyses/compatibility testing,
- Methodology describing sampling and analytical procedures,
 and
- O QA and safety.

The plan will be reviewed to ensure compliance with applicable CCWASTE - 15 -

WSSRAP programs (See Section 3.0 - Technical Approach).

Container sampling activities, including sample collection, analysis, quality assurance and control, chain of custody, and documentation and reporting will be conducted in accordance with the guidelines set forth by the EPA in SW 846 (EPA, July 1982). In addition, the hazardous waste requirements established by the State of Missouri Department of Natural Resources (MODNR) and EPA Region VII will be implemented for these sampling activities.

Sampling of the drums and laboratory containers will be performed as the containers are opened in the staging area(s). Concurrent with the required analytical analysis, compatibility testing will be performed on-site to reduce turn-around time and increase staging/opening/bulking efficiency.

As each container is opened, the contents will be scanned for radioactivity. If activity levels are above background, one sample will be collected for radiologic identification and another collected for chemical analysis. After sampling, the container will be marked and removed to the designated radiological waste interim storage area. The final disposition of the radiologically contaminated wastes is not within the scope of this program. If activity levels equal background, a sample will be taken for compatibility testing.

Compatibility testing protocols, which have been developed by the Chemical Manufacturers Association (CMA, 1982), will be followed as outlined in Figure 6.

These compatibility testing procedures shown may require tailoring for site-specific containers at the WSCP or to meet the testing requirements of prospective treatment disposal facilities. A form similar to that shown in Figure 7 will be

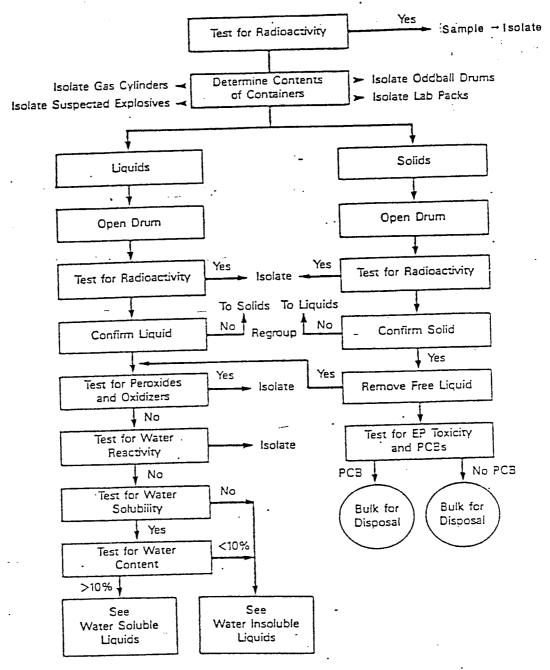


FIGURE 6 Compatibility Testing Protocol (Modified by Princeton Aqua -Science)

(Reprinted courtesy of Chemical Manufacturer's Association, Washington, D.C.)

Water Insoluble Liquids Testing

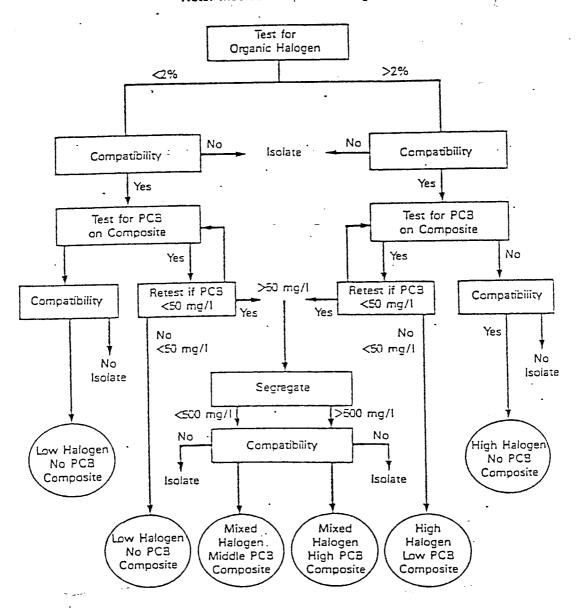


FIGURE 6 (continued) (Modified by Princeton Aqua Science).
(Reprinted courtesy of Chemical Manufacturer's Association, Washington, D.C.)

Water Soluble, Scan

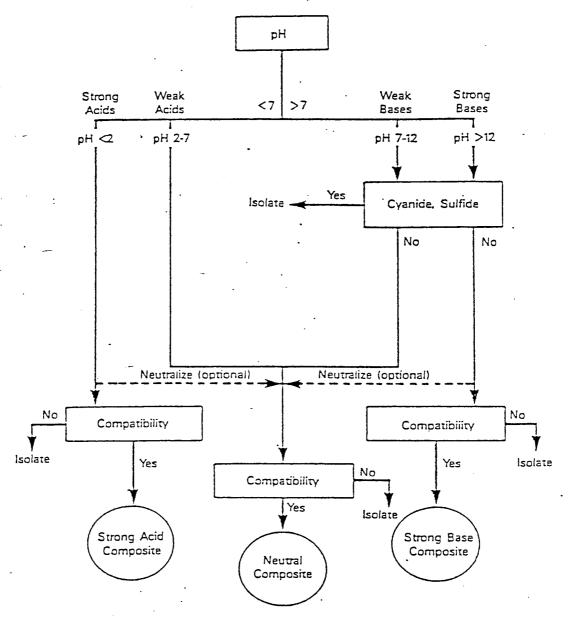


FIGURE 6 (continued) (Modified by Princeton Aqua Science)
(Reprinted courtesy of Chemical Manufacturer's
Association, Washington, D.C.)

FIGURE 7

Weldon Spring Site Remedial Action Project (WSSRAP) Route 2, Highway 94 South, St. Charles, Missouri 63303 Phone (314) 441-8086 Telecopy (314) 447-0803

CONTAINERIZED CHEMICAL CONTENTS IDENTIFICATION

Container Number CC-	<u>2 -0587 </u>
Radioactivity Level	
Alpha	Uranium-Natural
Beta	Uranium-238
Gamma	Thorium-230
	Radium-226
•	Others
Radiologically Releasable	e Off-Site: Yes or No
COMPATABILITY DATA	
pН	
Peroxide or Oxidizer	Yes or No
Reducing Agent	Yes or No
Water-Reactive	Yes or No
Halogen Content	Low or High
Acid	Strong, Weak, or No
Base	Strong, Weak, or No
Cyanides	Yes or No
Sulfides	Yes or No
Flammability	105 01 1.0
Specific Gravity	
Heat Content	BTU/1b
Solids Content	percent
	Yes or No
Hydrocarbon	Yes or No -
Pesticides	Low or High
Sulfur Content	Yes or No
Phenols	
Oil and Grease	percent
Water	percent
Viscosity	
Organochlorine Content	percent v
EP Toxicity Metals	Yes or No
Solubility	H ₂ O H ₂ SO ₄ or (CH ₃) ₂ SO
HAZARDOUS MATERIAL:	
	Yes or No
Comments:	
	·· (ai matuma)
Lab Reports Transcribed by	:(signature)
·	Date:
Form CC-3	
MK-Ferguson Company	Project Management Contract

used to compile the results of these test.

Based on the CMA protocol, wastes may be segregated into the following broad waste categories:

- o Liquids
 - Radioactive
 - Peroxides and oxidizing agents
 - Reducing agents
 - Water-reactive compounds
- o Water insoluble
 - Low halogen, low PCB
 - Mixed halogen, high PCB
 - High halogen, low PCB
- o Acids
 - Strong (pH <2)</pre>
 - Weak (pH 2-7)
- o Bases
 - Strong (pH >12), with or without cyanides or sulfides
 - Weak (pH 7-12), with or without cyanides or sulfides
- o Solids
 - Radioactive
 - Nonradioactive

The collection and analysis of samples from process equipment, tanks, vessels, sumps, and lines will require specialized access and sampling methodology. Because of their relative volume and quiescent condition, the contents will be stratified. It is anticipated that as many as three phases (oil, liquid, sludge) may be discovered and a sample from each will be collected to

determine material characteristics, compatibility, and handling requirements.

Unique or special containers, such as gas cylinders, batteries, fire extinguishers, lab packs, over-pressurized drums, etc. will be handled on a case-by-case basis as these containers are found. Since a much higher level of risk is associated with the handling or disturbance of these containers, separate procedures will be developed for handling and removal of these materials.

3.1.4 Bulking

By identifying broad waste categories, compatible waste types can be safely combined into bulk containers on-site without risk of fire or explosion, and disposal options can be determined without exhaustive or costly analysis of individual drums.

Once a compatible group of samples is identified a final disposal analysis will be conducted. Analytical requirements of a number of disposal facilities have been compared (MBL, 1982). The tests identified in Table 1 are representative of tests that may be required prior to acceptance of liquids and solids for disposal.

3.1.5 <u>Interim Storage</u>

Liquid and solid wastes identified during the sampling which exhibit radiation levels statistically significant above background will be transferred into interim storage at the WSCP for later incorporation into treatment/disposal activities to be performed under the WSSRAP. The specific location of the WSCP interim storage facility will be determined by the PMC once the total volume of radiologically contaminated containerized wastes are determined.

3.1.6 Treatment/Disposal

Once the containerized wastes have been categorized, they will be assigned appropriate treatment/disposal options. These options will be selected based on such factors as protection of public health, regulatory requirements, availability of treatment/disposal facilities, and applicability to site specific conditions (i.e., numbers of containers, volume of waste type(s), etc.).

A matrix, such as the one shown in Table 2, will be utilized to relate the type of containerized wastes to be handled from the WSCP to the types of waste treatment and disposal systems available. Only those treatment methods which have potential applicability to a given waste type and/or disposal categories (based on best engineering judgment) will be considered.

After the final treatment/disposal options have been determined, the wastes will be prepared to meet the requirements of the treatment or disposal facility and applicable Department of Transportation (DOT) shipping regulations.

Wastes will be removed and transported from the WSCP by the subcontractor to the designated treatment/disposal facility.

TABLE 1

POTENTIAL ANALYTICAL REQUIREMENTS FOR DISPOSAL *

- 1. Flammability
- 2. pH
- 3. Specific gravity
- 4. PCB analysis
- 5. Thermal content (BTU/lb)
- 6. Physical state at 70°F
- 7. Phases (layering in liquids)
- 8. Solids (%)
- 9. Hydrocarbon composition
- 10. Pesticide analysis
- 11. Sulfur content
- 12. Phenols
- 13. Oil and grease (%)
- 14. Water (%)
- 15. Viscosity
- 16. Organochlorine percentage
- 17. Metals analysis
 - a. Liquids for soluble metals.
 - b. Solids extracted according to the EPA Toxicant Extraction Procedure (24 hr.) which shows leachable metals.
 - c. Both liquid and solids checked for concentrations of the following metals:

Arsenic

Mercury

Barium

Nickel

Cadmium

Selenium

Chromium

Silver

TABLE 1 (continued)

Copper Lead Zinc

- 18. Both free and total cyanide content.
- 19. Solids checked for solubility in water, sulfuric acid, and dimethyl sulfoxide.

^{*} Reprinted from Muller, Broad, and Leo, 1982. Table originally printed in the Proceedings of the National Conference on Management of Uncontrolled hazardous Waste Sites, 1982. Available from Hazardous Materials Control Research Institute, 9300 Columbia Blvd., Silver Spring, MD 20910.

TABLE 2 CATEGORIES AND POTENTIAL TREATMENT/DISPOSAL TECHNOLOGIES

DH-SITE PREP						707A	TMFW	~		POSAL			
	ACT	TITI	:			-					-		5
TANK AND DRUM WASTE CATEGORIES	Bulking/ Censelidation	Above plane / Salidi Acathan	Physical/Chankel Provestimans	Salkitheatlan/ Chammal fleatlan	Deleasing	Physical/Chemical	wisquaal Castment	and Application		namaratum/Them Reduction	Recycle/Bouse C./ frostmost	Recuchs/Asuss V/s frastmant	(Suprell Injection
MUEDE PASIES	Boleson Censelle	Salking the	2	2 5 2 4	Ostensie	Physical/C	Brakqual frattman	3	•	Prepared Beautiful	35	œ >	3
. Acid: pH < 2.0	1	2	1	2	2	1	Z	2	Z	z	1	1	10
• 84541 gH > 12,0	1	2	1	2	2	1	2	2	2	2	1	1	1*
6439 VITA Sulfur	1	z	1	2	2	1	2	2	2	2	2	2	2.
• Bass with Cusards	'	2	ı	2	2	1	2	2	2	2	2	2	2*
OBCTRIC FIGURES							<u>. </u>				,	,	
+ High Helogen content (>2%)	1	Z	2	2	2	z	2	2	2	1	2	2	z•
Organic figures including * (av Helegen content (0 - 270)	1	2	2	2	2	1	1	1	2	1	1	2	2*
• (71)3 man->CB cantamounted	1	2	2	2	2	1	1	1	Z	1	1	1	2
PCS Conteminated Liquid (30-500 ppm)	1	2	2	2	2	1	2	2	2	1	2	2	2
PC3 Contaminated Liquid (> 500 tom)	1	2	2	2	2	,	2	2	2	1	2	2	2
CONTAMINATED WATER													
- 2 cpm < 12	1	z	1	2	ż	1	2	z	2	2	z	2	10
• Inorqueic	1	2	1	2	2	1	2	2	2	2	2	z	10
- Organic	1	2	1	2	2	1	1	1	2	2	2	2	10
₹3 Contaminated Liquid (< 50 tom)	1	z	2	2	2	1	2	2	2	1	2	2	2
• Pesticion	1	Z	1	2	2	1	1	2	z	Z	2	2	2
21.01201.020													
* Organic Law Halogan	1	1	1	1	z	1	1	2	1	1	z	2	2
* Organic High Halogen	1	2	2	2	z	1	ī	z	1	1	z	2	2
- Inorquinic	1	1	1	1	2	1	2	2	1	Z	2	z	2
- Flash point (470 4 f)	2	7-	2	2	2	2	2	2	Z	1.	z	Z	2
• Flesh point (+70 - 1-10 F)	1	z	2	2	2	z	2	Z	1.	1	z	2	2
. [lasn point (> i 40 2 7)	1	1	1	1	2	1	2	2	1	1	2	z	2
PCB Conteminated Solid * (150 ppm)	1	1	z	2	z	1	1	1.	1	1	2	2	2
PCS Conteminated Solid () SU - SUU com)	1	2	2	2	2	2	1	2	1.	ı	2	2	2
PCS Contamirated Solid (> 500som)	1	2	2	2	2	2	1	z	1 -	1	2	z	2
. Pasiicion	1.	2	1	2	2	1	Z	2	t	1	2	2	2
. Petroleum Residues	1	1	1	2	ı	ı	ı	1	ı	1	2	2	
SPECIAL WASTES													
• Redirective	•	•	•	•	•	٠	•	•	·	•	•	•	•
- Strong Oxidizer	Z	Z	1	2	2	1	2	2	1	2	2	2	2
- Strang Reducer	1	2	1	2	2	1	2	2	1	2	Z	Z	z
. Lab Pects	Z	2	2	2	1-	1	2	2	1	1	2	2	Z
• Explaires	2	2	2	z	1	1	2	2	2	1	2	2	2

Phone vants shall be repersied and handled as individual wasts ifems. There wasts shall be classified under the mare restrictive dispesal, calegory.

DRAFT.

^{1 -} Resonable Oation

^{2 -} Not Resourable (Impractical, Technically Infessible)

^{. -} further detailed analysis is verranted

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D.C., 1982.

APPENDIX A

Building	Chemical Name	Reference
All All All	CO2 Fire Extinguishers Dry Powder Fire Extinguishers Janitorical Chemicals	D S D S S
All	Soda Acid Fire Extinguishers	D S
Most Most	Asphaltic Mastic Sealer Asbestos Pipe Insulation	D S
Many Many	Flaking Paint (May be Lead) Sumps	, S D S
Overhead Utilities	Asbestos Pipe Insulation	S
Transformers Transformers	No-PCB Oil PCB Oil	S S
101	Elevator Cable Cleaner	S .
102 102 102 102 102	Hexane Nitric Acid Nitrogen Sodium Hydroxide Sulphuric Acid	D D D D
103 and 403	Alkali Wash Aluminum Solution ANN Calcium Oxide Condensate Water Deionized Water Dust Collector (Vacuum) Catch Filtrate Floor Wax Fuel Gas Hydrostatic Precipitator Iron Lime Slurry Magnesium Oxide Misc. Metal Nitric Acid N. G. Liquor from Ether Ext. Orange Salt O.X. Liquors Raffinate Sawdust Converter Waste	
103 and 403 103 and 403 103 and 403 103 and 403 103 and 403 103 and 403 103 and 403	Scrap Metal Sodium Carbonate Sodium Hydroxide Sump Liquor (Bldg 103) U303 UNH U03	D D D D : D

Building	Chemical Name	Reference
103 and 403 103 and 403 103 and 403 103 and 403 103 and 403	Uranium Ore Concentrates Wash Water Water Wet Scrubber Waste	D D D
105 and 403 105 and 403	Deionized Water Deionizer Chemicals Ether Ethylene Glycol Filter Press Waste Hexane Nitric Acid Raffinate Rubberized Mastic Sodium Carbonate Sodium Hydroxide Sump Liquor (Bldg 105) Tributyl Phosphate	D D D D D D D D D D D D D D D D D D D
108 108	Ferric Nitrate Nitric Acid	D D
_ 109	Compressed Gas Cylinders	S
201 201 201 201 201 201 201 201	Anhydrous Hydrogen Fluoride Ammonia Asbestos Cement Siding Asbestos Insulation (on Tanks) Azeo Bottle of Clear Liquid Condensate Drums of Unknown Chemicals Dust Collector Catches Elevator Cable Cleaner Fibered Roofing Sealer Green Salt Hydraulic Oil Hydrofluoric Acid Hydrogen Gas Lime Scrubbing System Waste Nitrogen Gas Orange Oxide Propane Reverter Off-Gas Waste (Rerun) Reverter Product	
202 202 202 202 202	Anyhdrous Hydrogen Fluoride Ammonia Asbestos Cement Siding Hydrofluoric Acid	D D D S D
301	Air Filter Catches	ם

Building	Chemical Name	Ref	erence
=======================================		D	S
301	Asbestos Cement Siding	Ď	_
301	Caustic Flake	Ď	
301	Caustic Liquid	Ď	•
301	Condensate From Process	D	
301	Dust Collector Catches	Ď	
301	Green Salt	Ď	•
301	Magnesium	Ď	
301	Metal Oxide (from Saws) Precipitation Tank Sediment	Ď	
301	Process Sewer Filter Cake	Ď	
301	Salt Bath Supplies	Ď	
301	Salt (Unknown Type)	Ď	•
301	Settling Tank Sediment	D	. •
301		Ď	
301	Slag	Ď	
301	Uranium Metal	ע	
302	Magnesium	D	S
302	Wet Scrubber Catch	D	
302	Graphite Sheets		S
		D	
401	Acid	ע	S
401	Asbestos (Bags)	D	S
401	Asbestos Cement Siding	Ď	2
401	Asbestos Floor Tile (offices)	ע	S
401	Asbestos Pipe Insulation	D	D .
401	Ash	ע	S
401	Batteries	D	٥
401	Diesel	D D	
401	Fuel Oil	.D	
401	Gasoline	ע	S
401	Hydrofluoric Acid	ъ.	. .
401	Hydrogen ZeoliteWater Softener	D	
401	Lube Oil	D	•
401	Oil Sorber (for Air Compressor)	D	
401	Phosphate	D	
401	Refrigeration Brine	D	
401	Salt		
401	Sodium Zeolite Water Softener	D D	-
401	Sulfite	ע	c
401	Water Testing Chemicals		S
403	(See Buildings 103 and 105)		٠
404	Barium Fluoride		S
404	Degreaser Chemicals	D	
404	Filter Cake	D	
404	Graphite		S
404	Green Salt	D	
404	Helium	D	
404	Hydraulic Oil	D	
404	Hydrogen	D	
404	Lithium Fluoride		S

Chemicals Used at the Weldon Spring Feed Materials Plant

104	Building	Chemical Name	Reference
104	=======================================		ת
10			_
10	404	Uranium metal	2
A06	405	Dust Collector Catch	D
August	406	Drum Storage	
Graphite Blocks		Dry Chemical Storage	. D
Waste from Feed Materials Plant In Drums filled During Army Decontamination Efforts S	= '	Graphite Blocks	S
In Drums filled During Army Decontamination Efforts S		Waste from Feed Materials Plant	_. S
Decontamination Efforts	400	In Drums filled During Army	
406	•	Decontamination Efforts	
### Asbestos Cement Siding	406		
407			S
Asbestos Pipe Insulation 407	400		
Asbestos Pipe Insulation Ether 407 Ether 407 Hydrofluoric Acid A07 Laboratory Chemicals A07 Non-Sparking Floor (Ether Lab) A07 Perchloric Acid A07 Perchloric Acid A07 Photograph Developing Chemicals 408 Acetylene Cylinders A08 Air Filters 408 Asbestos (Bags) A08 Asbestos (Bags) A08 Asbestos Ceiling Board A08 Asbestos Ceiling Board A08 Batteries A08 Carbide Tool Grinder Dust A08 Cleaning Solvent A08 Compressed Gas Cylinders A08 Compressed Gas Cylinders A08 Electrical Maintenance Supplies A08 Gasoline A08 Grease A08 Hydraulic Oil A08 Grease A08 Hydraulic Oil A08 Instrument Maintenance Supplies A08 Instrument Maintenance Supplies A08 Machine Shop Supplies A08 Milwright Supplies A08 Millwright Supplies A08 Millwright Supplies A08 Oxygen Cylinders A08 Pipefitting Supplies A08 Pipefitting Supplies A08 Pipefitting Supplies B0 Sheet Metal Working Supplies B1 Sheet Metal Working Supplies B2 Sheet Metal Working Supplies	407	Asbestos Cement Siding	D S
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408 Rigger's Supplies D 408 Sheet Metal Working Supplies D			D
408 Sheet Metal Working Supplies D		Rigger's Supplies	D
		Sheet Metal Working Supplies	
	408		S

Building	Chemical Name	Ref	erence	
408 408 408	Thread Cutting Oils	D D	s s	
408	Window Washing Supplies	D		
409	Ammonia		S	
409	Asbestos Pipe Insulation		S	
409	Lead Wall Lining (Teletype Room)		S	
409	Photograph Developer Chemicals		s s	
409	Printing Press Cleaner Printing Press Inks		S -	
409	rrincing riess inks			
410	Asbestos Pipe Insulation	D	S	
410	Laundry Room Supplies	D	S	
410	Medical Clinic Supplies	D	S	
410	Medical Laboratory Chemicals	D	S	
410	Perchloric Acid	D.		
410	SCBA Oxygen Generation Canisters	D	S	
412	Batteries	D	S	
413	Chlorine	D		
413	Chromate Phosphate	D		
413	Sulphuric Acid	D		
414	(Not Inspected)			
415	(None Found)		S	
416	Raffinate Pits	D	S	
417	Asbestos Pipe Insulation		S	
417	Flammable Materials	D	S	
417	Paint		- S	
417	Paint Solvents		S	
426	Water Tower (Drinking Water for	Coun	ty)	
427	Sewage Sludge	D.	S	
428	Propane	Ď		
429	(County Water Company Building)			
430	(None Found)		S	
431	(Not Inspected)			
432	(None Found)		S	
433	Dehydrating Agents		S	
433	Cement Mix		S	
433	Drums of Unknown Contents		S	

Chemicals Used at the Weldon Spring Feed Materials Plant

Building	Chemical Name	Reference
434	Drum of Unknown Contents	S
435	Batteries	S .
435	Drum of Unknown Contents	S .
435	Cryogenic Liquid Containers	S
435	Water Treatment Chemicals	S
436	Cans of Unknown Contents	S
437	Drums of Unknown Contents	S
438	Drums of Unknown Contents	s ·
438	Sodium Fluoride	S
438	Paints	S
438	Paint Solvents	S
439	(None Found)	S
441	Compressed Gas Cylinders	S
443	Drums of Unknown Contents	S
Ash Pond Area	Open Dump with Assorted Wastes	S

D = Drawings by Blaw-Knox of Plant As-Built.

NOTE: This list is not exhaustive, other chemicals are present.

Source: WSSRAP, 1987.

DRAFT

S = Survey by PMC Personnel Confirmed Presence of Container or Chemical.

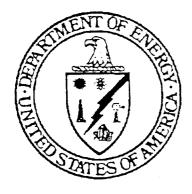
DOE/OR/21548-128 CONTRACT NO. DE-AC05-860R21548

WASTE ANALYSIS PLAN

Weldon Spring Site Remedial Action Project Weldon Spring, Missouri

AUGUST 1992

REV. 4



U.S. Department of Energy Oak Ridge Operations Office Weldon Spring Site Remedial Action Project



Weldon Spring Site Remedial Action Project Contract No. DE-AC05-860R21548 Rev. No. 4

PLAN TITLE: Waste Analysis Plan

APPROVALS

Man & Thron	8/17/92
Compliance Manager	Date
Grus Ciarte	08/5/72
Quality Assurance Manager	Date
013 44	8/18/92
Deputy Project Diffector	Date
a Thus & Form	<u> 8/18/9</u> 2
Project Director	Date
	Date
Project Manager	Date

Weldon Spring Site Remedial Action Project

Waste Analysis Plan

August 1992

Revision 4

Prepared by

MK-FERGUSON COMPANY and JACOBS ENGINEERING GROUP 7295 Highway 94 South St. Charles, Missouri 63303

Prepared for

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Oak Ridge Operations Office
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1 INTRODUCTION

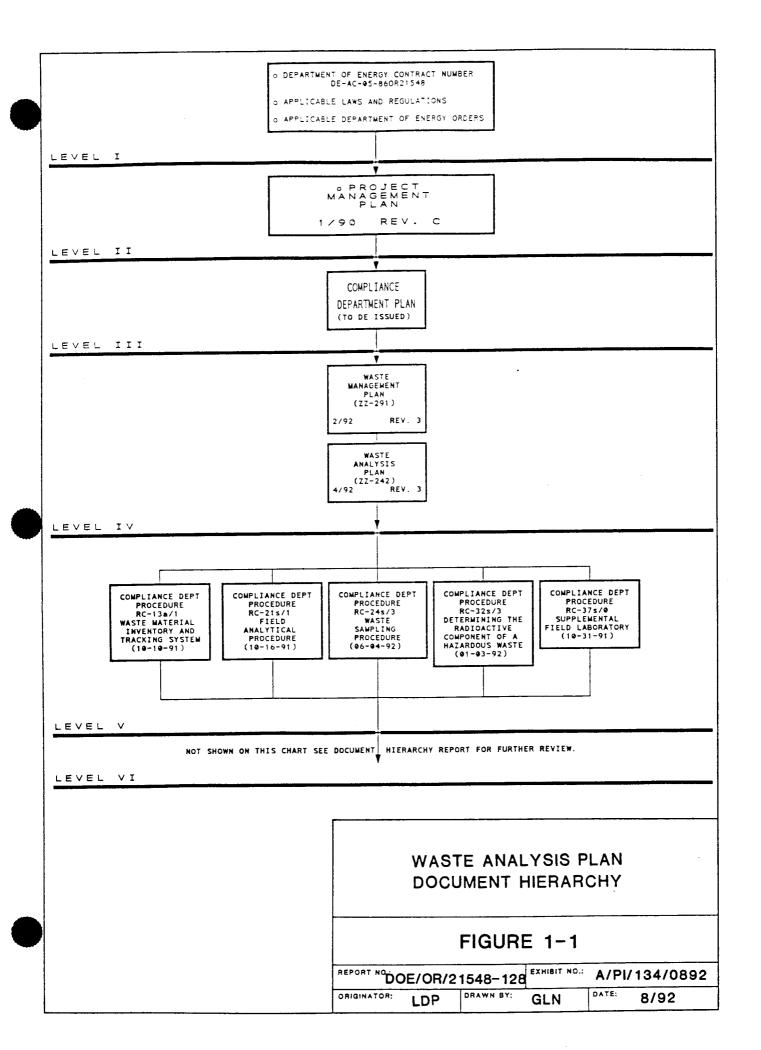
This Waste Analysis Plan has been designed to address chemical and radiochemical characterization requirements for management of wastes encountered at the Weldon Spring site, (WSS) Weldon Spring, Missouri. At a minimum, this plan will provide the means to obtain information necessary to classify, treat, store, or dispose of these wastes. The Waste Analysis Plan, which is designated as level IV in the Compliance Department document hierarchy (Figure 1-1), supports the Weldon Spring Site Remedial Action Project (WSSRAP) Waste Management Plan (MKF and JEG 1992a) and provides guidance to Waste Management personnel in the preparation of detailed sampling and analysis plans.

1.1 Background

Under the provisions of the Comprehensive Environmental Response, Compensation, and Liability Act (CERCLA), the site is required to comply with the applicable substantive requirements of the Resource Conservation and Recovery Act (RCRA) regulations. The requirements for long-term storage of hazardous waste are found in Title 40 of the Code of Federal Regulations, Part 264, which pertains to operation of Department of Energy Technical Service Division hazardous waste facilities. These regulations require detailed chemical and physical analysis of a representative sample of each waste.

The key elements of this Waste Analysis Plan, which are consistent with the General Facility Standards contained in 40 CFR 264.13(b)(1-3), specify:

- The appropriate sampling method for obtaining a representative sample of the waste for analysis.
- The parameters for which each waste will be analyzed, and the rationale for the selection of those parameters.
- The appropriate test methods required for these parameters.



1.2 Purpose

The purpose of this Waste Analysis Plan is to accomplish the following three objectives:

- 1. Characterization for compatibility and proper container selection prior to bulking and recontainerization of unknowns.
- 2. Identification of those wastes that are subjected to the storage and disposal requirements of RCRA and/or the Toxic Substances Control Act.
- 3. Recharacterization of site generated wastes to detect any changes in the concentrations of chemical constituents, the appearance of new constituents, or variations in physical properties.

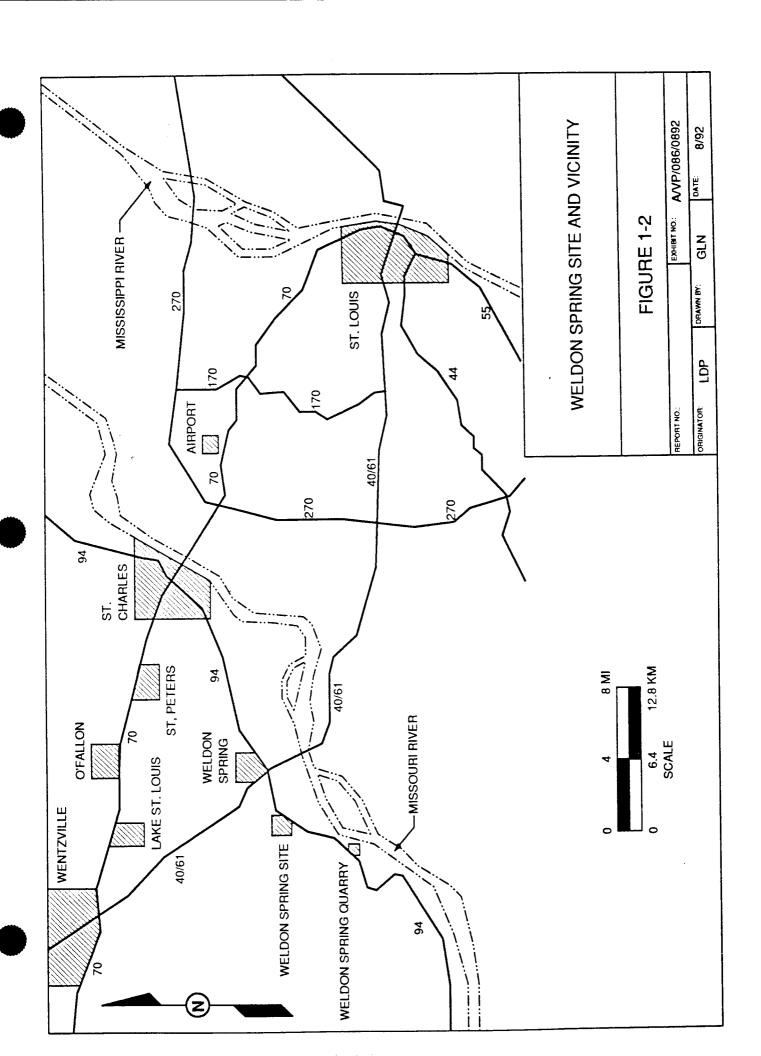
1.3 Scope

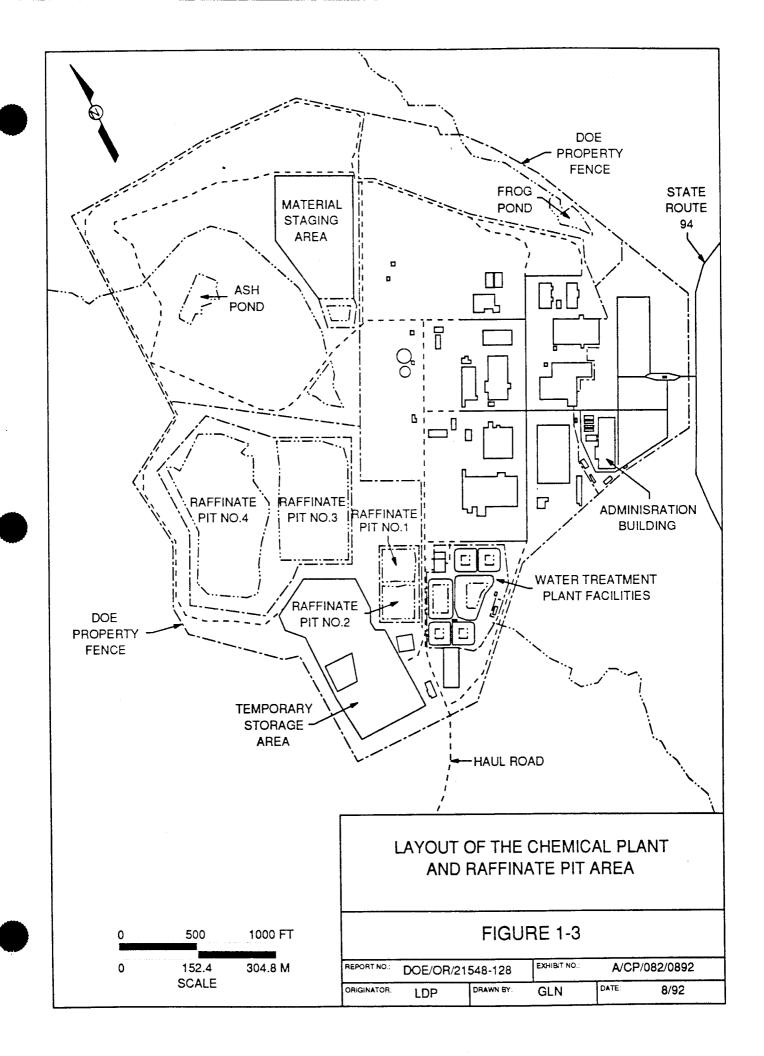
The plan is tailored to the requirements of the Weldon Spring Site Remedial Action Project in its effort to manage and ultimately dispose of hazardous and mixed wastes at the Weldon Spring site. Since the WSS is an inactive facility and a broad array of chemical wastes have been left at this site, this plan is generic in nature. It considers chemical and radiological analyses required for mixed waste characterization, interim storage, and disposal.

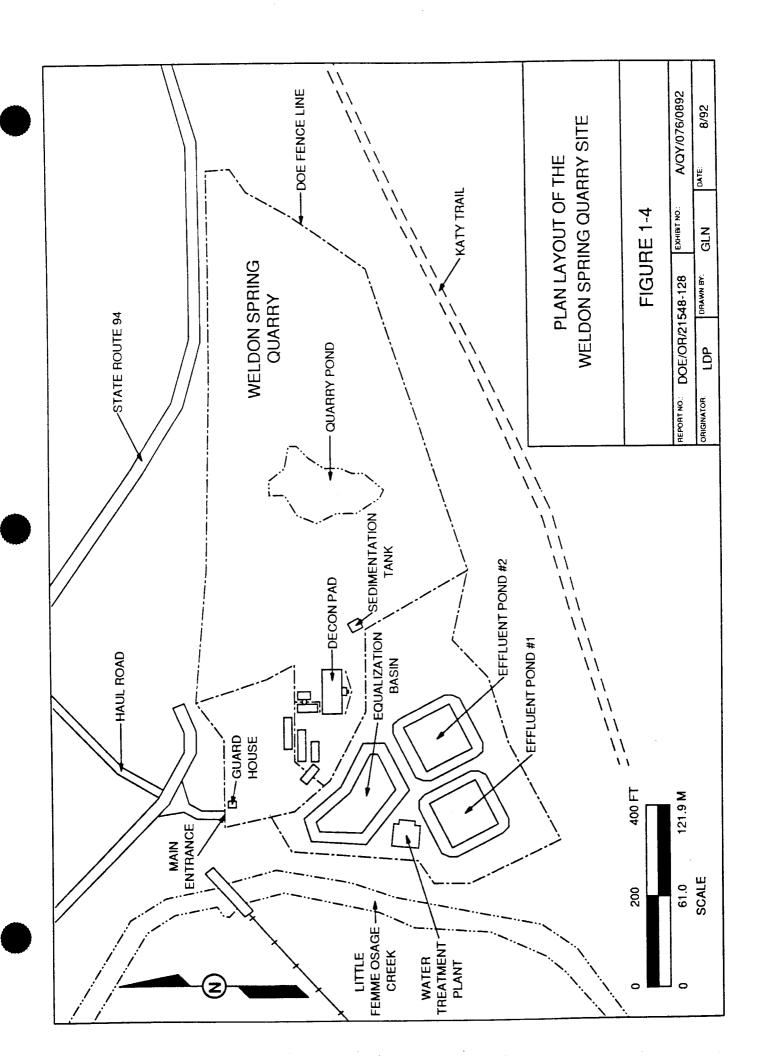
1.4 Site Description and Location

The Weldon Spring site (Figure 1-2) is located approximately 30 mi west of St. Louis in western St. Charles County, Missouri. St. Charles, the largest city in St. Charles County, is located approximately 15 mi northeast of the site. The site is bordered by large tracts of land owned by the Federal Government and the State of Missouri.

The WSS is divided into two distinct areas; one includes the raffinate pits and the chemical plant (Figure 1-3), which encompass 51 acres and 166 acres respectively, and the other is the quarry (Figure 1-4) which is located about 4 mi south-southwest of the raffinate pit/chemical plant area. These areas are located on Missouri Highway 94 southwest of the junction of U.S. Highway 40 (also known as Interstate 64 or U.S. Highway 61) and Missouri Highway 94.







The raffinate pit area contains four surface impoundments (raffinate pits) which contain wastes primarily from uranium and thorium processing and cover approximately 26 acres. The chemical plant consists of 13 major buildings, approximately 30 support structures, and the Ash Pond and Frog Pond areas. Ash Pond and Frog Pond are the two surface water bodies within the chemical plant area. Frog Pond is a surface impoundment near the northwest edge of the site.

The quarry is located in limestone and covers about 9 acres. The deepest part of the quarry is filled with water covering about 0.5 acres and is the only surface water body within this controlled area. A major source of potable groundwater in this area is the county well field located about 1 mi southeast of the quarry in the Missouri River alluvium.

1.5 Site History

From 1941 to 1944, the U.S. Department of the Army operated the Weldon Spring ordnance works (WSOW), constructed on the land that is now the WSS, for production of trinitrotoluene (TNT) and dinitrotoluene (DNT). The Army used the quarry for disposal of rubble contaminated with TNT. In the mid 1950s, 205 acres of the ordnance works property was transferred to the U.S. Atomic Energy Commission (AEC). This is now the raffinate pits and chemical plant area. An additional 15 acres was later transferred to the AEC for expansion of waste storage capacity. From 1957 to 1966, the AEC operated a uranium-processing facility at the Weldon Spring Uranium Feed Materials Plant, which subsequently became the Weldon Spring Chemical Plant (WSCP). Ore concentrates and some scrap metal were processed at the plant. Products that included uranium metal were shipped to other sites. Thorium-containing materials were processed on an intermittent basis. Radioactive raffinates from the processing were placed in four on-site pits. Other radioactive wastes were disposed of in the quarry.

After closure by the AEC, the chemical plant was reacquired by the Army in 1967. The Army partially decontaminated several buildings, dismantled some of the equipment, and began converting the facilities to produce herbicides. In 1969, prior to becoming operational, the herbicide project was canceled prior to bulk quantities of production chemicals being brought on site. As successor to the AEC, the U.S. Department of Energy (DOE) assumed responsibility for the raffinate pits. In 1984, the Army repaired several of the buildings at the chemical plant, decontaminated some of the floors, walls, and ceilings, and isolated some contaminated equipment.

In May 1985, the DOE designated the control and decontamination of the WSS as a Major Project (this project has since been designated as a Major System Acquisition). In October 1985, custody of the chemical plant was transferred to the DOE. A Project Management Contractor (PMC) for the Weldon Spring Site Remedial Action Project (WSSRAP) was selected in February 1986, and a DOE project office was established on the site in July 1986. The Project Management Contractor, MK-Ferguson Company, assumed control of the WSS on October 1, 1986.

On October 15, 1985, the U.S. Environmental Protection Agency (EPA) proposed to include the quarry on the National Priorities List (NPL). This listing occurred on July 30, 1987. On June 24, 1988, the EPA proposed to expand this designation to include the raffinate pits and chemical plant area. On March 30, 1989, these areas were also included in the listing and resulted in a single designation as the WSS.

1.6 Overview of Site Contamination

The WSS is contaminated with radioactive materials, nitroaromatics, heavy metals, asbestos, and various organic compounds. Contamination has been detected in various media groups including surface water, groundwater, soil, sludge, structures, and equipment. Asbestos has been found throughout the site buildings in the form of transite siding and insulation. Most of the asbestos is radiologically contaminated. Contamination in the vicinity of the WSS is located mainly along ditches, drainageways, and roads; some of the nearby ponds and lakes also contain elevated concentrations of certain contaminates.

Raffinate Pits 1, 2, and 3 contain raffinate sludge and slag resulting from the refining of uranium ore concentrates and the recycling of scrap metal at the chemical plant. In addition to this type of slag and sludge, Pit 4 contains wastes from the processing of thorium-containing materials and drums and rubble from the partial decontamination of the chemical plant. The pits contain approximately 168,200 m³ (220,000 cy) of wastes. Thorium-230 is the predominant radionuclide in the pit wastes.

The chemical plant consisted of 43 buildings and miscellaneous structures. The actual processing of radioactive materials occurred in only a small number of these buildings. However, contamination has been confirmed in most of the non-process buildings and external areas as well. The spread of contamination most likely occurred as a result of (1) routine plant

operations (e.g., tracking of contaminants from process areas and temporary relocation of contaminated equipment for repair), (2) processing support activities (e.g., waste handling), and (3) surficial deposition of airborne particulates. Some contamination also may have occurred subsequent to plant closure as a result of relocation of some contaminated equipment from process buildings into non-process buildings during earlier cleanup activities, and from transport of contaminated material by environmental factors (e.g., wind) and local biota (e.g., wasps).

Numerous smaller containers of miscellaneous wastes were left in various locations about the chemical plant. These containers were collected, consolidated (where appropriate), and containerized and placed into storage.

The quarry was used intermittently for disposal of chemically and radioactively contaminated materials from the early 1940s to 1968. The radioactively contaminated materials are those associated with uranium and thorium processing activities previously carried out at the chemical plant and at other sites in the St. Louis area. The chemical contaminants include polychlorinated biphenyls (PCBs), polynuclear aromatic hydrocarbons (PAHs), and nitroaromatics. There are approximately 72,600 m³ (95,000 cy) of contaminated wastes in the quarry. These wastes are in the form of rubble, soil, equipment, structural steel, and sludges. In addition to uranium, elevated concentrations of several metals including arsenic, copper, lead, and nickel have been detected.

Table 1-1 illustrates the wide variety of chemicals that has been used at the WSS.

1.7 Facility Description

Building 434 is currently used for storage of drummed waste at the WSS. This building, which was formerly used as a storage structure, was cleaned and refitted to RCRA and Toxic Substances Control Act (TSCA) standards. The defective concrete was removed and the floor was cleaned with a High Efficiency Particulate Air (HEPA) vacuum and washed with a trisodium phosphate solution. It was then etched with a 15% hydrochloric acid solution which was subsequently neutralized with a weak ammonium hydroxide solution and flushed with water.

All joints and cracks were sealed utilizing a polyurethane sealant. An epoxy coating with slip resistant granules was applied to the floor after the installation of curbing for secondary containment for liquids (see Figure 1-5).

TABLE 1-1 Chemicals Used at Former Explosives Production Facility and Uranium Feed Materials Processing Plant

TNT, DNT Production (chemicals used at site)

Ammonia Caustic soda Fuel oil Oleum (sulfuric acid)

Toluene Nitric Acid

Sellite (sodium sulfite)

Uranium Processing

Nitric acid
Sodium hydroxide
Sulfuric acid
Sodium carbonate
Phosphate

UNH (uranium amine)
UO₃ (uranyl oxide)
Uranium ore concentrate

Lime Ether

Ethylene glycol Tributyl phosphate Ferric nitrate Paint and catalysts Anhydrous hydrogen

Ammonia Green salt

(uranium tetrafluoride Hydrofluoric acid Hydrogen gas Nitrogen gas Orange oxide

(uranium oxide orange)

Propane
Caustic liquid
Magnesium
Graphite sheets
Diesel fuel
Fuel oil
Gasoline
Hydrogen zeolite
Refrigeration brine

Sodium zeolite
Sulfite
Helium
Hydraulic oil
Uranium metal
Laboratory chemicals
Perchloric acid
Grease
Chlorine

Chromium phosphates

Acid (misc.)
Benzene

Corrosive resistant coating Epoxy paint and catalysts Unspecified flammable

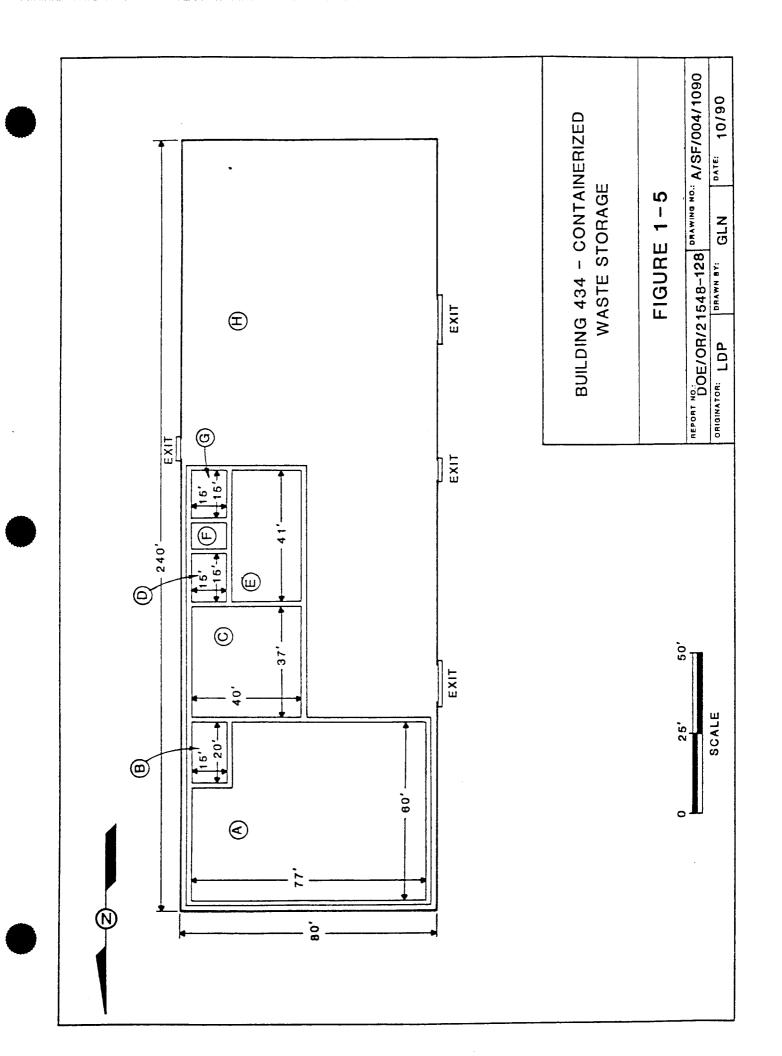
materials
Hot die lube
Linseed oil (boiled)
Lubriplate
Melcolene
Metalube
Methylene glycol
Methylisobutyl ketone

Motor oil
Paint
Paint solvents
Phenoline thinner
Polyclad

Polyurethane paint Rustbound primer

Tar

Source: (MKF and JEG 1988; reproduced from JEG 1989).



Lighting is provided by lights suspended from the roof support structure with service provided by 240 volt panelboards located inside the structure.

An acrylic/aliphatic urethane resin coating designed for seamless encapsulation was applied to the roof of the structure to prevent leakage. Three secondary containment areas were created within the building by constructing 6 in. x 6 in. concrete berms. Four smaller containment areas were constructed within these areas by securing 4 in. x 0.25 in. angle iron to the floor. These containment areas are designed to provide secondary containment in accordance with both RCRA and TSCA requirements.

1.8 Overview of Waste Characterization

For the purpose of characterization, there are seven general categories of waste that require different laboratory sample preparation and analytical procedures present at the Weldon Spring site. The manner in which the wastes are to be grouped is dependent on chemical composition and the waste disposal or treatment technology to be utilized.

Aqueous liquid wastes which are considered inorganic can be acids, bases, solutions containing toxic metals and/or organics, or solutions containing reactive cyanide or sulfides. These wastes are candidates for chemical treatment or incineration.

Non-aqueous liquid (NAL) wastes, which include chlorinated and non-chlorinated solvents, fuels, and organophosphorus liquids such as tributyl phosphate, are candidates for incineration. The majority of the NALs are radioactively contaminated to the extent that commercial incineration scenarios are not feasible. Thus, the analytical and physical characterization requirements specified in the Waste Acceptance Plan for the K-1435 Toxic Substance Control Act (TSCA) Resource Conservation and Recovery Act (RCRA) Incinerator (DOE 1986) should be met for organic liquid wastes. Table 1 of the K-1435 Waste Acceptance Plan has been included as Appendix A to this Waste Analysis Plan.

Oils and paints, although they are NALs, are treated as a separate category for the purposes of waste analysis. Untreated wastes classified as oils and paints are not viable land disposal candidates due to the amount of free liquids, toxic metals such as chromium and lead, and land-disposal-restricted organics (LDR) in these wastes which precludes land disposal. Although the amount of heavy metals typically present in paint-related materials make this type

of waste a marginal incineration candidate, oils and paints destined for off-site disposal are to be analyzed according to the requirements for incineration.

As a generic category, non-soil solids (NSS) include both inorganic and organic materials. Inorganic NSS materials may be inorganic salts or metals. It is possible that these wastes are characteristically hazardous due to elevated levels of toxic characteristic leaching procedure (TCLP) metals, cyanide or sulfide levels in excess of regulatory guidelines or water reactivity.

Organic NSS materials include organic chemicals in crystalline or powdered form (as well as wood and plastics). It is possible that some of these materials are "listed substances" or contain "listed substances". The presence or absence of any such "listed substance" must be determined to allow proper management of these wastes. These wastes may also be considered hazardous if they contain in excess of the regulatory limits of any TCLP constituent listed in 40 CFR 261.24.

Soils and solids include materials such as soil, sand, roofing material, concrete, and dust collected in vacuum cleaner bags. During site activities, a variety of chemically contaminated soils will be encountered which will require characterization to ensure proper management. Demolition activities will also result in a variety of wastes that will require characterization.

Site generated wastes are those wastes generated at the Weldon Spring site through onsite laboratory support activities, vehicle maintenance, personal protective equipment usage, and monitoring activities. In many cases, the chemical make-up of the waste will be known or can be approximated through knowledge of the generation process and will allow these materials to be placed in one of the above mentioned categories.

The final general category of wastes is of unknown liquids and solids. These are unknowns which are primarily small quantities of laboratory reagents and commercial products, or those materials contained in sumps or tanks which have not been characterized or consolidated. The consolidation of unknowns will not occur. Only after proper identification and subsequent reclassification into one of the other classes will materials be consolidated.

The field analytical requirements of each category are shown in Table 1-2 and the detailed laboratory analytical requirements are shown in Table 1-3. Each analysis must be

TABLE 1-2 Field Waste Analysis for Generic Waste Types

Parameter	Aqueous Liquids	Non-Aqueous Liquids	Oils & Paints	Inorganic Non- Soil Solids	Organic Non- Soil Solids	Soils & Solids	Unknowns
Visual inspection	×	×	×	×	×	X	×
Surface radiological survey	×	×	×	×	×	X	×
Hq	×			×		þ	þ
Redox	×	×		×	X	þ	×
Solution-reactivity	×	×	X	×	×	þ	×
Flame test	×	×	×	×	×	q	×
Sodium fusion		X			X	P	3
Ferrox		×			×	q	v
Hydrocarbon functionality		×			×	q	ű
Oxygen functional group		×			×	þ	၁
Nitrogen functional group		×			×	q	S
Sulfur functional group		X			×	4	o o
PCB screen			×			ą	v
Flashpoint		X	×			ą	S
Field compatibility	×	X	×	×	×	۽	×
Fingerprint screening						٩	
Inorganic functional group	×			X		q.	၁

Unknowns are materials, which based on visual inspection and surface radiological scan, cannot be classified into any other class until sufficient field analyses are completed to allow reclassification.

These tests will be performed based on the judgement of the Waste Management Group. Manager or his designee on a case-by-case basis.

These tests will be performed once it has been determined that the waste can be classified in another generic class that is amenable to the indicated analysis.

د م

TABLE 1-3 Laboratory Waste Analysis for Generic Waste Types

Parameter	Aqueous Liquids	Non-Aqueous Liquids	Oils & Paints	Inorganic Non- Soil Solids	Organic Non- Soil Solids	Soils & Solids	Unknowns
Suspended solids	×		Х				6
Total metals	×	Х	x	X		×	15
Volatile organics		Х	X			×	62
Semi-volatile organics		Х	x			×	æ
Moisture content		X	x	X	×	×	83
Specific gravity	×	X	X				63
Ash content	х	Х	x	X	×	×	4
Flash point		Х	x				6
Corrositivity	×	Х	×			×	8
Heating value		Х	×		×	×	4
Reactivity	×	X	×	×	×	×	8
Total organic halides	×					×	æ
Total organic carbons	×		×			×	6 2
Paint filter test		X	×	X	×	×	æ
Radiological characterization	×	X	×	×	×	×	45
Viscosity		X	×			×	æ
PCBs		Х	×		×	×	e 5
Dioxins and furans		×			X		eş.
Oil and grease	×					×	6
							

TABLE 1-3 Laboratory Waste Analysis for Generic Waste Types (Continued)

Parameter	Aqueous Liquids	Non-Aqueous Liquids	Oils & Paints	Inorganic Non- Soil Solids	Organic Non- Soil Solids	Soils & Solids	Unknowns
Suspended solids	×		Х				8
Sulfur	×	×					æ
Chlorine	×	×	X	X	×		63
Phosphorus	×	Х	Х	X	×		•
Bromine	×	×	X	x	X		e q
Fluoride	×	X	x	X	X	×	æ
TCLP metals	×	×	X	X	×	×	æ
TCLP pesticides	×	X			×	×	65
TCLP herbicides	×	X			×	×	æ
TCLP volatiles (ZHE)	×	Х	x	X	×	×	e z
TCLP semi-volatiles	×	Х	x	X	X	×	45
Asbestos				×	×	×	æ
Polyaromatic hydrocarbons	×				×	×	45
Nitroaromatics	×				×	×	æ

Analysis to be performed after consolidation into one of the remaining groups

tailored to the specific waste; however, the choice of analytical parameters can generally be limited to those indicated in the table. The appropriate method for each parameter is shown in Table 1-4.

Wastes that can be categorized or determined to be nonhazardous based on original container labels do not require analysis for chemical characterization.

TABLE 1-4 Method Reference

Laboratory Parameter	Method Reference
Suspended solids	EPA 1601
Total metals	SW 6010, SW 7XXX ^a
TCL volatiles	EPA CLP
Volatile organics	SW 8240
ВТЕХ	SW 8020
Semi-volatile organics	SW 8270
Moisture contents	ASTM D1533 or ASTM D2216
Specific gravity	ASTM 3142 or ASTM C854
Ash content	ATM D482 or ASTM D3174
Flash point	sw 1010
Heating value (BTU)	ASTM D3286 or ASTM D240 or SW 5050
Corrositivity	SW 1110 or SW 9045
Elemental analysis/anions	
1. Sulfur	ASTM D129 or ASTM D3286/EPA 3000
2. Phosphorus	EPA 36528
3. Fluorine	ASTM D3761 or ASTM D3286/EPA 3000
4. Chlorine	SW 9251, ASTM D2361 or ASTM D3286/EPA 3000
5. Sulfate	SW 9036/EPA 3000
6. Nitrate	SW 9200/EPA 3000
Reactivity (cyanide and sulfide)	SW 7332 and SW 7341
TCLP	Extraction SW 1311
1. Metals	Digestion: SW 3010 Analysis: SW 6010
2. Pesticides	Extraction: SW 3510 Analysis SW 8080
3. Herbicides	Extraction: EPA 615 Analysis EPA 615
4. Volatile organics	Analysis 8240
5. Semi-volatile organics	Extraction 3510 Analysis 8270
Total organic halides (TOX)	SW 9020
Total organic carbon (TOC)	SW 9060
Paint filter test	SW 9095

TABLE 1-4 Method Reference (Continued)

Laboratory Parameter	Method Reference
Suspended solids	EPA 1601
Radiological characterization	
1. Total uranium	EPA 520/5-84-006 ^b Method 00-05
2. Th-230	EPA 520/5-84-006 ^b Method 00-05
3. Th-232	EPA 520/5-84-006 ^b Method 00-05
4. Gross alpha and beta	SW 9310
Viscosity	ASTM D445
PCBs	SW 8080
Oil and grease/TPH	EPA 4131 SW 9073
Dioxins and furans	SW 8280
Asbestos	EPA 600/M4-82-020
Polyaromatic hydrocarbons	SW 8310
Nitroaromatics	SW 8330
a 7000 Series Methods (Atomic Abs b Eastern Envrionmental Radiation Fa August 1984	orption Methods) Icility Radiochemistry Procedure Manual,

2 QUALITY ASSURANCE AND QUALITY CONTROL

Weldon Spring site (WSS) detailed laboratory data must be legally defensible and valid. This requirement must be understood, planned, and provided for to ensure that adequate quality control is maintained. The project quality assurance (QA) organization is detailed in procedure QAPP-1 and the *Environmental Quality Assurance Program Plan* (EQAPP) (MKF and JEG 1991c).

2.1 Data Quality Objectives

Due to the wide variety of waste types, information sought, and end use of the data from waste analysis, no single set of data quality objectives (DQOs) can be established. DQOs are developed during the planning stages for each waste management sampling activity.

Since the Weldon Spring Chemical Plant is not an active facility, and because of documentation regarding the generation process of these wastes and waste management practices common to industry at the time of plant shutdown, many wastes at the WSS require detailed characterization to determine their regulatory status. Additionally, due to the extent of known radioactive contamination present on site, all wastes are characterized to determine if such contamination is present.

There are basically five reasons for collecting samples at the Weldon Spring Site Remedial Action Project (WSSRAP). Samples may be collected solely for the purpose of subjecting them to qualitative field analyses. These analyses are designed to provide gross chemical characterization information; determine the extent and appropriateness of detailed laboratory analyses; and aid in determining shipping classification and packaging, marking, and labeling requirements for transportation purposes.

Semiquantitative field analyses are performed on various samples collected at the WSSRAP. These analyses determine crude concentration levels to support the decision making process for various projects such as liner leak detection, well tracer studies, and water treatment plant equipment settings.

Quantitative or detailed laboratory analyses are performed to provide precise and accurate data regarding the characteristics of given wastes to determine proper regulatory classification, appropriateness of various treatment or disposal options, and to satisfy the analytical requirements of any treatment, storage, or disposal facility (TSDF) that may accept wastes from the WSS. Additionally, prior to releasing any waste material from the radioactive materials management areas (RMMA) of the WSS, wastes must be sampled and subjected to detailed radiological analyses as detailed in procedure RC-32. This procedure is used to verify that waste materials meet the requirements of DOE 5400.5, Radiation Protection of the Public and Environment, as well as the WSS off-site release policy.

Lastly, waste materials proposed for consolidation must be sampled and verified to be compatible by two waste management engineers (WME). This action is necessary to minimize the potential for adverse reactions and unnecessary chemical or radiological exposures.

Accuracy and precision must be determined and assessed through the use of quality control samples including distilled water blanks, trip blanks, equipment blanks, field duplicates, matrix spikes, and matrix spike duplicates. Table 2-1 documents the frequency that these samples must be collected and the DQOs routinely applied. It must be noted that the DQOs are intended for guidance only. DQOs specific to a sampling activity may vary slightly and should be documented in the sampling and analyses plan for that activity.

Distilled water blanks are used to monitor the purity of the distilled used to prepare preserving reagents, trip blanks, and equipment blanks. These samples will be collected quarterly from the distilled water reservoir.

Trip blanks document volatile organic contamination attributable to shipping and field handling procedures. Trip blanks must be collected prior to entering the field to perform any sampling activity. Trip blanks are collected by filling the vial directly from the distilled water reservoir, transporting the vials unopened to the sampling site, and then shipping unopened to the laboratory. A trip blank is required each day for each sampling activity where samples are collected for volatile organic analysis.

Equipment blanks are useful in documenting adequate decontamination of reusable sampling equipment. They are collected after completion of decontamination, and prior to sampling, by collecting a sample of the media used to rinse the sampling equipment.

TABLE 2-1 OA Sample Frequency and Data Quality Objectives for Waste Analysis

The second of th	5			,		•				
	Dis Wat	Distilled Water Blank	Trip Blank	Blank	Equipment Blank	. Blank	Matrix Spike & Matrix Spike Duplicate	k Matrix icate	Field Duplicate	licate
Analyte	Frequency	Criteria	Frequency	Criteria	Frequency	Criteria	Frequency	Criteria	Frequency	Criteria
Volatile organics	1/QTR	-	1/Event	В	1/Event	æ	1/20 or 1/batch	Ą	1/20 or 1/batch	٩
Semi-volatile organics	NA		ΝA	1	1/Event	æ	1/20 or 1/batch	þ	1/20 or 1/batch	þ
Pesticides/PCBs; herbicides	٧×		NA		1/Event	æ	1/20 or 1/batch	Ъ	1/20 or 1/batch	p
Nitroaromatics	NA		NA	-	1/Event	æ	1/20 or 1/batch	þ	1/20 or 1/batch	٩
Polyaromatic hydrocarbons	νV	1	NA		1/Event	ec	1/20 or 1/batch	þ	1/20 or 1/batch	q
Dioxins and furans	ΥN		٧N		1/Event	æ	1/20 or 1/batch	þ	1/20 or 1/batch	q
Oil and grease/TPH	NA	l	NA	-	1/Event	83	1/20 or 1/batch	þ	1/20 or 1/batch	<20% RPD
Total organic carbon	1/QTR	mddI >	٧N	***	1/Event	æ	1/20 or 1/batch	þ	1/20 or 1/batch	<25% RPD
Total organic halides	1/QTR	All CMPD < DL	NA		1/Event	ಪ	1/20 or 1/batch	þ	1/20 or 1/batch	<25% RPD
Metals	1/QTR	AII CMPD < DL	NA		1/Event	æ	1/20 or 1/batch	þ	1/20 or 1/batch	q
CI, SO ₄ , F, NO ₃ , PO ₄	1/QTR	All CMPD < DL	NA		1/Event	B	1/20 or 1/batch	þ	1/20 or 1/batch	P
Total U, Th-228, 230, 232	1/QTR	All CMPD < DL	NA		1/Event	æ	1/20 or 1/batch	þ	1/20 or 1/batch	P
Heating value (BTU)	٧X	-	NA	1	1/Event	æ	NA		1/20 or 1/batch	<20% RPD
Flash point	NA		NA		1/Event	æ	NA	-	1/20 or 1/batch	<20% RPD
Ash content	NA		Ϋ́	•	1/Event	ec	NA	-	1/20 or 1/batch	<20% RPD
Suspended solids	1/QTR	<dl< td=""><td>ΝΑ</td><td>-</td><td>1/Event</td><td>65</td><td>NA</td><td></td><td>1/20 or 1/batch</td><td><20% RPD</td></dl<>	ΝΑ	-	1/Event	65	NA		1/20 or 1/batch	<20% RPD
Specific gravity/viscosity	NA	1	٧X		1/Event	æ	NA	-	1/20 or 1/batch	<20% RPD
Swipes	Y Z	1	Ϋ́	1	1/Event	æ	٧٧	-	1/20 or 1/batch	<50% RPD
No higher than the highest of the following: All commonteds < 0.03 Regulatory Threshold: All compounds < 0.05 Sample Concentration	et of the follows	or All compounds <	DI: All compo	unds < 0.05 Reg	rulatory Threshol	d: All compour	ids < 0.05 Sample Co	oncentration		

No higher than the highest of the following: All compounds <DL; All compounds <0.05 Regulatory Threshold; All comp Evaluated on a case-by-case basis.

Field duplicates are independent samples that are collected as close as possible to the same point in space and time. They are two separate samples taken from the same source, stored in separate containers, and analyzed independently. These samples are useful in documenting the precision of the sampling process.

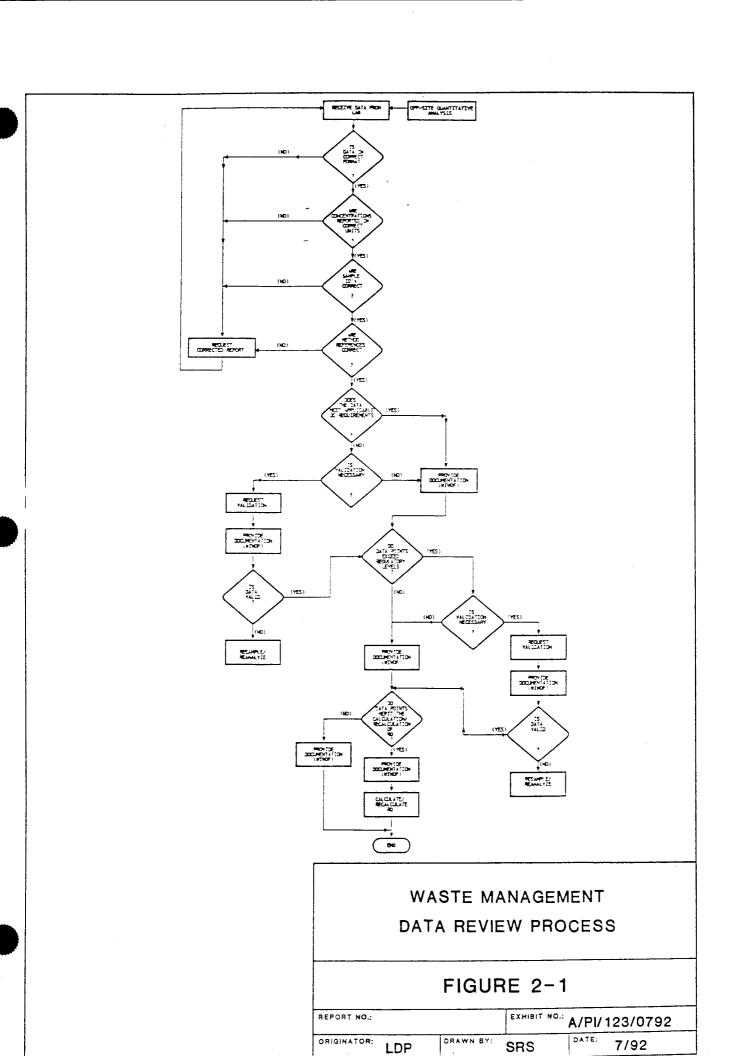
Matrix spikes and matrix spike duplicates are intralaboratory splits of a single sample that receive identical spike concentrations of the target analyte and are used to document the precision and bias of a method in a given sample matrix.

2.2 Data Verification

All detailed laboratory data must be reviewed and verified upon receipt by procedure ES&H 4.9.1 to ensure that documentation and data are reported in compliance with established reporting requirements and standard operating procedures (SOPs), and to ensure all requested analyses are performed. Only analytical methods specified in Table 1-4, or any attachments to this Waste Analysis Plan, are to be used for detailed laboratory analyses. Substitutions or departures from these methods must not be allowed without written Project QA or Waste Management Department consent prior to the analysis.

Chain-of-custody procedures as detailed in procedure ES&H 4.1.2, including Laboratory Work Authorization Form completion, must be maintained for all samples submitted for detailed analysis. Waste Management Field Sampling Data Forms, as required by procedure RC-24s, must be maintained during all sampling operations to document sampling methods, equipment, and observations.

Analytical data in the form of a formal hard copy report must be reviewed as indicated in Figure 2-1 by technical Waste Management Group (WMG) personnel familiar with the investigation and/or the analytical protocols requested. This review determines the general acceptability of the data and the regulatory status of the material. Review of the data for each material must be documented on a Waste Inventory Tracking System (WITS) Data Sheet in accordance with procedure RC-13a.



2.3 Data Validation

Data validation documents the quality and usefulness of the data and the documentation developed during sample analysis. Laboratory records of analytical data and quality related field data are reviewed to assess laboratory performance as compared to quality control criteria, data quality requirements, and procedural requirements.

Approximately 5% of the data points from waste samples submitted to the contract laboratories will be validated. These data points will be selected randomly by the laboratory coordinator. An additional 5% of the data points will be selected for validation by technical personnel familiar with the particular waste analysis program after the data review portion of the verification process is complete.

2.4 Waste Analysis Records and Documentation

All waste analysis QA records must be maintained on file by the QA Department in accordance with procedure QAPP-9 for the duration of the project or until otherwise directed by the U.S. Department of Energy (DOE). At a minimum, these records will be retained for 10 yr after the completion of remedial actions as required by the *Comprehensive Environmental Response*, Compensation and Liability Act (CERCLA).

Waste analysis QA records must include the following:

- Waste Management Field Sampling Data Forms (RC-24s)
- Chain-of-Custody Forms (ES&H 4.1.2)
- Laboratory Authorization Request Forms (ES&H 4.1.2)
- Field Analysis Data Sheets (FADs) (RC-21s)
- Supplemental Analysis Report (SAR) (RC-37s)
- Bench Scale Compatibility Determination (RC-26s)
- Drum Inventories (RC-26s)
- Waste Inventory Tracking System (WITS) Data Sheet (RC-13a)
- Analytical data reports received from detailed laboratory analyses
- Certification of Nonradioactive Waste (RC-32s)

Duplicates of all the above are maintained by the WMG in the waste analysis files. For containerized wastes, all data and forms pertaining to a particular container are placed into a file unique to the WITS number assigned to that container. All other data are filed by sampling event or activity.

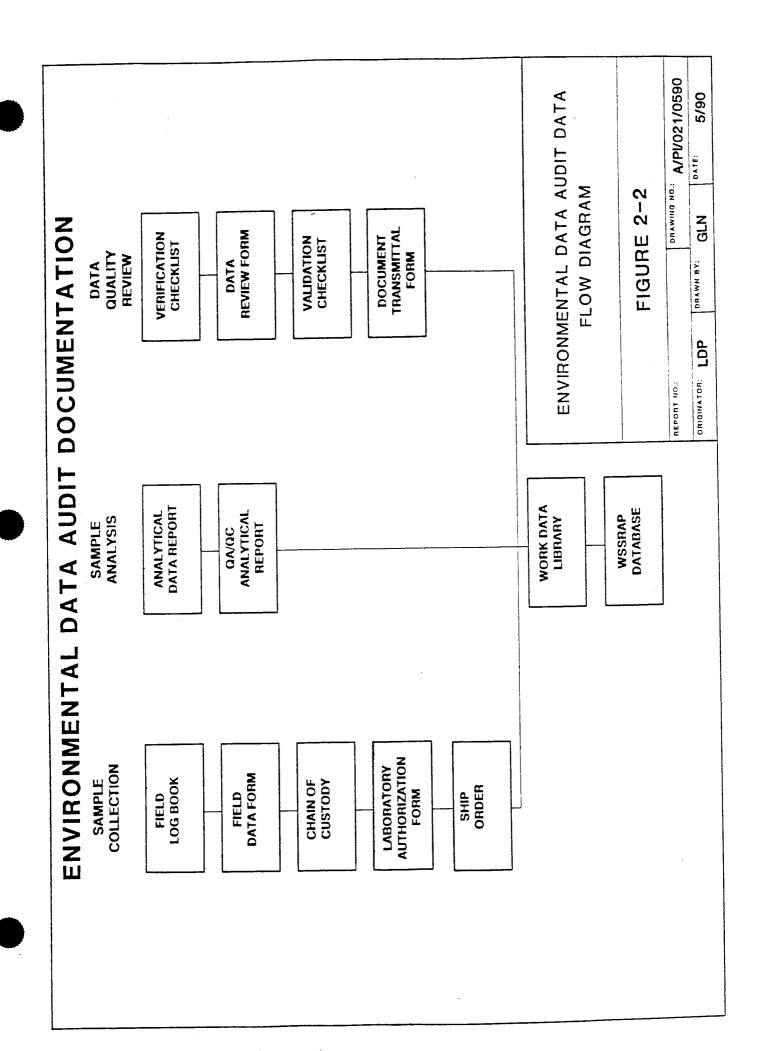
Additionally, all WMG field personnel are required to maintain field logs in accordance with procedure RC-38s. These logs should contain additional information such as discussions and observations that can further support or detail the decision making process logic. When complete, these logs are transferred to the QA department for maintenance as project QA records.

All waste analysis data from detailed laboratory analyses are placed in the generic universal report utility (GURU) data base. Data in the GURU database can be extracted and sorted based on sample identification number, parameter, or any other field definition. Users are allowed to view or copy records, but records cannot be modified or deleted in the GURU system.

A WITS database maintained by the WMG is designed to identify the locations and various types of materials during all phases of the project including final disposition. The WITS data form is used to document original inventory along with all additions and/or changes to the WITS based on field observations, field analysis or consolidation, and data from detailed laboratory analysis. The database includes regulatory class information, U.S. Environmental Protection Agency (EPA) or Missouri Department of Natural Resources (MDNR) waste identification numbers, container type and weight, as well as waste form descriptions.

The waste analysis data audit trail is detailed in Figure 2-2. It should be noted that only project QA-approved laboratories are used for detailed laboratory analysis. Technical specifications for analytical support, which detail the specific criteria for approval, must be used to solicit responses from qualified analytical laboratories. Ability to perform work in accordance with SW-846 and EPA Contract Laboratory Program (CLP) protocol must be included as requirements.

Audits and surveillances, in accordance with procedure QAPP-10, must be performed at laboratories providing analytical support to the WSSRAP to ensure that the laboratory meets the requirements of the technical specification and to monitor laboratory performance.



QA surveillances shall also be periodically performed on various WM activities in accordance with procedure SQA-2a. These surveillances monitor compliance with site policies, procedures, and regulatory requirements.

2.5 Training

All WMG personnel involved in sampling, characterization, or data evaluation must be trained in accordance with the Waste Management Training Plan (MKF and JEG 1992b). Additionally, persons performing analysis in the waste management field laboratory must be trained according to the Laboratory Chemical Hygiene Plan (MKF and JEG 1991a) as required by Occupational Safety and Health Administration (OSHA).

3 SAMPLING METHODS

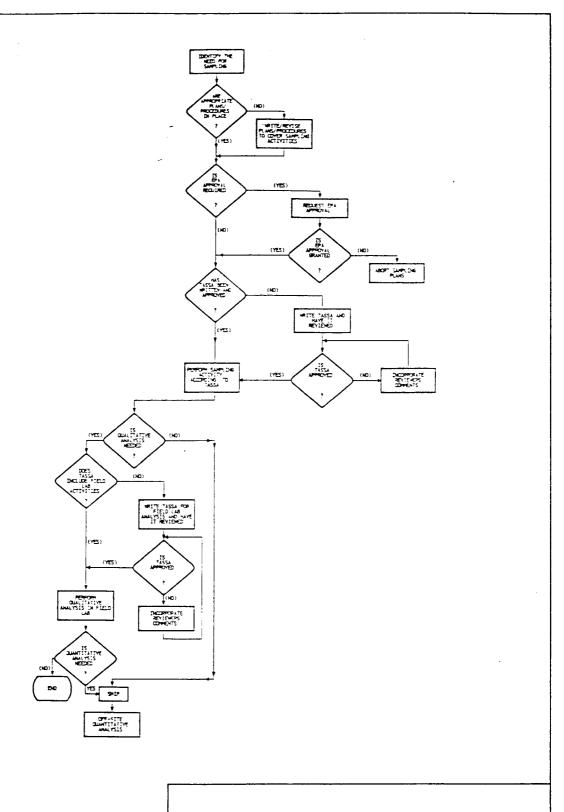
Sampling requires collecting representative samples of the waste and is performed with both accuracy and precision to increase the validity of the waste analysis. The sampling method of choice must be capable of providing both a horizontally and vertically representative sample of the waste material being sampled. The sampling methods and equipment used for sampling waste materials will vary with the form and consistency of the waste materials. The wide variety of waste types and containment devices present at the Weldon Spring site (WSS) makes it impossible to recommend a universal sampling procedure. Procedure RC-24s details the requirements for sampling containers such as drums, bags, cans, and bottles as well as swipe sampling and volumetric sampling of porous materials such as wood and concrete. Detailed sampling plans specific to waste types and configurations will be developed as necessary to support site activities. These plans must detail sampling methods, locations, number of samples, and data quality objectives to be utilized to ensure that sufficient data are obtained to adequately characterize all wastes. Figure 3-1 details the sampling process and sampling plan development logic.

3.1 General

All waste management sampling activities will require that the task be detailed daily as required by procedure CM&O-15a Task Specific Safety Assessment (TaSSA). The TaSSA will specify, at a minimum, the material to be sampled, the sampling equipment to be utilized, and manpower requirements. At a minimum, waste management sampling teams will consist of two persons.

Appropriate air monitoring measurements listed on the TaSSA will be made to assess the possibility of the container holding volatile or flammable constituents and to assist in determining appropriate protective measures to be implemented by the sampling team and all support personnel.

Sufficient volume of a sample, adequate in size for all needs such as laboratory analysis and quality control, and splitting with other organizations, must be collected. Samples for detailed laboratory analysis must be collected in sufficient volume (where possible) to analyze the sample volume specified in the appropriate analytical methodology, including quality control samples. Samples of wastes known to contain high concentrations of hazardous components



REQUIREMENTS PRIOR TO THE SAMPLING PROCESS

FIGURE 3-1

REPORT NO.: EXHIBIT NO.: A/PI/124/0792

ORIGINATOR: LDP DRAWN BY: SRS DATE: 7/92

require only a small aliquot for analysis. Samples requiring preservation (see Table 3-1) will have the appropriate preservation agent added in the field. Sample containers shall be filled to capacity when possible. The use of metal sample containers must be avoided. Wide-mouth brown glass bottles or jars with teflon lined closures are the preferred sample container. In all instances, however, the sample container must be compatible with its proposed contents.

A representative sample is crucial to characterization of the waste and is dependent upon proper selection of sampling points. No single series of sampling points can be specified for all types of receptacles. Table 3-2 lists examples of common types of receptacles used for waste materials and the corresponding recommended sampling equipment. Extreme care must be exercised to overcome stratification or encapsulation that may inhibit obtaining a representative sample. If stratification is suspected, several vertical "core" samples may be required to be consolidated as a representative sample. In the event of suspected encapsulation, several vertical and horizontal cross sectional cuts may be required to determine the degree of encapsulation proper to sampling.

Sampling procedures vary depending on the medium sampled (liquid, semisolid, or solid) and the type of structure containing the waste. The following discussion addresses procedures that are recommended for sampling wastes in various media and types of containers. Environmental compliance procedure RC-24s details the sampling process. When collecting samples for volatile organics analysis, care must be exercised to avoid unnecessary mixing or agitation that may volatilize or lose analytes of interest and lead to the generation of erroneously low results.

3.2 Sampling Strategies

Sampling methods employed at the Weldon Spring Site Remedial Action Project (WSSRAP) will vary depending on the nature and extent of known or suspected contamination, the information desired, and the end use of the data. Simple random sampling will be utilized as a probable statistical technique for obtaining accurate and precise samples when no information on the chemical or radiological properties of a waste are available. In random sampling, every unit in the waste population has a theoretically equal chance of being sampled and measured. The method of choice at the WSS for selecting a random sample is to divide the

TABLE 3-1 Required Containers, Peservation Techniques, and Holding Times For Wastes Samples*

Name	Container ^a	Preservation	Maximum holding time
Inorganic Tests:			
Bromide	P,G	None required	28 days
Chloride	P,G	None required	28 days
Cyanide, total and amenable to chlorination	P,G	Cool, 4°C, NaOH to pH>12, 0.6g ascorbic acid	14 days
Fluoride	P	None required	28 days
Hydrogen ion (pH)	P,G	None required	Analyze immediately
Kjeldahl and organic nitrogen	P,G	Cool, 4°C, H ₂ SO ₄ to pH < 2	28 days
Moisture	P,G	None required	
Nitrate-nitrite	P,G	Cool, $4 \circ C$, $H_2 SO_4$ to $pH < 2$	28 days
Oil and grease	G	Cool, $4 \circ C$, $H_2 SO_4$ to $pH < 2$	28 days
Organic carbon	P,G	Cool, 4°C, HC1 or H ₂ SO ₄ to pH<2	28 days
Phenois	G only	Cool, 4°C, H ₂ SO ₄ to pH < 2	28 days
Phosphorus, total	P,G	Cool, 4°C, H ₂ SO ₄ to pH < 2	28 days
Residue, total	P,G	Cool, 4°C	7 days
Residue, Nonfilterable (TSS)	P,G	Cool, 4°C	7 days
Residue, Settleable	P,G	Cool, 4°C	48 hours
Residue, Volatile	P,G	Cool, 4°C	7 days
Sulfate	P,G	Cool, 4°C	28 days
Sulfide	P,G	Cool, 4°C, add zinc acetate plus	7 days
		sodium hydroxide to pH>9	
Metals:			
Chromium VI	P,G	Cool, 4∘C	24 hours
Mercury	P,G	HNO ₃ to pH < 2	28 days
Metals, except chromium VI and mercury	P,G	HNO ₃ to pH < 2	6 months
Sulfite	P,G	None required	Analyze immediately
Viscosity	P,G	None required	
Sulfur	P,G	None required	
Asbestos	P,G	None required	
Paint Filter Test	P,G	None required	
Heating Value (BTU)	G	None required	

^{*} Aqueous Samples that are highly acidic (pH<2) or causite (pH>12) or concentrated waste samples no preservation is required.

a Polyethylene (P) or Glass (G)

TABLE 3-1 Required Containers, Peservation Techniques, and Holding Times For Wastes Samples (Continued)

Name	Container ^a .	Preservation	Maximum holding time
Organic Tests:			
Purgeable Halocarbons	G, Teflon- lined septum	Cool, 4°C, 0.008% Na ₂ S ₂ O ₃	14 days
Purgeable aromatic hydrocarbons	G, Teflon- lined septum	Cool, 4°C, 0.008% Na ₂ S ₂ O ₃ , HC1 to pH2	14 days
Acrolein and acrylonitrile	G, Teflon- lined septum	Cool, 4°C, 0.008% Na ₂ S ₂ O ₃ . Adjust pH to 4-5	14 days
Phenois	G, Teflon- lined cap	Cool, 4°C, 0.008% Na ₂ S ₂ O ₃	7 days until extraction, 40 days after extraction
Benzidines	G, Teflon- lined cap	Cool, 4°C, 0.008% Na ₂ S ₂ O ₃	7 days until extraction
Phythalate esters	G, Teflon- lined cap	Cool, 4∘C	7 days until extraction 40 days after extraction
Nitrosamines	G, Teflon- lined cap	Cool, 4°C, store in dark, 0.008% Na ₂ S ₂ O ₃	40 days after extraction
PCBs, acrylonitrile	G, Teflon- lined cap	Cool, 4°C	40 days after extraction
Nitroaromatics and isophorone	G, Teflon- lined cap	Cool, 4°C, 0.008% Na ₂ S ₂ O ₃ store in dark	40 days after extraction
Polynuclear aromatic hydrocarbons	G, Teflon- lined cap	Cool, 4°C, 0.008% Na ₂ S ₂ O ₃ store in dark	40 days after extraction
Haloethers	G, Teflon- lined cap	Cool, 4°C, 0.008% Na ₂ S ₂ O ₃	40 days after extraction
Chlorinated hydrocarbons	G, Teflon- lined cap	Cool, 4°C	40 days after extraction
TCDD	G, Teflon- lined cap	Cool, 4°C, 0.008% Na ₂ S ₂ O ₃	40 days after extraction
Total organic halogens	G, Teflon- lined cap	Cool, 4°C, H ₂ SO ₄ to pH<2	7 days
Pesticides Tests:			
Pesticides	G, Teflon- lined cap	Cool, 4°C, pH 5-9	40 days after extraction
Radiological Tests:			
Alpha, beta and radium	P,G	HNO ₃ to pH<2	6 months

^{*} Aqueous Samples that are highly acidic (pH<2) or causite (pH>12) or concentrated waste samples no preservation is required.

[•] Polyethylene (P) or Glass (G)

Sampling Equipment for Selected Waste and Containers TABLE 3-2

	Ď	Drum			į	1	d F		
Waste Type	Bung Type	Open Top	Sacks & Bags	Small Bottles" & Cans	I anks or Bins	waste Piles	Sumps, Fils, oc Lagoons	Pipe	Equipment & Items
Liquids and slurries	(p)	(Q)	A N	Glass Tube	Weighted bottle or Coliwasa	NA A	Weighted Bottle/ Dipper	Dipper /Swipe	Grab/Swipe
Sludges	Trier/ Dipper	Trier/Dipper	NA	Trier	Trier	(2)	(b)	Dipper	Dipper
Moist powders or granules	Trier	Trier	Trier	Thrier	Trier	Trier	Trier	Dipper	Dipper
Day sand, powders, or granules	Thief	Thief	Thief	Thief	(၁)	(c)	Trier/Thief	Dipper	Dipper
Sand or packed solids	Auger	Auger	Auger	Auger	(2)	Thief/ Auger	Auger	Dipper	Dipper/Scoop
Solid sheets or rods	NA	Grab	Grab	Grab	ΝΑ	Grab	A'A	٧×	٧×
Wood	۲×	(c)	NA	NA	NA	(၁)	(၁)	٧×	٧٧
Concrete	NA	Auger	(p)	(þ)	NA	(c)	٧٧	NA	٧×

a Generally less than 10 gallon total volume.
 b Approved sampling equipment (coliwasa, glass tube, dipper, vac sam or peristaltic pump) to be selected based on quantity and homogeneity of waste.
 c Sampling equipment must be selected based on waste conditions and configuration on a case-by-case basis.
 d If loose powder use device as indicated for dry sand, powder, or granules, otherwise use on auger.

population by an imaginary grid, assign a series of consecutive numbers to the units of the grid, and select the units to be sampled through the use of a random numbers table.

This principle can also be applied when the waste population to be sampled consists of a collection of discreet units such as drums, telephones poles, railroad ties, and pieces of equipment. In this instance, each unit is assigned a consecutive number and the units to be sampled are selected through the use of a random numbers table.

Systematic random sampling can be useful when contamination is known (or suspected) and the boundaries of the contamination zone must be determined. In this type of probability sampling, the first sample to be collected from a population is randomly selected, but all subsequent samples are taken at fixed space or time intervals. Table 3-3 indicates the number of samples required for various container or grid populations.

Composite sampling is utilized when the proposed sample population is very large and analytical costs are a factor, as well as when sample volumes from units are insufficient for analysis when taken individually. In composite sampling, a number of random samples are initially collected from a waste and combined into a single sample, which is then analyzed for the chemical contaminants of concern.

Grab sampling is utilized when discreet units are taken as samples and when knowledge is desired on batches from continuous generation operations.

Swipe sampling is used when knowledge is desired about surface contamination. This form of sampling is useful in determining if hazardous residuals from spills or processes may be present on floors, equipment, and process vessels such as tanks and piping.

Volumetric sampling is utilized on porous materials such as wood or concrete to determine the extent of penetration of the contaminant of interest. It involves boring or coring holes into the material to predetermined depths and removing samples for analysis.

A number of factors in addition to statistical considerations will influence sampling strategies. Obviously, one of the most important factors is the waste and its properties. The following waste properties are examples of what must be considered when choosing a sampling strategy.

TABLE 3-3 Sample Quantities for Various Waste Populations

<u>Population</u>	Number of units to be sampled
1-10	5
11-15	7
16-20	8
21-30	10
31-40	13
41-50	15
More than 50	Minimum of 33%

• Physical state: The physical state of the waste will affect most aspects of a sampling effort. The sampling device will vary according to whether the sample is liquid, solid, or multiphasic. If the waste is a liquid, consideration must be given to whether it is viscous or free-flowing. If it is a solid, consideration must be given to whether the solid is hard or soft, powdery, or clay-like.

The sampling strategy will have to vary if the physical state of the wastes allows for stratification, homogenization, or random heterogeneity.

- Volume: The volume of the waste, which must to be represented by the samples collected, will have an effect upon the choice of sampling equipment and strategies. Sampling the raffinate pits requires a different approach from sampling a drum. Although a drum may be sampled with a coliwasa or drum thief, a weighted bottle may be required to sample a 20 ft tank.
- Hazardous properties: Safety and health precautions and methods of sampling and shipping will vary dramatically with the toxicity, ignitability, corrosivity, and reactivity of the waste.

Site-specific factors such as accessibility must be considered when designing a sampling strategy. The accessibility of waste can vary substantially. Some wastes are accessed simply by opening a drum; others may require that a tank be emptied or a container excavated.

3.2.1 Sampling Drum

Access to a drum will affect the number of samples that can be taken from the drum and the location within the drum from which samples can be taken. Bung top drums, which generally indicate the presence of liquids or sludges, limit access to the contained waste and restrict sampling to a single vertical plane.

Such drums will be positioned with the bung in an upright position. The bung will be slowly loosened using a non-sparking bung wrench allowing any gas pressure to release. If the bung cannot be removed or loosened it may be necessary to remove the top of the drum or other method to gain access to the material.

If the material is known to be homogeneous, a coliwasa, glass tube, vacsam sampling pump, or peristaltic sampling pump may be used to collect the sample. However, for most drums on the WSS the extent of homogeneity of the material cannot be verified and mixing of liquids will be required prior to sampling with the above equipment.

For sludge or solids, the top of the drum will be removed to allow better access to the material. The surface of the waste will then be divided into an imaginary grid composed of eight sections as detailed by procedure RC-24s and selecting a minimum of five grid points to be sampled. Each selected grid point will then be sampled in a vertical manner along the entire length from top to bottom of the drum. These samples will then be composited to form a single sample.

3.2.2 Sampling Barrels, Fiberdrums, Bags, or Sacks, Containing Powder, Granular, or Packed Solids

Where possible containers will be positioned upright prior to sampling. If possible, sacks or bags will be sampled as found to avoid rupturing them and spilling the contents.

The container will be opened slowly and in such a manner so as to minimize the generation of airborne contaminants. For drums, the surface of the waste will be divided into an imaginary grid composed of eight sections and a minimum of five grid points will be selected for sampling. Each selected grid point will then be sampled in a vertical manner along the entire

length from top to bottom using an auger, trier, thief, scoop, or shovel. A composite sample will then be created from these samples.

3.2.3 Sampling a Tank

Tanks can generally be viewed as large containers or drums. Thus, in many cases, the principles that are applied to sampling a drum can also be applied to sampling a tank.

The majority of tanks found at the WSSRAP are found at the Weldon Spring Chemical Plant (WSCP) and were present when the plant was closed. These tanks have been visually inspected and the majority were found to be empty.

For tanks located within the structures at the WSCP, a team of representatives from various departments of the Project Management Contractor (PMC) is required to assess the best means of approaching and gaining access to the tank because of the generally poor physical condition of the buildings. This assessment will also attempt to determine the best location to obtain the sample. Normally the sample can be obtained through the sampling hole or hatch which is usually located at the top of the tank. In some instances it may be necessary to remove a flange or a pipe to gain access to a tank. If no other access point is available, a sample may be obtained through a bottom valve. Such a sample will generally not be representative of the overall tank contents and must be documented on the waste management sampling data form as required by procedure RC-24s.

For tanks containing liquids, a measurement of the depth of the liquid in the tank shall be made using a weighted line or rigid pole and measuring the length of stain on the device.

If a tank is equipped with a recirculating pump, the pump should be activated and the contents of the tank allowed to mix for a minimum of 5 min prior to sampling. A sample can then be collected using a coliwasa, peristaltic pump or a vacuum pump.

If the tank is not equipped with a recirculating pump, a sample will then be collected from selected depths using a weighted bottle depending on the estimated volume of waste in the tank. A coliwasa can also be used to sample in a vertical manner along the entire length from top to bottom if a coliwasa of sufficient length is available.

In some instances, it may be more feasible to obtain a sample for detailed analysis as the contents are being transferred to new containers or drums. In this instance, samples can be collected at random intervals during the transfer process or by collecting samples from the containers or drums after the transfer is complete.

If the tank is an open one, allowing unrestricted access, then a representative set of samples is usually obtained by dividing the tank into an imaginary three-dimensional grid and assigning sections numbers. Specific levels and grid locations are then sampled using a random number table.

3.2.4 Sampling a Pit, Pond, or Sump

Wastes found in pits, ponds, or sumps may range from liquids to dried sludge residues. Liquid or semi-solid wastes in these configurations are often best sampled using strategies applicable to large tanks.

If the pit, pond, or sump is sufficiently small, it may be sampled from the banks or edges or by use of a catwalk or bridge. For pits or ponds greater than 12 ft across, a boat or other such device may be required. In either case, the requirements for working on or near water as detailed in procedure SAFE-8a must be met.

The surface of the area to be sampled will be divided into an imaginary grid. The number of grid sections will be determined by the desired number of samples to be collected. Samples will be collected from random points on the grid and in various depths in accordance with procedure ES&H 4.3.1s. Based on the findings of these activities, composite samples will be created by combining compatible samples as determined by RC-26s.

Compressed air or other circulating devices may also be utilized to mix the waste within a pit or sump in order to improve the homogeneity of the waste prior to sampling. In this manner any solids or sediments that may have settled will be resuspended or dissolved prior to sampling.

A ponar sampler lowered to the bottom of the pond, pit, or sump by a rope or cable may be used to sample sediments or sludges that may have settled if that is to be the focus of the sampling activity.

A weighted bottle may be used to sample liquids. A peristaltic or vacuum pump may be used if the waste can be made homogeneous. A coliwasa may even be used if the pit, pond, or sump is sufficiently shallow.

Hollow-stem augers and/or split-spoon samplers are appropriate for sampling solids. Water-driven or water-rinsed coring equipment should not be used for sampling because the water can rinse chemical components from the sample. Excavation equipment may be useful in obtaining samples at various depths or in gaining access to buried targets. The resulting holes may be useful for viewing and recording information on buried debris or stratified materials.

3.2.5 Sampling a Waste Pile

A waste pile can range from a small heap to a large aggregate of wastes. The wastes are predominantly solid and can be mixtures of powders, granules, and large chunks and may be composed of wood, metal, plastic, or other debris. Obtaining a representative sample through the employment of a universal sampling procedure is not possible and therefore must be addressed on a case-by-case basis. When samples are required from a waste pile, a unique sampling plan will be prepared by the Waste Management Group in an effort to obtain accurate and representative samples.

The accessibility of waste within a waste pile is usually a function of pile size. Ideally, piles containing unknown wastes are sampled using a three-dimensional random sampling strategy. If sampling is limited to certain portions of the pile, then the collected sample will be representative only of those portions, unless the waste is known to be homogeneous.

Samples from the temporary storage area (TSA) or material storage area (MSA) should be taken from the truckloads used to create the pile prior to the waste being placed in the pile, where possible.

The sampling devices most commonly used for small piles include thief, triers, and shovels.

3.2.6 Sampling a Pipe

Sampling the contents of a pipe can present unique and difficult problems. Identifying a sampling point that may yield a representative sample may not be an easy task.

Available drawings from the WSCP, which detail the piping configurations, will be reviewed in an effort to tentatively identify the materials that may be present in the piping system as well as potential sampling points. If sufficient information cannot be obtained from these drawings, visual inspections by a group of representatives from various PMC departments will be performed to determine appropriate sampling points. This assessment is detailed in the Building Characterization Work Plan (MKF and JEG 1991b).

A hole can be drilled in the top of a pipe to gain access if no valve, fitting, or flange is present at or near the chosen sampling point. The hole must be sufficiently large to allow access to the waste by the chosen sampling device.

For active piping systems, sampling ports or valves must be utilized to obtain samples from the process.

Swipes may be utilized to determine the presence of residues within piping systems.

3.2.7 Volumetric Sampling of Concrete and Wood

Many areas at the WSSRAP have been found to be chemically contaminated from spills that occurred during the active life of the facility. For porous surfaces such as concrete and wood, it is imperative that the extent of migration of these materials be determined to determine management practices for these wastes.

For concrete surfaces and flat wood surfaces, the surface area to be sampled will be divided into an imaginary grid. The number of grid sections will be determined by the desired number of samples to be collected in accordance with guidelines established in SW-846. Samples will be collected by boring at random points on the grid to various depths in accordance with procedure RC-24s.

For wood materials such as railroad ties or telephone poles, random boring samples from various locations within the material will be consolidated. The number of samples to be collected and the number of composites required shall be determined based on the population available to be sampled.

For floors and flat surfaces, samples will be collected in areas where visible residues are on the floor and/or swipe sample data has shown residues to be present by establishing an imaginary grid system. Samples can be collected using a drill with masonry bits and a scoop or vacuum pump to collect the material.

Samples will be collected by drilling to a predetermined depth at selected points within the sample location. Sampling at selected increments should continue to a predetermined maximum depth. After each depth increment has been completed and the sample collected, the sample location area must be wiped clean of any residues using a paper towel or masselin cloth.

A minimum of five and a maximum of 20 spots should be composited within each grid location chosen for sampling. Less than five spots generally will not yield enough material to compromise a sample. However, if a sample location has been identified where the total surface area is less than 4 in.², it may not be possible to sample more than five spots. Using 20 samples as a maximum gives a valid representation of a large area. The representation will be optimum if the entire area is visibly oil stained and/or swipe data indicate the presence of residues in the entire area to be sampled. For areas where no visible stains are present, a grid system will be used in accordance with procedure RC-24. Regardless of size or how the area is laid out, all samples should be collected using a symmetrical method of sampling.

Most floors on site are structurally sound. However, some areas of known contamination may be cracked throughout. If this is the case, sampling must be performed along a representative distance of the crack. All sample locations will be identified on respective building drawings. These drawings will be included in the final characterization report.

3.2.8 Sampling Soil

The techniques for soil sampling are numerous. The procedure utilized at the WSS is detailed in procedure ES&H 4.4.5s. This procedure is consistent with the objective of collecting

soil samples to determine the amount of hazardous material deposited on a particular area of land.

3.2.9 Sampling for Determining the Radioactive Component of a Waste

Before wastes can be released off-site to an unlicensed facility, they are required to be certified to have nondetectable or background levels of radioactivity in accordance with procedure RC-32s. This requirement applies to all wastes originating and/or stored within the radioactive material management area (RMMA) and applies only to radionuclides from the natural U-238, U-235, and Th-232 decay series.

For wastes with a potential for volumetric contamination, at least three representative samples, adequate in size to perform radiological parameters, must be taken from the waste quantity. If radionuclides are heterogeneously dispersed within the waste, the coliwasa sampler or similar method must be used to obtain sample aliquots from the top, middle, and bottom of the waste quantity in accordance with RC-24s.

At least three samples must be taken from a matrix similar to the waste matrix except each matrix shall be known to be free of radioactivity in excess of naturally occurring levels. Each matrix will be approved on a case-by-case basis by the Waste Management Manager or his designee. For example, motor oil purchased from an auto parts store is similar to oils used at the WSS. Samples from the matrix must be collected in the same manner as potentially contaminated samples.

Samples must be analyzed radiochemically by an approved method detailed in this plan to measure radioactivity in excess of the lower limit of detection (LLD) in samples known to be free of radioactivity in excess of natural background levels.

Data must be evaluated and documented in accordance with the guidelines contained in procedure RC-32s.

Waste materials can be determined not to contain radioactivity based on process knowledge. Process knowledge includes information on the origin, storage, use, and potential exposures of the waste material. Process knowledge principally applies to sealed containers (e.g., pressurized paint cans, laboratory reagents, and motor oils); however, bulk materials may

also be included. For example, large containers used as stock may be used for dispersal into smaller containers.

Criteria for designating materials nonradioactive per process knowledge are:

- The material must originate outside the RMMA. The approximate date the material was received on site must be recorded on the certification form. Information on purchased materials can be obtained through the Procurement Department.
- Documentation showing that the material has been tracked and controlled since received on site by specifically identifying the time and on-site locations during the life of the material.
- Proof that the material has not been exposed to known radioactivity. Possible exposures include direct contact with contaminated waste via mixing and exposure to airborne contamination via open containers. A statement on the use of the material must be included on the certification form.

4 WASTE ANALYSIS

Chemical analysis of the waste materials at the Weldon Spring Site Remedial Action Project (WSSRAP) is intended to identify hazardous characteristics or hazardous components present in each waste. It is also designed to provide information to ensure safe storage of the materials in compliance with all applicable regulations. In some cases, analytical data will provide the basis for choosing an appropriate means of treatment or disposal for a particular material.

The analytical procedures chosen for this Waste Analysis Plan can be divided into two groups: (1) field analyses; and (2) laboratory analysis.

Field analyses are intended to provide general information such as functional groups, general hazard class, and chemical compatibility for the purposes of consolidation and confirming label information if necessary. These tests, which will be performed at the Weldon Spring site (WSS), can also be used to determine the appropriateness of additional laboratory analyses.

Laboratory analysis, where appropriate, will provide more detailed information about the hazardous components or constituents of the waste. Laboratory analysis may also be required to determine exact concentrations of materials shown to be present during field analyses. In most instances, detailed laboratory analysis will be performed on containerized wastes after wastes have been consolidated or placed in their final storage containment.

The following sections contain a discussion of the analytical methodology appropriate to the general categories of wastes at the WSSRAP as well as the logic for choosing each analysis. It is conceivable that, during the course of this project, additional methodologies and technologies will become available. As these methodologies are identified and determined to be amenable to the wastes at the Weldon Spring site, they will be procedurialized, reviewed and approved by the Project Management Contractor (PMC).

4.1 Field Analyses

The procedures employed for chemical categorization of wastes in the field are organized into the procedural sets summarized below. These qualitative procedures are designed to provide

general information concerning the material being tested. These tests are detailed in procedure RC-21s and RC-37s. If specific analyte concentrations are desired, representative portions of the waste must be submitted for laboratory analysis.

Knowledge of the functional groups present may be sufficient to determine the gross composition of the waste and minimize the need for more extensive laboratory analysis by allowing classification into a given reactivity group (see Table 4-1). The appropriateness of using any of the field testing procedures, prior to or in lieu of more sophisticated instrumental techniques, is left to the professional judgement of the supervising engineer. Figure 4-1 shows an example of a decision tree for determining the appropriateness of any field test to any given waste.

4.1.1 pH

Measurement of the pH aids in the identification of acids and bases and allows determination of proper handling, packaging and storage requirements. pH also indicates whether neutralization may be required as part of the treatment or disposal process (see Figure 4-1).

4.1.2 Redox

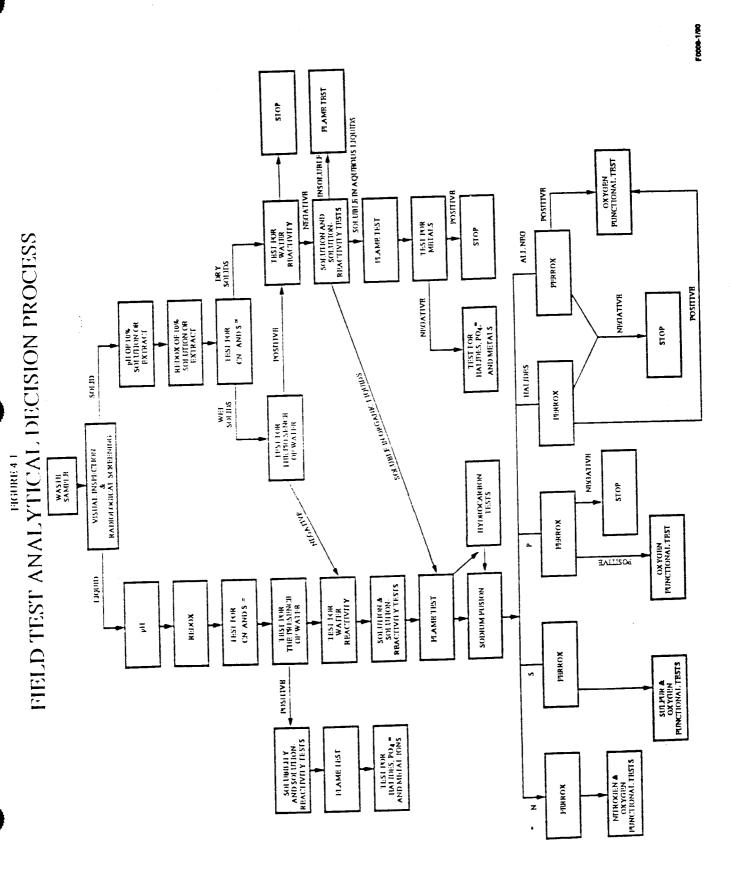
The oxidizing or reducing potential of a waste material is identified by this procedure. Due to the reactive nature of oxidizing and/or reducing agents, it is essential that these types of materials receive special packaging, handling and storage considerations. These materials must be segregated from all incompatible materials during storage (see Figure 4-2).

4.1.3 Solution-Reactivity and Special Functionality Tests

Treatment of the waste materials with water, a base, and various acids and solvents allows determination of its reactivity and solubility characteristics. This procedure allows identification of sulfides and cyanides, reactivity and solubility in acids and solvents, reactivity with water and the presence of water (see Figure 4-3).

TABLE 4-1 Reactivity Group Definitions

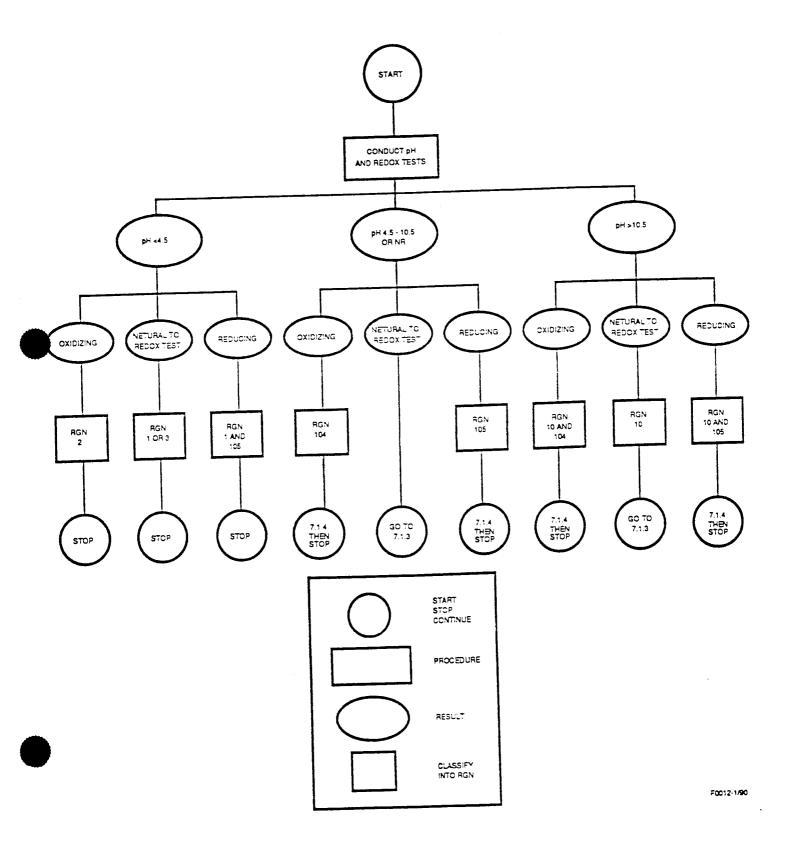
Reactivity Group No.	Reactivity Group Name
1	Acids, mineral, nonoxidizing
2	Acids, mineral, oxidizing
3	Acids, organic
4	Alcohols and glycols
5	Aldehydes
6	Amides
7	Amides, aliphatic and aromatic
8	Azo compounds, diazo compounds, and hydrazines
9	Carbamates
10	Caustics
11	Cyanides
12	Dithiocarbamates
13	Esters
14	Ethers
15	Fluorides, inorganic
16	Hydrocarbons, aromatic
17	Halogenated organics
18 19	Isocyanates Ketones
20	Mercaptans and other organic sulfides
20	Metals, alkali and alkaline earth, elemental and alloys
21	Metals, other elemental and alloys in the form of powders, vapors, or sponges
23	Metals, other elemental and alloys as sheets, rods, moldings, drops, etc.
24	Metals and metal compounds, toxic
25	Nitrides
26	Nitriles
27	Nitro compounds
28	Hydrocarbons, aliphatic, unsaturated
29	Hydrocarbons, aliphatic, saturated
30	Peroxides and hydroperoxides, organic
31	Phenois and cresois
32	Organophosphates, phosphothioates, phosphodithioates
33	Sulfides, inorganics
34	Epoxides
101	Combustible and flammable materials, miscellaneous
102	Explosives
103	Polymerizable compounds
104	Oxidizing agents, strong
105	Reducing agents, strong
106	Water and mixtures containing water
107	Water reactive substances
108	Radioactively contaminated substances

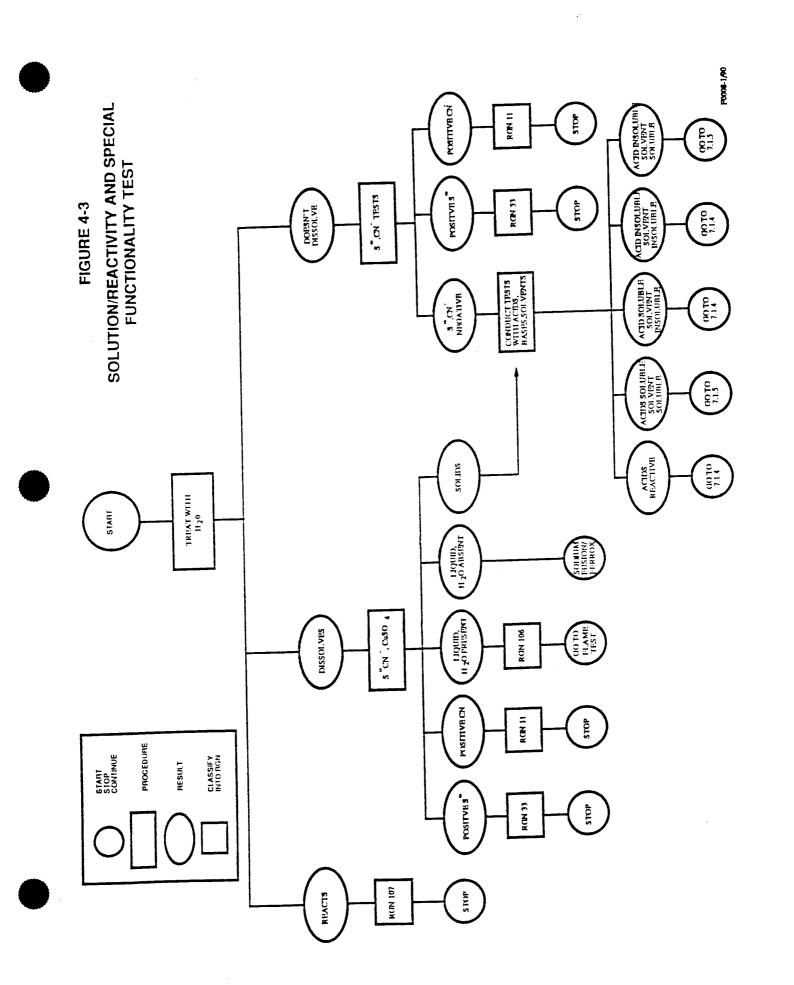


. =:

FIGURE 4-2

PH AND REDOX TESTS





4.1.4 Flame Test

Observation of the behavior of a material upon ignition can provide a great deal of insight regarding its composition. The combustibility of a material as well as its explosive tendencies can be determined using this procedure. This test can also aid in classification as inorganics or organics (see Figure 4-4).

Halogenated organics presented in a waste may create a characteristic flame during this test.

4.1.5 Sodium Fusion

The sodium fusion test, which is performed on organic waste materials, allows determination of the presence of certain halogens (chlorine, bromine or iodine), sulfur, nitrogen and/or phosphorus (see Figure 4-5). The results obtained from this procedure and the ferrox test are used as a guide for which specific functionality tests are to be performed.

4.1.6 Ferrox Test

The ferrox test is performed on organic waste materials, in the absence of sulfur, nitrogen, or phosphorus, to determine the presence of oxygen (see Figure 4-5).

4.1.7 Organic Functionality Tests

This set of procedures includes specific tests, subdivided into four major subsets, for determination of 21 reactivity groups. Each of the procedure subsets for nitrogen or sulfur is performed only if the respective element is determined to be present in the sodium fusion test. If the element was found to be absent, it can be concluded that all reactivity groups determined in that subset are absent. Oxygen is assumed to be present if sulfur, nitrogen, or phosphorus is confirmed by the sodium fusion test; then the oxygen subset is performed in addition to the other appropriate functionality test.

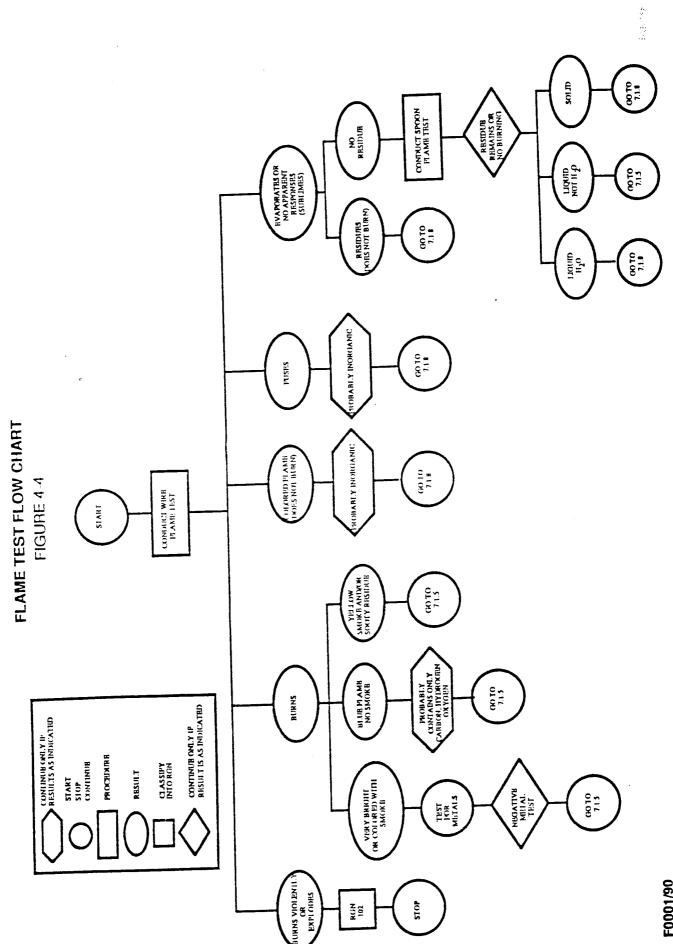


FIGURE 4-5
SODIUM FUSION AND FERROX TEST FLOW CHART

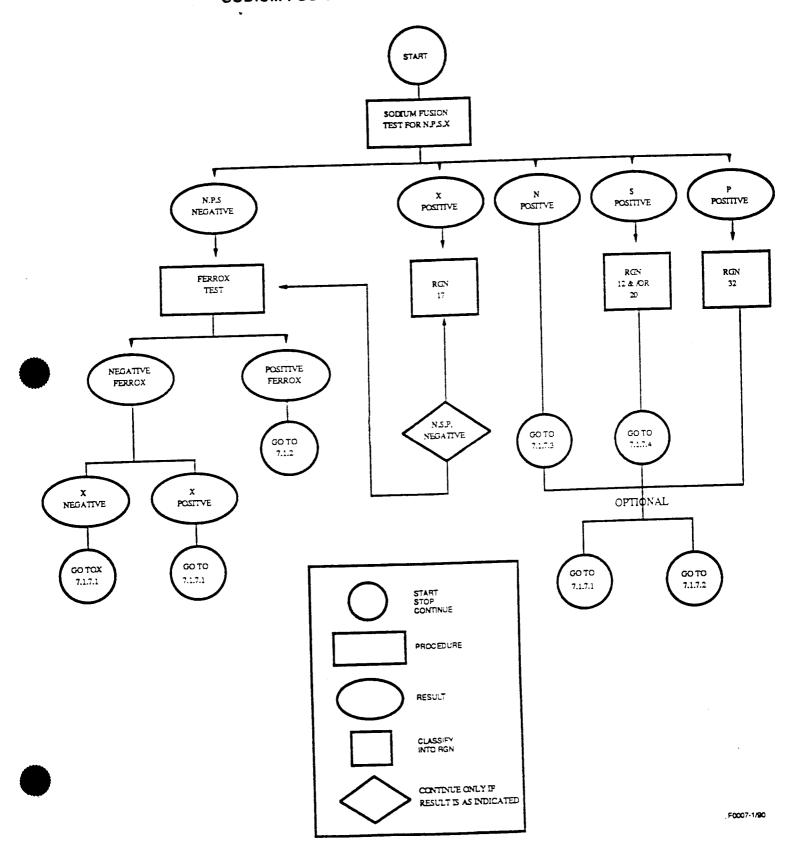
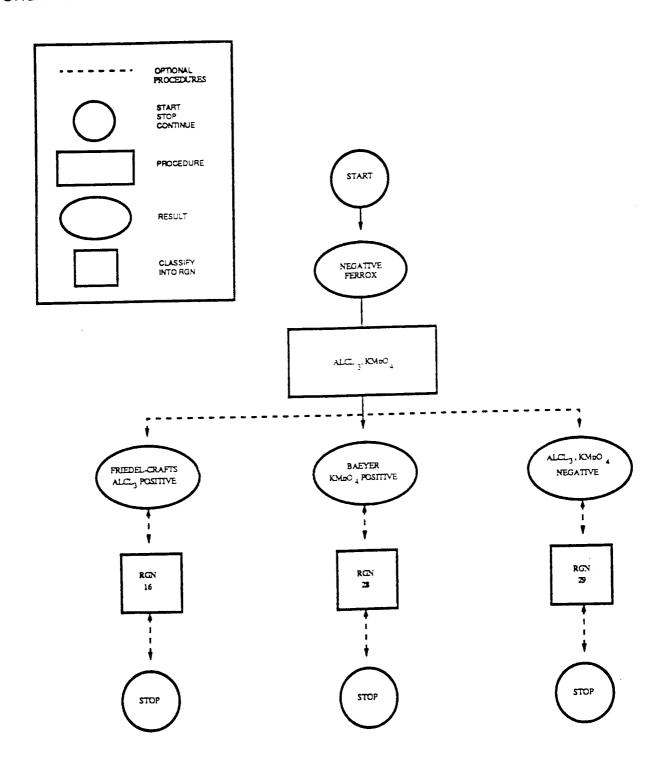


FIGURE 4-6
ORGANIC FUNCTIONALITY TESTS FOR HYDROCARBON CLASSIFICATION FLOW CHART



4.1.7.1 Hydrocarbon Functionality Test

Saturated and unsaturated hydrocarbons and/or aromatic hydrocarbons are distinguished by this procedure. Due to the mutually exclusive nature of these reactivity groups, if one test is positive, the other need not be run (see Figure 4-6).

4.1.7.2 Oxygen-Containing Organic Functional Group Test

Nine specific reactivity groups can be distinguished by using this procedure. These tests are performed on all organic wastes except those shown to be negative for sulfur, nitrogen and phosphorus by the sodium fusion test and a negative ferrox test (see Figure 4-7).

4.1.7.3 Nitrogen-Containing Organic Functional Group Test

The tests for the seven reactivity groups in this subset are performed only if nitrogen is detected by the sodium fusion procedure (see Figure 4-8).

4.1.7.4 Sulfur-Containing Organic Functional Group Test

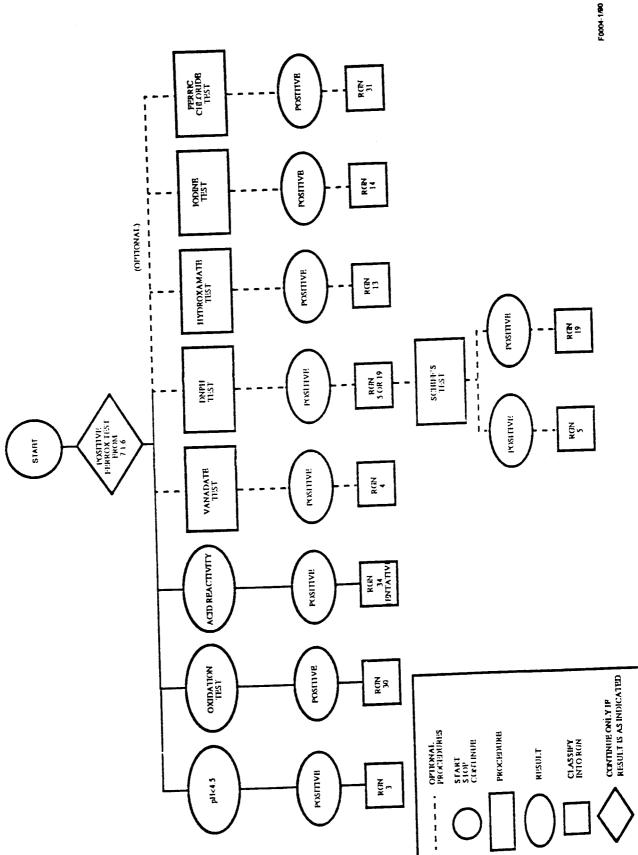
This procedure distinguishes the two reactivity groups contained within this subset (see Figure 4-9). If sulfur is shown to be absent in the sodium fusion procedure, this procedure need not be performed.

4.1.8 Inorganic Functionality Test

The presence of elemental metals, heavy metal compounds, halides, phosphates, and/or organic fluorides are confirmed by this procedure (see Figure 4-10). Additionally, semi-quantitative analyses can be performed in accordance with procedure RC-37s for selected parameters on aqueous and water soluble materials.

4.1.9 Geiger-Müller Counter Radiological Survey

Surface radioactivity is determined in the field using a Geiger Müller (G-M) counter as detailed in procedures ES&H 2.3.8 and 2.6.3. This will serve to indicate gross levels of contamination and determine the applicability of additional laboratory testing. If the GM survey



NITROGEN-CONTAINING ORGANIC FUNCTIONAL GROUP TEST FLOW CHART FIGURE 4-8

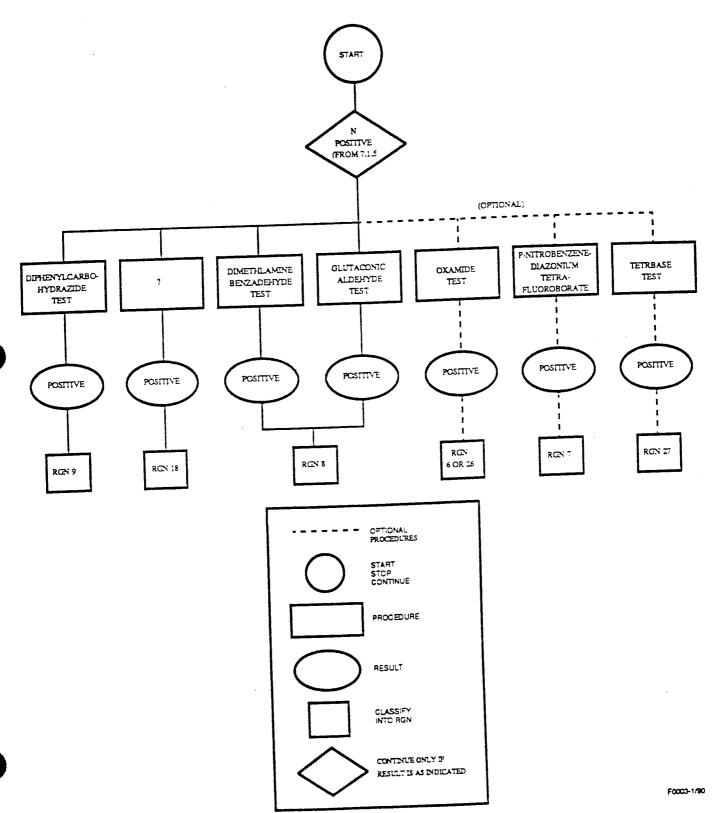
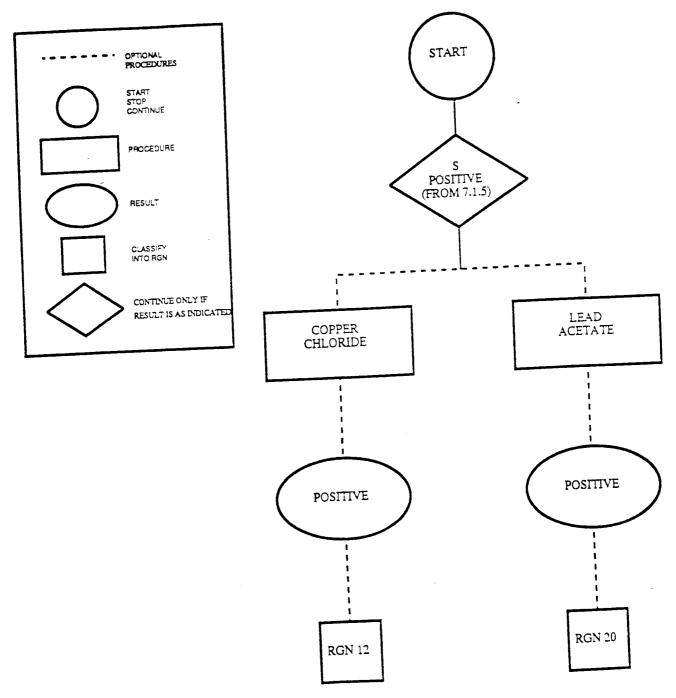
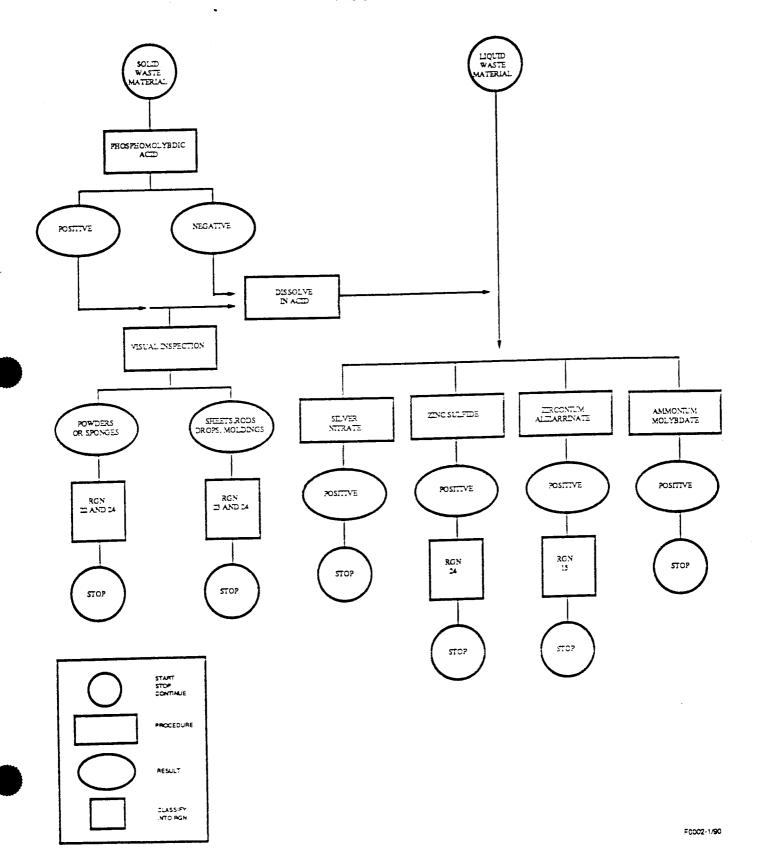


FIGURE 4-9
SULFUR-CONTAINING ORGANIC FUNCTIONAL GROUP TEST FLOW CHART



INORGANIC FUNCTIONALITY TEST FLOW CHART

FIGURE 4-10



indicates activity in excess of natural background, no further actions are required. If the GM survey indicates only natural background, then a representative portion of the waste will be analyzed utilizing the GeLi detector in the WSSRAP laboratory as detailed in procedure ES&H 2.6.9s.

4.1.10 Visual Examination

Visual examination of waste materials will aid in consolidation and provide insight into the appropriateness of field or laboratory analysis.

4.1.11 Polychlorinated Biphenyls (PCBs)

This procedure works on the principle of chloride determination. PCBs, which contain chlorine, can be detected by this test; however, any other organic chlorine-containing compound will also be detected by this test and may cause a false positive result. All waste materials that yield a positive result shall be confirmed by a detailed laboratory analysis.

4.1.12 Flash Point

The flash point is necessary to determine regulatory classifications of organic liquids and to ensure safe handling and storage of flammable or ignitable materials.

4.1.13 Field Compatibility Determination

All materials will be verified to be compatible prior to consolidation to ensure employee safety and verity field analytical data. Prior to any consolidation, small representative portions of waste materials will be combined as indicated in Procedure RC-26s.

4.2 Laboratory Analyses

Quantitative analysis is performed in the laboratory to identify hazardous components present in wastes. The presence of listed substances must be determined to allow proper management of WSSRAP wastes.

Chemical analysis data will assist in the development of treatment and/or disposal options, development of burn plans for incinerable wastes, providing data requested by disposal facilities, and confirmation of field analysis data.

The analytical regime specified in this section is comprehensive and should be adequate to provide the information required. Should other preferable technologies evolve in the near future, they will be taken into consideration. Figure 4-11 details the decision process for determining appropriate analytical testing for organic materials and Figure 4-12 details the decision process of aqueous liquids and inorganic solids. The process of determining appropriate analytical testing is complex and requires a great deal of judgement on the part of the Waste Management Group. No strict analytical regime can be prescribed for any given classification but must be evaluated for each waste on a case-by-case basis.

4.2.1 Suspended Solids

Suspended solids can affect the handling and pumping characteristics of the waste during treatment and their measurement is used as a fundamental indicator of the suitability of the waste for various treatment or disposal options.

4.2.2 Total Metals

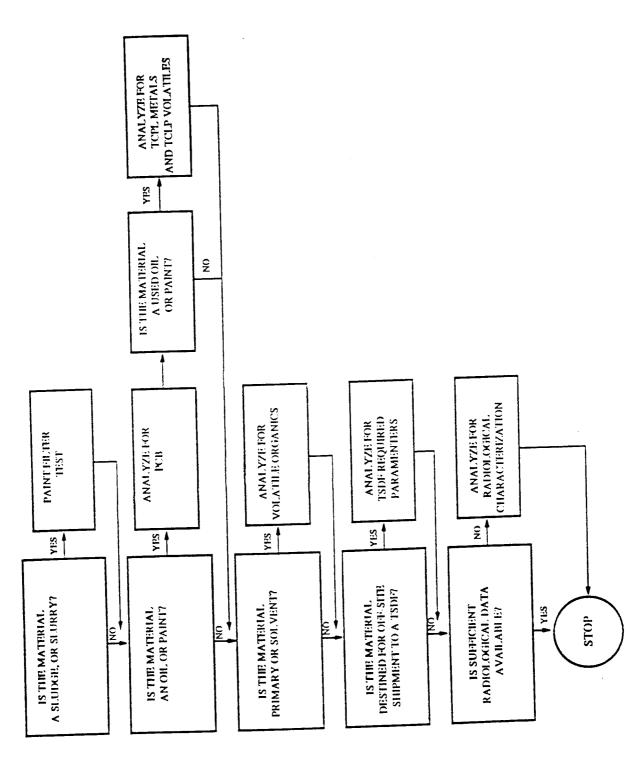
This analysis meets the requirements to identify hazardous metal constituents identified in Appendix VIII to 40 CFR 261 which is a prerequisite to treatment, storage and disposal facility (TSDF) acceptance for final treatment or disposal. TSDF Part B permits also specify limits on the level of heavy metals in the waste that the incinerator can accept to ensure that a release of toxic metals to the environment does not occur during incineration. Concentrations of total metals may also indicate the potential of a waste to be characteristically hazardous by the toxic characteristic leaching procedure (TCLP).

4.2.3 Volatile Organic Analysis by Gas Chromatography/Mass Spectroscopy (GCMS)

This analysis is designed to identify and quantify volatile organic analytes which appear on the Comprehensive Environmental Response, Compensation and Liability Act (CERCLA) Hazardous Substance List (40 CFR Part 302.4 Table 302.4) of which the Appendix VIII substances are a subset. This data is also used to determine whether those wastes proposed for

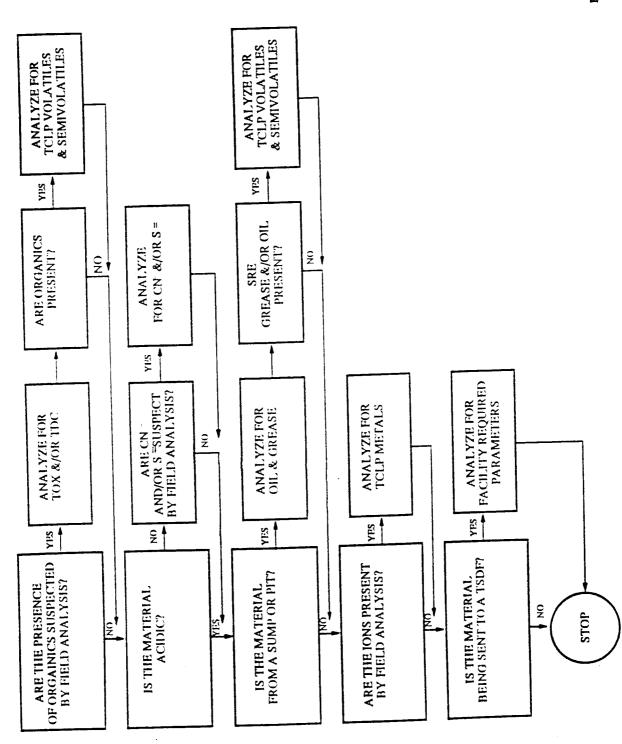
ORGANIC WASTE ANALYTICAL DECISION PROCESS

FIGURE 4-11



INORGANIC WASTE ANALYTICAL DESIGN PROCESS

FIGURE 4.12



incineration can be destroyed by the incineration process. This analysis can also be used to detect compounds that may cause the waste to be characteristically hazardous by TCLP and therefore require further analyses by TCLP.

4.2.4 Semi-Volatile Organic Analysis by Gas Chromatography/Mass Spectroscopy (GCMS)

An analysis for these compounds satisfies permit and regulatory requirements to identify organic components which appear on the Appendix VIII list in 40 CFR 261.

4.2.5 Moisture Content

Water content can be used to determine storage compatibility and handling requirements and to provide data for material balance calculations.

4.2.6 Specific Gravity

The specific gravity value is important in calculating material balance across a treatment process and to give a true value of the amount of materials being handled.

4.2.7 Ash Content

This information is important for evaluating wastes for potential slag and ash formation, to assess particulate loading, and to verify that the slag and ash handling system is adequate.

4.2.8 Heating Value

This data is used in determining the acceptability of a waste for incineration, and to ensure that the incinerator heat load is maintained at the proper operating range.

4.2.9 Corrosivity

A material's corrosivity to steel indicates the appropriate type of container to use to avoid incompatibilities and container failures and to dictate regulatory classification.

4.2.10 Elemental Analysis (Carbon, Nitrogen, Sulfur, Phosphorus, Fluorine, Chlorine, Bromine, and Iodine)

Knowledge of these parameters is necessary for determination of incinerator system conditions, computation of stoichiometric air requirements, and to ensure adequacy of pollution control equipment.

4.2.11 Reactivity (Cyanide or Sulfide)

The presence or absence of cyanide and sulfide must be known to assure safe handling and storage, and to determine if treatment to destroy or stabilize these compounds is warranted. Analysis of low pH (≤ 2) materials is unnecessary since, as a rule, materials with an acidic pH will have reacted and liberated any HCN and H²S present and will not contain cyanide or sulfide.

4.2.12 Toxicity Characteristic Leaching Procedure (TCLP)

TCLP should be performed on all wastes and the solid created after stabilization of liquids and unknown wastes, if it is necessary to determine if the material meets the regulatory definitions of a characteristic waste or the requirements for land disposal in 40 CFR Part 268.

4.2.13 Total Organic Halide (TOX)

The data from this analysis can be used to screen aqueous wastes that may contain levels of halogenated organic contaminants that may not be detectable during field analysis and may need further organic constituent analysis.

4.2.14 Total Organic Carbon (TOC)

This procedure is used to screen aqueous wastes for low level organic contamination that may not have been detected during field analysis. It will also be used to determine if a more specific analysis for organic species may be required.

4.2.15 Paint Filter Test

This will provide information regarding the amount of free liquids that are present in the waste. This information can be used for insuring proper storage and secondary containment.

4.2.16 Radiological Characteristics

Knowledge of the concentration of uranium, thorium, and other appropriate radioactive isotopes is necessary to assure that the limits specified in any specific TSDF Part B or National Emissions Standards for Hazardous Air Pollutants (NESHAP) permit are not exceeded. This information is also for safe handling and storage.

4.2.17 Viscosity

Knowledge of the viscosity of a waste material will allow determination of the type of pump required to handle the waste as well as the need for blending the waste to achieve a more pumpable mixture.

4.2.18 Polychlorinated Biphenyls (PCBs)

The analysis will verify the presence or absence of PCBs and provide insight to the applicability of TSCA regulations to the management of the waste material.

4.2.19 Oil, Grease and Total Petroleum Hydrocarbons (TPH)

This analysis will be limited primarily to samples obtained from sumps and underground storage tanks and will be used to identify treatment or disposal options and to verify that cleanup and/or removal criteria are met.

4.2.20 Dioxin

Dioxin analyses will be performed on those wastes, such as wood, suspected of being treated with chemicals that may have been contaminated with dioxin. The presence of dioxin in a waste will greatly influence the disposal options available.

4.2.21 Asbestos

For materials suspected of asbestos contamination, knowledge of the asbestos content and asbestos type is required to ensure that the material is properly containerized and labeled in accordance with regulatory requirements.

4.2.22 Polyaromatic Hydrocarbons

An analysis for these compounds satisfies regulatory requirements to identify specific organic components that appear on the Appendix VIII list.

4.2.23 Nitroaromatics

Analysis for nitroaromatics will indicate the presence of potentially explosive compounds in soils or wastes that may have been contaminated as result of Weldon Spring Ordnance Works activities. Knowledge of the level of these contaminants is necessary to determine if cleanup criteria for the materials apply or have been met.

4.3 Recharacterization

Recharacterization of site generated wastes is required to verify that the concentration of constituents has not changed significantly, new constituents have not been added, or that the physical properties have not changed. Since the WSS is an inactive facility, recharacterization will not be required for the majority of wastes managed at this project. Wastes generated by site activities, such as field and radiological laboratory waste, will receive full characterization prior to placement into Building 434 for storage.

Other wastes such as purge and development water are generated from wells and areas where sufficient analytical data are maintained from required monitoring activities. Therefore, recharacterization can be evaluated based on data available from the previous quarter.

4.3.1 Fingerprint Screening

Fingerprint screening consists of selected tests that can be performed at the WSS to verify that the gross chemical composition of the waste has not changed significantly. For aqueous and

inorganic wastes, these tests consist of pH, screening for oxidation potential, cyanide, sulfide, and metal ions. Organic materials are tested for the presence of water, sodium fusion, appropriate functional group, and flash point for liquids. The appropriate tests will be run on randomly selected drums of wastes stored in Building 434.

5 SAMPLE MANAGEMENT

Sample management begins as soon as the sample is collected and continues during the transportation to the laboratory, through the analytical phase, and culminates with the return of the sample to the Weldon Spring site (WSS).

As discussed previously, waste samples are collected in accordance with procedure RC-24s. Chain of custody is initiated at this time and documented in accordance with procedure ES&H 4.1.2a. Chain of custody is maintained throughout the packaging phase and custody is transferred to the analytical laboratory to allow for completion of the requested analyses.

All waste management samples must be assigned a unique identifier as follows:

WM-AAAA-BBBBBB-DDD

where

"WM" denotes that this sample is a waste sample taken for characterization purposes.

"AAAA" represents a four digit code denoting the sample location or original container identification.

For containerized wastes that have been assigned waste inventory tracking system (WITS) identification numbers, AAAA must be that number.

For bulk waste samples or containerized wastes that have not been assigned a WITS identification number, the following shall apply to the AAAA designation:

- AAAA PSXX for process sewer samples.
 - SUXX for sump samples.
 - SXXX for soil samples.
 - HXXX for water samples.
 - LXXX for oil samples.
 - WXXX for swipe samples.
 - CXXX for concrete samples.

- WDXX for wood samples.
- SSXX for sanitary sewer samples.
- UKXX for all other samples.

"BBBBB" represents a six-digit number indicating the sample date (month/day/year).

"DDD" represents a letter denoting quality control status and is not required for all samples.

FD - field duplicate.

MS - matrix spike.

MSD - matrix spike duplicate.

TB - trip blank.

EB - equipment blank.

WB - distilled water blank.

The characteristics of the sample determine the packaging of samples for shipment to off-site laboratories. The Site Consolidated Transportation Activity Manual (SCTA) (MKF and JEG 1992c) provides guidance on the proper classification, packaging, labeling, and marking of samples.

The Site Shipping Officer (SSO) administers the shipping program and provides guidance and training to all Project Management Contractor (PMC) or Subcontractor personnel. The SSO also inspects sample shipments for compliance with site procedures and government regulations as well as upon return from off-site laboratories.

Samples returned from off-site laboratories are archived or disposed of in accordance with RC-22a.

6 WASTE CLASSIFICATION

The U.S. Environmental Protection Agency (EPA) identifies a hazardous waste as meeting the criteria shown in two sections in Part 261, 40 CFR. Subpart C outlines the waste's characteristic ignitability, corrosivity, reactivity, and toxic characteristic leaching procedure (TCLP) toxicity, and Subpart D specifically lists hazardous wastes by source or name.

Once a waste material is identified and characterized and found to be a hazardous waste, that hazardous waste must be correlated with a Department of Transportation (DOT) hazard class. A hazardous waste meeting the definition of more than one DOT hazard class shall be classed according to the DOT Hazard Precedence List (49 CFR 173.2).

Any waste material found to have a polychlorinated biphenyl (PCB) concentration greater than 50 ppm is regulated by the Toxic Substances Control Act (TSCA) and must be marked as required by 40 CFR 761 Subpart M.

Any waste material found to have uranium concentrations greater than background by a standard Geiger-Müller (G-M) counter shall be considered radioactive and labeled as such until detailed radiological characterization is completed. Once detailed radiological data are available, the material will be classified as defined by the Department of Transportation in 49 CFR Section 172.

7 RECORDKEEPING

In keeping with the intentions of the reporting and recordkeeping requirements of 40 CFR Part 262, complete documentation of all field and laboratory analyses will be required.

All field sampling and analytical data will be documented on a Weldon Spring Site Remedial Action Project (WSSRAP) Field Analysis Data Sheet.

Off-site laboratory documentation, when requested, will consist of all sample preparation logs, notebook pages, chromatograms (or other instrument printouts) and laboratory custody transfer records.

All field and laboratory data will be maintained on file at the WSSRAP by the Quality Assurance (QA) Department in accordance with QAPP-9 until completion of the project or as directed by the U.S. Department of Energy (DOE) or a minimum of 10 yr after the completion of remedial actions.

All movements of containerized wastes, whether on-site or off-site, will be entered into the Waste Inventory Tracking System (WITS) as detailed in procedure RC-13.

Where appropriate and applicable, photographs will be taken to document field observations. These will be taped to the field data sheet or kept in separate logs intended for this purpose, as necessary.

Data sheets that document methods and equipment used will be maintained during all sampling operations and will be consistent with all WSSRAP standard operating procedures and requirements outlined in SW-846. The required information for these data sheets is detailed in procedure RC-24s.

Data required by the selected treatment or disposal facility for disposal purposes will be made available to the facility in accordance with the facility's requirements. In many instances, this will require completion of a waste profile sheet or similar form required by the chosen facility which details the chemical and physical characteristics of the waste. All such paperwork will be completed by the Waste Management Group and forwarded to the facility with a

representative sample of the waste if required or requested. Copies of any such paperwork will be maintained on file in the Environmental Compliance Departmental files.

8 ACRONYMS

AEC Atomic Energy Commission

ALARA as low as reasonably achievable

ARAR applicable or relevant and appropriate requirements
ATSDR Agency for Toxic Substances and Disease Registry

BADT best available disposal technology

CAA Clean Air Act

CERCLA Comprehensive Environmental Response Compensation and Liability Act

CLP Contract Laboratory Program
CRF Code of Federal Regulations

CWA Clean Water Act
DNT dinitrotoluene

DOE United States Department of Energy

DOT United States Department of Transportation

DQO data quality objective

EMSL-LV Environmental Monitoring and Support Laboratory, Las Vegas

EPA United States Environmental Protection Agency

EPP emergency plans and procedures
ES&H Environmental Safety and Health

FADS Field Analysis Data Sheet

FUSRAP Formerly Utilized Sites Remedial Action Program

GCMS gas chromatography/mass spectroscopy

G-M Geiger Müller

HCP Hazard Communication Plan
HEPA high efficiency particulate air
HSL Hazardous Substance List
LDR Land Disposal Restriction

MDNR Missouri Department of Natural Resources

MSA material staging area

MSDS Material Safety Data Sheet

NIOSH National Institute for Occupational Safety and Health NPDES National Pollutant Discharge Elimination System

NPL National Priorities List

NRC United States Nuclear Regulatory Commission

OSHA Occupational Safety and Health Administration

PAH polynuclear aromatic hydrocarbons

PCB polychlorinated biphenyls

PMC Project Management Contractor
PPE personal protective equipment

PPERM Personal Protective Equipment Requirements Manual

OA/OC quality assurance/quality control

RCRA Resource Conservation and Recovery Act radioactive materials management area

RQ reportable quantity

RSFMP Remote Surplus Facility Management Program
SARA Superfund Amendments and Reauthorization Act

SCTA Site Consolidated Transportation Activity

SDWA Safe Drinking Water Act

SFMP Surplus Facility Management Program

SSO Site Shipping Officer

TCLP toxic characteristic leaching procedure

TNT trinitrotoluene

TOC total organic carbon TOX total organic halide

TPH total petroleum hydrocarbons

TSA temporary storage area

TSCA Toxic Substances Control Act

TSDF treatment, storage, and disposal facility

USATHAMA U.S. Army Toxic and Hazardous Materials Agency

WITS Waste Inventory Tracking System

WME Waste Management Engineer
WMG Waste Management Group
WSCP Weldon Spring Chemical Plant
WSOW Weldon Spring Ordnance Works
WSVP Weldon Spring vicinity property

WSS Weldon Spring site

WSSRAP Weldon Spring Site Remedial Action Project

WSO Weldon Spring Quarry

WSCP Weldon Spring Chemical Plant

9 REFERENCES

9.1 Regulations

- 40 CFR Part 61, Subpart M National Emission Standard for Asbestos
- 40 CFR Part 261 Identification and Listing of Hazardous Wastes
- 40 CFR Part 262 Standards Applicable to Generators of Hazardous Wastes
- 40 CFR Part 264 Standards for Operators and Owners of Hazardous Waste Treatment, Storage, and Disposal Units
- 40 CFR Part 268 Land Disposal Restrictions
- 40 CFR Part 302 Designation, Reportable Quantities, Notifications
- 40 CFR Part 761 PCBs Manufacturing, Processing, Distribution in Commerce, and Use Prohibitions
- 49 CFR Part 17 2 Hazardous Materials Tables and Hazardous Materials Communications
 Regulations
- 49 CFR Part 173 Shippers General Requirements for Shipments and Packaging
- 10 CSR 25-11 Missouri State Regulations for Waste Oil
- 29 CFR Part 1910.1450 Occupational Exposure to Hazardous Chemicals in Laboratories

9.2 Federal Documents

- EPA, see U.S. Environmental Protection Agency
- U.S. Department of Energy Oak Ridge Gaseous Diffusion Plant, 1986. Waste Acceptance Plan and Analytical Protocol for the K-1435 Toxic Substance Control Act Incinerator, Rev. 0. Prepared for the U.S. Department of Energy, Oak Ridge Operations Office. St. Charles, Missouri. October.
- U.S. Department of Energy Order 5400.5 Radiation Protection of the Public and Environment
- U.S. Environmental Protection Agency SW 846, 1986. Test Methods for Evaluating Solid Wastes, Physical/Chemical Methods. 3rd Edition.
- U.S. Environmental Protection Agency 600/4-79-020, 1981. Chemical Methods for the Examination of Waters and Wastes. Cincinnati, Ohio.
- U.S. Environmental Protection Agency, 1982. Resource Conservation and Recovery Act Inspection Manual.

9.3 Project Management Contractor Plans

- MK-Ferguson Company and Jacobs Engineering Group, 1988. Phase I Chemical Soil Investigation Data Report for the Weldon Spring Chemical Plant/Raffinate Pits, DOE/OR-21548-016. Prepared for the U.S. Department of Energy, Oak Ridge Operations Office. St. Charles, MO. January.
- MK-Ferguson Company and Jacobs Engineering Group, 1989. Weldon Spring Site Remedial Action Project Safety and Health Manual.
- MK-Ferguson Company and Jacobs Engineering Group, 1991a. Laboratory Chemical Hygiene Plan, Rev. 0. DOE/OR/21548-184. Prepared for the U.S. Department of Energy, Oak Ridge Operations Office, Weldon Spring Site Remedial Action Project. St. Charles, MO.

- MK-Ferguson and Jacobs Engineering Group, 1991b. Building Characterization Work Plan, Rev. 0. DOE/OR/21548-185. Prepared for the U.S. Department of Energy, Oak Ridge Operations Office. St. Charles, Missouri. April.
- MK-Ferguson Company and Jacobs Engineering Group, 1991c. Environmental Quality Assurance Plan, Rev. 0. DOE/OR/21548-240. Prepared for the U.S. Department of Energy, Oak Ridge Operations Office, Weldon Spring Site Remedial Action Project. St. Charles, MO. November.
- MK-Ferguson Company and Jacobs Engineering Group, 1992a. Waste Management Plan, Rev. 3. DOE/OR/21548-166. Prepared for the U.S. Department of Energy, Oak Ridge Operations Office. St. Charles, MO. February.
- MK-Ferguson Company and Jacobs Engineering Group, 1992b. Waste Management Training Plan, Rev. 2. DOE/OR/21548-149. Prepared for the U.S. Department of Energy, Oak Ridge Operations Office. St. Charles, MO. October.
- MK-Ferguson Company and Jacobs Engineering Group, 1992c. Site Consolidated Transportation Activity Manual, Rev. 0. DOE/OR/21548-309. Prepared for the U.S. Department of Energy, Oak Ridge Operations Office. St. Charles, MO. In press.

9.4 **Environmental Compliance Department Procedures**

- Field Laboratory Operating Procedure RC-4s RC-9s Containerized Waste Labeling Procedure Waste Inventory Tracking Procedure RC-13
- Management of Site Generated Waste RC-18a
- RC-20a Container Management Procedure
- Field Analytical Procedure RC-21s
- Management of Samples Returned from Off-site Laboratories RC-22a
- RC-24s Waste Sampling Procedure
- Field Compatibility Procedure RC-26s
- RC-31a Environmental Monitoring Data Validation
- Supplemental Analysis Procedure RC-37s
- **RC-38s** Environmental Compliance Logbook Procedure

RC-32s Determining the Radioactive Component of a Hazardous Waste

9.5 ES&H Department Procedures

ES&H 2.3.8	Contamination Survey
ES&H 2.6.3	GM Detector Calibration, Operation and Usage
ES&H 2.6.9s	Instructions for Calibration and Operation of the Computer Interface Multi-
	Channel Analyzer with a High Purity Germanium Detector
ES&H 4.1.2	Chain of Custody
ES&H 4.3.1s	Surface Water Sampling
ES&H 4.4.5	Soil and Sediment Sampling
ES&H 4.9.1	Environmental Monitoring Data Verification
ES&H 6.2.1	Hazardous Waste Site Health and Safety Training

9.6 Quality Assurance Procedures

QAPP-1	Quality Assurance Program Plan Organization
QAPP-9	Quality Assurance Records
OADD-10	Andits

QAPP-10 Audits

SQA-2a Quality Assurance Surveillance

9.7 Other

Acurex Corporation, 1984. Design and Development of a Hazardous Waste Reactivity Testing Protocol. Mountain View, CA.

Appendix A
TABLE 1 K-1435 Waste Acceptance Plan

TABLE 1 Waste Analysis Parameters and Methods

Parameter	Method	References							
Chlorine	1. A004	"Sampling and Analysis Methods for Hazardous Waste Incineration," EPA-600/8-84-002, February 1984							
	2. EPA-9020	 "Test Methods for Evaluating Solid Waste," Physical and Chemical Methods; USEPA SW-846, 3rd Edition 							
	3. ASTM-2361	3. ASTM D-2361-85; "Test Method for Chlorine in Coal"							
	4. ^a ASTM-3286/ EPA-300.00	 ASTM D-3268-84; "Test Method for Gross Calorimetric Value of Coal and Coke by the Isothermal Bomb Calorimeter"/"The Determination of Inorganic Anions in Water by Ion Chromatography," EPA-600/4-84-017, March 1984 							
Sulfur	1. ASTM-129	 ASTM D-129-64 (1978); "Test Method for Sulfur in Petroleum Products (General Bomb Method)" 							
	2. ^a ASTM-3286/ EPA-300.00	 ASTM D-3286-84; "Test Method for Gross Calorimetric Value of Coal and Coke by the Isothermal Bomb Calorimeter"/"The Determination of Inorganic Anions in Water by Ion Chromatography," EPA-600/4-84-017, March 1984 							
Phosphorus	1. A021 (ICAP)	 "Sampling and Analysis Methods for Hazardous Waste Incineration," EPA-600/8-84-002, February 1984. 							
	2. EPA-6010 (ICAP)	2. "Test Methods for Evaluating Solid Waste," Physical and Chemical Methods; USEPA SW-846, 3rd Edition							
Fluorine	1. ASTM-3761	 ASTM D-3761-84; "Test Method for Total Fluorine on Coal by the Oxygen Bomb Combustion/Ion Selective Electrode Methods" 							
	2. ^a ASTM-3286/ EPA-300.00	 ASTM D-3286-84; "Test Method for Gross Calorimetric Value of Coal and Coke by the Isothermal Bomb Calorimeter"/"The Determination of Inorganic Anions in Water by Ion Chromatography," EPA-600/4-84-017, March 1984 							
Ash (Liquid)	1. A001	 "Sampling and Analysis Methods for Hazardous Waste Incineration," EPA-600/8-84-002, February 1984 							
	2. ASTM-482	2. ASTM D-482-80; "Test Method for Ash from Petroleum Products"							

TABLE 1 Waste Analysis Parameters and Methods (Continued)

Parameter	Method	References							
Ash (Solid)	1. A001	"Sampling and Analysis Methods for Hazardous Waste Incineration," EPA-600/8-84-002, February 1984							
	2. ASTM-3174	 ASTM D-3174-82; "Test Method for Ash in the Analysis Sample of Coal and Coke from Coal" 							
Water	1. ASTM-1533	ASTM D-1533-83; "Test Method for Water in Insulating Liquids (Karl Fischer Method)"							
	2. ASTM-1064	2. ASTM E-1064-85; "Test Method for Water in Organic Liquids by Colorimetric Karl Fischer Titration"							
Cyanide	1. EPA-335.2	 "Methods for Chemical Analysis of Water and Wastewater," EPA- 600/4-79-020, March 1983 							
PCB	1. EPA-8080	 "Test Methods for Evaluating Solid Waste," Physical and Chemical Methods, USEPA SW-846 2nd Edition 							
Uranium (Total)	1. Colorimetric	TP-0803; "Colorimetric Uranium," ORGDP Analytical Chemistry Department Technical Procedures Manual							
	2. Fluorometric	2. TP-0803; "Fluorometric Uranium," ORGDP Analytical Chemistry Department Technical Procedures Manual							
	3. EPA-6010	 "Test Methods for Evaluating Solid Waste," Physical and Chemical Methods,: USEPA SW-846 3rd Edition 							
235U Assay	Thermal lonization Mass Spectrometric Method	TP-2101; "Thermal ionization Mass Spectrometric Method," ORGDP Analytical Chemical Department Technical Procedures Manual							
Specific Gravity	1. ASTM-3142	 ASTM D-3142-84; "Test Method for Specific Gravity or API Gravity of Liquid Asphalts by Hydrometer Method" 							
Viscosity	1. ASTM-445	 ASTM D-445-83; "Test Method for Kinematic Viscosity of Transparent and Opaque Liquids (and the Calculation of Dynamic Viscosity)" 							
Flash Point (F)	1. EPA-1010	 "Test Methods for Evaluating Solid Waste," USEPA SW-846, 2nd Edition 							

TABLE 1 Waste Analysis Parameters and Methods (Continued)

Parameter	Method	References							
Suspended Solids	1. EPA-160.1	 "Methods for Chemical Analysis of Water and Wastewater," EPA- 600/4-79-020, March 1983 							
Size (wt% > 20 mesh)	1. ASTM-310	 ASTM D-310-69(1989); "Test Method for Size of Anthracite" 							
pH (Liquid)	1. EPA-9040	 "Test Methods for Evaluating Solid Waste," Physical and Chemical Methods, USEPA SW-846, 3rd Edition 							
pH (Organics)	2. EPA-9041	 "Test Methods for Evaluating Solid Waste," Physical and Chemical Methods, USEPA SW-846, 3rd Edition 							
pH (Solids)	3. EPA-9045	 "Test Methods for Evaluating Solid Waste," Physical and Chemical Methods, USEPA SW-846, 3rd Edition 							
Heating Value Btu/lb	1. ASTM-3286	 ASTM D-3286-84; "Test Method for Gross Calorific Value of Coal and Coke by the Isothermal Bomb Calorimeter" 							
	2. ASTM-240	 ASTM D-240-76(1980); "Test Method for Heat of Combustion of Liquid Hydrocarbon Fuels by Bomb Calorimeter" 							
Number of Phases	1. Visually								
Metals by ICP (A1, Be, Cd, Fe, Li, Na, Pb, Cu, Mg, Mn, Ni, Ti, Zn)	1. EPA-6010	 "Test Methods for Evaluating Solid Waste," Physical and Chemical Methods, USEPA SW-846, 3rd Edition 							
Metals by AA (As, Se)	1. EPA-7060	 "Test Methods for Evaluating Solid Waste," Physical and Chemical Methods, USEPA SW-846, 3rd Edition 							
Mercury by Cold Vapor AA	1. EPA-7470/7471	 "Test Methods for Evaluating Solid Waste," Physical and Chemical Physical and Chemical Methods, USEPA SW-846, 3rd Edition 							

TABLE 1 Waste Analysis Parameters and Methods (Continued)

Parameter	Method	References							
Technetium-99	1. EC-186	EC-186; "Technetium-99 in Water (Radiochemical Method)," Martin Marietta Energy Systems Env. and Effluent Analysis Manual							
	2. EC-260	 EC-260; "Technetium-99 in Air Filters (Radiochemical Method), "Martin Marietta Energy System Env. and Effluent Analysis Manual 							
	3. EC-355	 EC-355; "Technetium in Sediment and Soil (Radiochemical Method)" Martin Marietta Energy System Env. and Effluent Analysis Manual 							
Corrosivity Toward Steel	1. EPA-1110	 "Test Methods for Evaluating Solid Waste," Physical and Chemical Methods, USEPA SW-846, 2nd Edition 							
Neptunium	1. TP-1635	 TP-1635; "Neptunium, Alpha Activity," ORGDP Analytical Chemical Department Technical Procedures Manual 							
Plutonium	1. TP-1635	TP-1635; "Plutonium, Alpha Activity," ORGDP Analytical Chemical Department Technical Procedures Manual							
Thorium	1. TP-1635	 TP-1635; "Thorium, Alpha Activity," ORGDP Analytical Chemical Department Technical Procedures Manual 							
Cesium	1. EC-134	 EC-134; "Gamma-Ray Emitting Nuclides (Nondestructive Spectrometric Method)" Martin Marietta Energy Systems Env. and Effluent Analysis Manual 							
Tritium	1. EC-189	EC-189; "Tritium in Water" (Radiochemical Method) Martin Marietta Energy Systems Env. and Effluent Analysis Manual							
Alpha/Beta	1. EPA-900	 EPA-600/4-80-32; August 1980 "Prescribed Procedure for Measurement of Radioactivity in Drinking Water" 							
Gamma Activity	1. EC-134	 EC-134; "Gamma-Ray Emitting Nuclides (Nondestructive Spectrometric Method)" Martin Marietta Energy Systems Env. and Effluent Analysis Manual 							

a ASTM-2361, ASTM-3286, ASTM-3761, and ASTM-129 all use the bomb calorimeter for sample preparation. Therefore, the solution obtained in ASTM-3286 to determine the heating value can also be used with EPA Method 300.00, "The Determination of Inorganic Anions in Water by Ion Chromatography," to determine chlorine, sulfur, and fluorine.

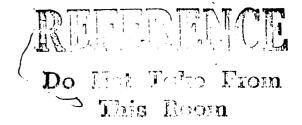
IRA-500-502

IRA-500-503



Department of Energy

Oak Ridge Operations
Weldon Spring Site
Remedial Action Project Office
Route 2, Highway 94 South
St. Charles, Missouri 63303



September 03, 1987

Ms. Katherine Biggs
United States Environmental
Protection Agency
Region VII
726 Minnesota Avenue
Kansas City, Kansas 66101

Dear Ms. Biggs:

Enclosed is the information regarding disposal of containerized chemicals from the Weldon Spring Site, which we agreed upon in our telephone conference on July 24, 1987.

The site contains 4,000 containers holding about 5,000 gallons of liquids and 2,500 cubic feet of solids of various chemicals, some of which are classified as hazardous. Many of the containers are deteriorating. This creates both a safety hazard for site personnel and a threat to the environment. Therefore, we plan to dispose of these materials.

The enclosed index lists three (3) attachments, including an inventory of containerized chemicals, locations of containers and specifications for the work. If you have any questions, please contact Jim Coyne of PEER Consultants, our support services contractor, at (314) 441-8472.

Sincerely,

61.67. rulson

R. R. Nelson Project Manager Weldon Spring Site Remedial Action Project

Enclosure: As stated

cc: Dave Bedan, MDNR

FILE NUMBER:

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INTERIM MEASURE

CONTAINERIZED WASTE HANDLING, TRANSPORT, AND DISPOSAL SUMMARY

The task consists of handling and stabilization, transporting to an EPA approved disposal facility and disposal of hazardous and non-hazardous wastes. Approximately 5,000 gallons of liquids and 2,500 cubic feet of solids are contained in 4,000 containers inside and outside of buildings on the Weldon Spring Chemical Plant site.

For those wastes which exhibit radiation levels that are statistically significant above background, those wastes will be segregated and stored in an onsite interim storage area.

The waste containers are deteriorating. The containers are scattered throughout the Weldon Spring Chemical Plant site (see Attachment No. 2 - Technical Specification with Attachments 1, 2, and 3). This situation creates a hazard to the health and safety of onsite workers and a threat to the environment.

We propose to dispose of all non-radiologically contaminated wastes at one or more EPA approved hazardous waste disposal facilities in compliance with 40 CFR 264-265 and 268 standards and regulations. Off-site transportation of this waste will be in compliance with 49 CFR 172-179, 49 CFR 387 (46 FR 30974, 47033), DOT-E 8876, and 40 CFR 262 and 263 regulations and standards.

We propose to consolidate and temporarily store all radiologically contaminated waste in Building 406 as an interim remedial action measure.

LIST OF ATTACHMENTS

Attachment 1: Interim Measures - Containerized Waste

Handling, Transport, and Disposal - Summary

Attachment 2: Technical Specifications - 3589-SC-WP056

Containerized Waste Handling, Transport, and

Disposal with Site Maps and Attachment 1 (Chemical Plant Containerized Chemical

Inventory), Attachment 2 (Chemical Plant Fire

Extinguisher Inventory), Attachment 3

(Locations of Wastes)

Attachment 3: Special Conditions - 3589-SC-WP056

Do Mis Room

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4.3 COURT ROAD

FIG. COX 529

ST. PELERS, MO 63376



3589-SC-WP056

SPECIFICATIONS FOR

CONTAINERIZED WASTE HANDLING, TRANSPORT, AND DISPOSAL

1.0 LOCATION

This project is located at the Weldon Spring Chemical Plant (WSCP) which is situated approximately two miles southwest of Weldon Spring on Highway 94 South in St. Charles County, Missouri (see Figure 1.1). The major structures at the WSCP are shown on Figure 1.2.

2.0 SCOPE

Selected containerized chemicals and materials (hereafter referred to as containerized wastes) which include liquids, solids, gases, sediments and sludges contained in surface and subsurface storage tanks, process vessels, drums, fuel tanks, laboratory hoods and containers, gas cylinders, fire extinguishers, batteries, bags, boxes, bottles, cans, jars, and other miscellaneous containers have been identified for removal from the WSCP by the Contractor for the Weldon Spring Site Remedial Action Program (WSSRAP).

The Contractor estimates that the following volume percentages are representative of the waste categories present at the WSCP which will require handling by the Subcontractor:

Waste Category	Volume Percentage
Hazardous wastes	55
Non-hazardous wastes	45
*Radiologically contaminated wastes	5

Subcontractor will be responsible for the handling of these materials onsite, and their transfer/consolidation to the WSSRAP interim storage facility located in Building 406. The Subcontractor will not be responsible for any offsite handling, transport, or disposal of the WSCP radiologically contaminated containerized wastes.

^{*}Approximately 5 percent of the WSCP containerized wastes have been identified as being radiologically contaminated.

During a contractor performed containerized waste inventory approximately 310 individual container groups were identified. In these 310 groups, there are approximately 4,000 individual containers. The Contractor estimates that there are approximately 5,000 gallons of liquids and approximately 2,500 cubic feet of solids included in the 4,000 containers. Many of the containers were found to be empty or have partially full contents.

Based on the contractor inventory the following waste volume estimates are anticipated for Subcontractor handling, transport, and disposal:

Category	Volume
Organic liquids, solvents, lubricants	4,500 gal.
Tar and asphalt liquids	250 gal.
Tar and asphalt solids	50 cu. ft.
Aqueous acids (pH <2.0)	100 gal.
Aqueous alkalines (ph >12.0)	150 gal.
Metals (magnesium scrap, metal powders, etc.)	600 cu. ft.
PCB contaminated liquids (50-500 ppm)	120 gal.
PCB materials (>500 ppm)	125 gal.
Laboratory pack liquids	200 gal.
Laboratory pack solids	70 cu. ft.
Strong oxidizers	300 cu. ft.
Strong reducing agents	225 cu. ft.
Fire extinguishers	525 units
Automotive or industrial batteries	
Unknown solids	258 units
	1,000 cu. ft.

Attachments 1 and 2 provide a general description as to the location, number, and type(s) of containers to be handled and the containerized waste to be removed from the WSCP. Attachment 3 provides figures which detail the locations of the selected containerized wastes at the WSCP.

During the containerized waste inventory, the contractor collected approximately 50 samples of selected materials which were analyzed for ignitability, corrosivity, and PCB content. The results of these analyses for all samples collected are reported in the "Comments" column of Attachment No. 1.

The Subcontractor shall be accompanied to each location by the Contractor Representative(s) who will also provide radiological monitoring during the Subcontractors performance of Scope of Work Subsection 2.1 activities listed below.

The following activities will be performed on the selected containerized wastes to satisfy this Scope of Work to be performed under this subcontract.

2.1 Handling and Stabilization

The Subcontractor shall supply all equipment, materials, labor, supervision and services necessary to handle the selected containerized wastes at the WSCP. Handling shall include the preparation, relocation, staging, opening, sampling, laboratory analysis for compatibility and disposal, stabilization, bulking, repacking and/or recontainerization, and compaction of empty containers.

For those designated containerized wastes which have previously been identified by the Contractor, or those which are identified during the Subcontractors performance of the Scope of Work in this Subsection, which exhibit radiation levels that are statistically significant above background, the Subcontractor will be required to segregate and relocate those containerized wastes to contractor designated onsite interim storage area located in Building 406.

For those storage vessels, tanks, drums, and other containers which have not previously been evaluated by the Contractor, but are included in the Attachment 1 inventory, the Subcontractor shall be required to open and/or inspect the inside of the container to evaluate the presence of waste materials which will require Subcontractor handling.

For those fire extinguishers identified in Attachment 2, the Subcontractor shall be responsible for their collection, pressure release and discharge of contents, and the handling of the released contents and empty casings.

2.2 Transport

The Subcontractor shall supply all equipment, materials, labor, supervision, and services necessary to transport those containerized wastes identified in Scope of Work, Subsection 2.1 off-site to an EPA approved disposal facility. Transport shall include the packaging, labeling, marking, manifesting, placarding, and off-site transport of those designated containerized wastes in compliance with 49 CFR 172-179, 49 CFR 387 (46 FR 30974, 47033), DOT-E 8876, and 40 CFR 262 and 263 regulations and standards.

2.3 Disposal

The Subcontractor shall be responsible for and shall provide for all equipment, materials, labor, supervision, and services necessary for the proper disposal of those containerized wastes identified in Scope of Work, Subsections 2.1 and 2.2. Proper disposal shall include the disposal of these containerized wastes at one or more EPA approved hazardous waste disposal facilities in compliance with 40 CFR 264-265 and 268 standards and regulations.

3.0 REQUIREMENTS

3.1 Permits and Licenses

The Subcontractor will obtain, provide, and be responsible for all permits and licenses which are required to perform the Scope of Work, Subsections 2.1, 2.2 and 2.3, and shall comply with and be responsible for all federal, state, and local regulations, including but not limited to the following: The Hazardous Waste Regulations defined by 40 CFR 261 - the Resource Conservation and Recovery Act (RCRA); the Hazardous Substance Regulations defined by 40 CFR 300.6 under the Comprehensive Environmental Response, Compensation, and Liabilities Act (CERCLA); and the Interim Final Rule for Hazardous Waste Operations defined by 29 CFR Part 1910 Subpart H under the Occupational Safety and Health Act (OSHA) - General Industry Standards.

Documented proof of compliance shall be provided to the Contractor Representative(s) prior to the onset of site activities.

3.2 Work Plan

The Subcontractor shall develop a detailed work plan which will be submitted to the Contractor Representative(s) for review and approval prior to the start of on-site activities identified in Scope of Work, Subsection 2.1. This work plan shall include, but not be limited to, the following topics:

o Site preparation requirements

o Container relocation, staging procedures

o Container opening procedures and methodology

O Container sampling procedures, methodology and strategy

o Sample analysis procedures and methodology

O Compatibility testing procedures and methodology

o Bulking, overpacking, and/or recontainerization procedures

o Quality Assurance plans

o Contingency plans

o Health and Safety plans

The Contractor Representative(s) will review the Subcontractor work plan to ensure compliance with applicable WSSRAP programs. The Subcontractor shall insure that all sampling, analytical, compatibility/bulking, and packing procedures comply with federal, state, and local requirements for off-site transport (as described in Scope of Work, Subsection 2.2) and disposal (as described in Scope of Work, Subsection 2.3). Figure 3-1 provides an example of the Contractors - Containerized Chemical Contents Identification form which may be used by the Subcontractor to document sample contents for compatibility and disposal.

A copy of all containerized waste laboratory analyses will be provided to the Contractor for review and inspection, as the data becomes available.

3.3 Site Preparation

The Subcontractor shall be responsible for all initial site preparation and set up areas required for the performance of the Scope of Work, Subsections 2.1 and 2.2. The number and location of support facilities, special operating areas, or temporary storage areas needed may include but may

not be limited to the following:

- o Staging areas
- o Container opening area(s)
- Waste consolidation, loading areas
- o Interim storage areas
- o Equipment and personnel decontamination areas
- o Drum crushing area
- o Mobile laboratory, and
- o Administration/office area(s)

3.4 Containerized Waste Access

The Subcontractor shall be responsible for the removal of various obstacles, including materials, objects, and equipment, which impede access to the handling of containerized wastes required for the performance of the Scope of Work, Subsection 2.1. A summary of the buildings and/or areas where access difficulty may be encountered, are listed below.

- o Building 101 (Sampling Plant) This is a multiple story building which is empty. Containerized wastes such as fire extinguishers, are located on each of the floors. Access to each floor is by inside stairs.
- O Building 103 (Digestion/Denitration Plan) This is a multiple story building. Much of
 the building is empty. Containerized wastes
 i.e. fire extinguishers, are located on each
 of the floors. Access to each floor is by
 inside stairs.
- o Building 105 (Extraction Plant) This is a multiple floor building. The building is empty. One fire extinguisher is located on the 2nd floor on the south side. Access to this floor is from outside stairs.
- o Building 108 (Nitric Acid Recovery) No containerized wastes are located inside the building. Fire extinguishers are located outside the building. Brick flooring around the outside of the building is broken and caving in some areas.

- o Building 201 (Green Salt Plant) This is a multiple story building which still contains its original equipment contents. Containerized wastes were found on all floors; however, the majority of the containers on upper floors were fire extinguishers. Access to each floor is by inside stairs.
- O Building 202 (Hydrofluoric Acid/Ammonia Tanks) This area consists of three outside and five inside above ground storage tanks. Tank access is by outside stairs to the top of tanks/building roof. Each tank has an access port located on the top, east end of the tank. The Contractor has opened three of these tanks, each of which were found to be empty.
- 0 Building 301 (Metal Plant) - This multiple story building still contains it original equipment. Contaminated materials equipment such as lathes, milling machines, and conveyers, etc. were placed in rows. Smaller pieces of equipment, tools, and small motors, etc., were placed in 55 gallon drums stacked two-drum high. contaminated equipment was stacked on top of drums to help support the cocoon structures. The drums and other equipment and machinery have been covered with a thin fiber mesh and then sprayed with a one (1) inch thick layer of hard setting polyurethane foam. Thirty-four (34) of the cocoons exist. Thirteen (13) of the cocoons have been partially opened by the Contractor twenty-eight (28) containers were located. The Subcontractor shall be responsible for the complete removal and disposal of cocoon structures. A table showing approximate size of each cocoon is shown in Figure A3-8A. The 28 containers are also included in the subcontract. Inventory, handling, identifying, transporting, disposal of an additional containers inside these cocoons following the specification stated in this Scope of Work will addressed by a Change Order.

- o Building 302 (Magnesium Building) This is a single story building which still contains its original equipment contents. Access to some containers may be hindered by equipment.
- O Building 401 (Boiler House) - This is a multiple story building which still contains its original equipment contents. Containerized wastes were found on floors; however the majority of containers on the upper floors were fire extinguishers. Access to each floor is by inside stairs.
- O Building 403 (Wet/Dry Chemical Pilot Plant) This is a multiple story building which still
 contains its original equipment contents.
 Containerized wastes were found on all
 floors. Access to each floor is by inside
 stairs.
- Building 407 (Laboratory) This is a single story building; however, some containerized wastes were located on the roof in service The Subcontractor shall responsible for the removal of all designated containerized wastes from inside the building from the roof area(s). Also, Subcontractor shall responsible be removal of designated containerized wastes from eight sealed hoods inside the laboratory which contain fire extinguishers, cans, and bottles. No access problems are anticipated inside the building. Access to the roof is by outside stairs on the south side of the building.
- o Building 408 (Maintenance and Stores) This is a single story building which is generally open floor space. Most of the original shelving and cabinets remain in the building. In some rooms, access may be hindered by these structures.
- O Building 409 (Administration Building) This is a multiple story building which has had much of its original furniture contents removed. Access to each floor is by inside stairs.

- o Building 410 (Services Building) This is a single story building which is generally open floor space. Most of the original cabinets and shelving remain in the clinic and laboratory areas. Access in these rooms may be hindered by these structures.
- O Building 417 (Paint and Oil Storage) This is a single story building which contains a separate shop area, a paint booth, and benches and cabinets used for paint and materials storage. Access to some areas within a room may be hindered by these structures. Outside the building, the open land to the east and south contain abandoned vehicles, machinery, and equipment.
- O Building 428 (Propane and Butane Tanks) This is a single story building which still
 contains pumps and equipment used to transfer
 propane. Two large propane/butane tanks are
 located immediately adjacent to this
 building. Access to these tanks are through
 a fence and up outside stairs.
- o Building 433 (Riggers' Storage) This is a single story building which is divided into several rooms. Many of the rooms are filled with equipment, machinery, and vehicles. Access may be hindered by this equipment and machinery.
- O Building 435 (Storage Building) This is a single story building which is divided into several rooms. Some of the rooms are filled with office supplies, equipment, and machinery which may hinder containerized waste access inside the building. Outside the building to the southwest, there is equipment and machinery which may restrict access to some of the rooms. The Subcontractor may need to relocate some of this material to facilitate containerized waste removal.
- Building 436 (Storage Building) This is a single story building which is divided into several rooms. All of the rooms are filled

with cabinets, shelves, equipment and machinery which may hinder direct access to containerized wastes. Outside the building to the southeast and southwest there is equipment and machinery which may restrict access to some of the rooms, and to containerized waste located proximate to this building. The Subcontractor may need to relocate some of this material/equipment to facilitate containerized waste removal, both inside and outside the building.

o Building 438 (Storage Building) - This is a single story building which is 2/3 full of equipment, machinery, and supplies. The Subcontractor may need to relocate some of this material to facilitate containerized waste removal.

3.5 Stabilization of Wastes

Prior to transport, and as necessary to meet the requirements and make them acceptable for offsite transport and disposal, the Subcontractor shall perform onsite waste pretreatment, including but not limited to:

- o Acid-Base Neutralization
- o Metals Precipitation
- Oxidation of Cyanide and Sulfide
- o Flash Point Reduction, and
- o Solidification

3.6 Containerized Waste Handling Report

After the Subcontractor completes the activities for the Scope of Work, Subsection 2.1, the Subcontractor shall provide to the Contractor Representative(s) with a written report as to the status of the containerized waste handling activities. This report shall include, but will not necessarily be limited to the following:

- o Quantity and Disposition of Wastes Handled
- O Results of Laboratory Analyses and Compatibility Testing
- o Volumes of Compatible Waste Classes
- o Proposed Transportation Options and Requirements
- o Proposed Disposal Options and Requirements

The Subcontractor shall provide these reports within five working days after completion of the Scope of Work, Subsection 2.1 activities.

3.7 Offsite Removal and Transport

After the Subcontractor has completed the activities identified in the Scope of Work, Subsection 2.1, the Subcontractor shall perform Subsection 2.2 activities.

The Subcontractor shall insure that all vehicles used for offsite transport of hazardous wastes are Department of Transportation approved and meet the specifications and requirements for the waste type being hauled.

The Subcontractor shall submit to the Contractor Representative(s) the EPA transporter number for each waste hauler, and the waste manifest records for each load, prior to the waste leaving the WSCP.

3.8 Notification of Disposal

The Subcontractor shall insure that all WSCP hazardous wastes are treated and/or disposed of using the most technically appropriate and regulatory acceptable disposal method(s). organic liquids will disposed be of by incineration. The Subcontractor shall responsible for and shall provide the Contractor disposal documentation for each disposal facility utilized for WSCP containerized wastes.

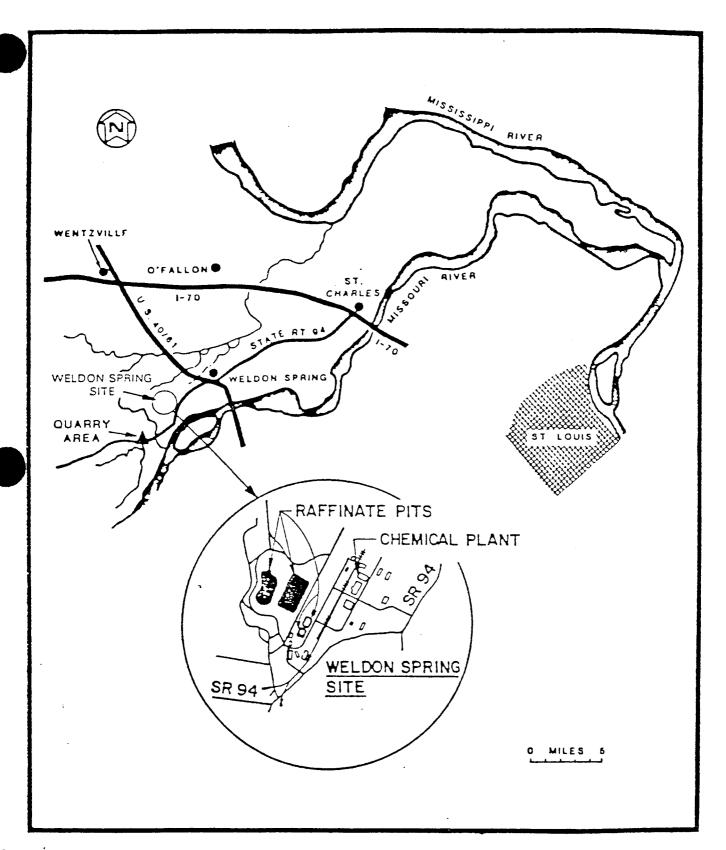


FIGURE 1-1 LOCATION OF WELDON SPRING CHEMICAL PLANT

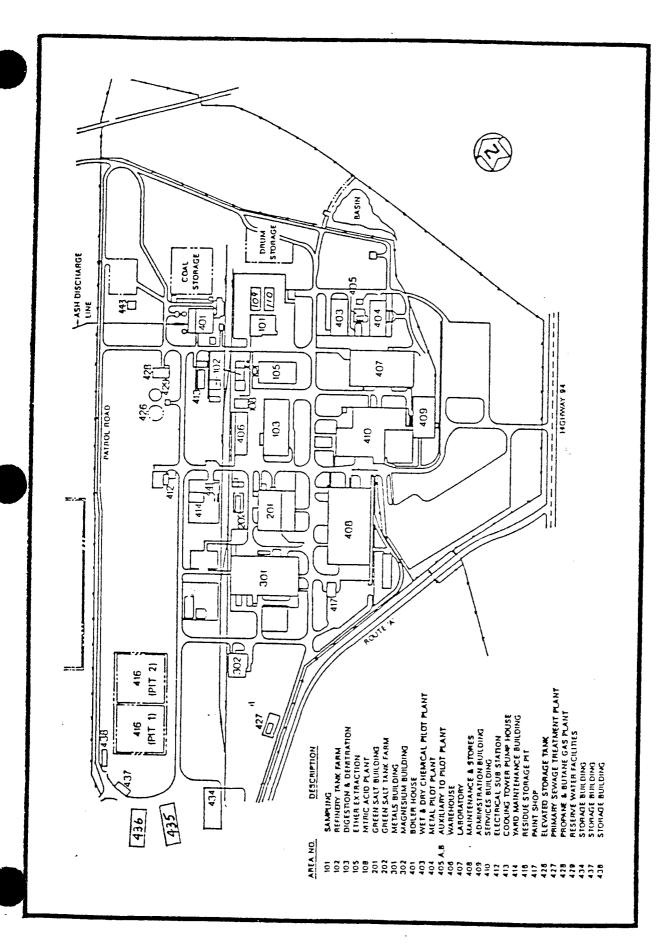


FIGURE 1-2 MAJOR STRUCTURES AT WSCP SITE

FIGURE 3-1

Weldon Spring Site Remedial Action Project (WSSRAP)
Route 2, Highway 94 South, St. Charles, Missouri 63303
Phone (314) 441-8086 Telecopy (314) 447-0803

CONTAINERIZED CHEMICAL CONTENTS IDENTIFICATION

CC-2	-0587	
Container Number CC-2		
Radioactivity Level		
Alpha	Uranium-Natural	
Beta	Uranium-238	
Gamma	Thorium-230	
Gamila	Radium-226	
	Others	
,		
Radiologically Releasable	Off-Site: Yes or	
COMPATABILITY DATA		
pH	Yes or No	
Peroxide or Oxidizer	Yes or No	
Reducing Agent	Yes or No	
Water-Reactive	Low or High	
Halogen Content	Strong, Weak, or	No
Acid	Strong, weak, or	No
Base	Strong, Weak, or Yes or No	110
Cyanides	Yes or No	
Sulfides	Yes or No	
Flammability		
Specific Gravity		BTU/lb
Heat Content		percent
Solids Content		percent
Hydrocarbon	Yes or No	
Pesticides	Yes or No	. =
Sulfur Content	Low or High	
Phenols	Yes or No	
Oil and Grease		percent
Water		percent
Viscosity		percent '
Organochlorine Content		bercame
EP Toxicity Metals	Yes or No	70a \a 50
Solubility	H ₂ O H ₂ SO ₄ or (JE3) 2 50
HAZARDOUS MATERIAL:	Ves or No	
	Yes or No	J
Comments:		
•		
Lab Reports Transcribed By:	(signature)	
	Date:	
Form CC-3	_ , , ,, ,,	Contractor
MK-Ferguson Company	Project Management	CONTOLACTOR

ATTACHMENT 1

Appendix I Key

Group Key	Room Key	
Trash Fuel or Solvent Empty Gas Battery Asphalt or Tar Unknown Paint Oil or Lubricant Magnesium Metal Alkaline Material Acidic Material Medical Supplies	SW W/O N/O E/O S/O WST W/R E/R EAS	Southwest West Outside North Outside East Outside South Outside West West Roof East Roof East
Water Soil or Dirt		·
	Fuel or Solvent Empty Gas Battery Asphalt or Tar Unknown Paint Oil or Lubricant Magnesium Metal Alkaline Material Acidic Material Medical Supplies Chemical/Laboratory reagents Water	Trash SW Fuel or Solvent W/O Empty N/O Gas E/O Battery S/O Asphalt or Tar WST Unknown W/R Paint E/R Oil or Lubricant EAS Magnesium Metal Alkaline Material Acidic Material Medical Supplies Chemical/Laboratory reagents Water Soil or Dirt

APPENDIX I
Weldon Spring Site Remedial Action Project
Chemical Plant Containerized Chemical Inventory
May 1987

COMMENTS	9 1-GALLON UNLABELED, UNRNOWN TOP OF ELEVATOR, LOOK LIKE SOLVENT OR GREASE CONTAINERS. SOME APPEAR TO BE EMPTY.	EMPTY EXCEPT FOR SOME MISC TRASH.	55-GALLON DRUM OF "NO. 130 CLEAR WAX BASE", RUSIED TOP.	REMAINING VOLUME IS UNKNOWN. EMPTY FLASH MIXER. 4'H X 3'D.	EMPTY 8-OUNCE GLASS CONTAINERS.	"NITROGEN", 4'H CALIBRATION GASES 0.3X HEXANE IN NITROGEN.	UNLABELED. 3'H. CONTENTS UNKNOWN.	"OXYGEN", 2'H.	"CARBON TETRACHLORIDE". 4'H.	2 1 1/2-OUNCE "NATURAL GAS",	BATTERIES IN ELECTRIC CARTS, 6 EACH.	3 EMPTY 1-GALLON CANS OF ROOF TAR.	1-GALLON OF SOLVENT.	ROOFING TAR.
PID READINGS	c	0	o	0	0	6	0	0		0	0	0	0	0
INTEGRITY	Good	1 1 3 4 1 1 1 1	्र ख भ		Good	त इ. म	Fair	Falr	Fair	Fair	Fair	r T	Falr	त
COLOR								•		1 1 1 1 1 1				
PHASE	Liquid	Solid	Liquid			S as	Gas	Gas	Gas	Gas	Solid		Liquid	Liquid
FUNCTIONAL GROUP	Solvent	Trash	Solvent	Етр с у	Empty	9 8	Gas	Gas	Gas	Gas	Battery	Empty	Solvent	Tar
VOLUME EST. (ft3 OR GAL.)	5.00	0.00	55.00	00.0	0.00	0.00	0.00	00.00	00.00	0.00	0.00	00.00	1.00	5.00
NUMBER VOLUME (ft3 OR	6	1	₩		25	12		'n	1	2	12	ဗ	T.	1
CONTAINER IYPE	1 Gallon Gan	Drum: Open Top	Drum: Bung Top	Tank	Bottle	Gas Cylinder	Gas Cylinder	Gas Cylinder	Gas Cylinder	Gas Cylinder	Other	1 Gallon Can	1 Gallon Can	5 Gallon Can
ROOM	₹	Sta	ga.	<	٧	<	<	⋖	<	٧	# 1 1 1 2 1	103	103	103
BUILDING #	101	101	103	106	106	109	109	109	109	109	110	201	201	201
CONTAINER I.D. #	CC-2216-0587	CC-2217-0587	CC-2222-0587	CC-2219-0587	CC-2219-0587	CC-2184-0587	CC-2184-0587	CC-2184-0587	CC-2184-0587	CC-2184-0587	CC-2185-0587	CC-2286-0587	CC-2286-0587	CC-2286-0587

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COMMENTS	EMPIY 55-GALLON.	AMBER GALLON JAR FILLED WITH CLEAR LIQUID.	1-GALLON JAR CONTAINING PINK LIQUID.	40-GALLON DRUM, FULL.	LABELED "NITROGEN". ACCESS PORT OPEN TANK IS EMPTY. 6'D X 15'H.	16,000 GALLON, 40'H X 9'D. 3 OUTSIDE TANKS CHECKED AND EMPTY. 5 INSIDE TANKS LABELED HF AND BELIEVED TO BE EMPTY.	19 EMPTY 55-GALLON DRUM OF "A £ B ISONATE SYSTEM", 3 DRUMS CONTAIN RAINWATER FROM LEAKING ROOF.	30-GALLON TRASH CAN.	EMPTY, "PROTECTIVE COATING".	ENTY, "HEXANE".	10 GAL. PLASTIC BOITLE POSSIBLY EMPIY.	IN COCOON #4.	IN COCOON #11, FULL OF TRASH. RADIOLOGICALLY CONTAMINATED.	
PID READINGS	0	0	6	0	0		0	0	0	0	0	0	0	O
Integrity	Falr	Poog	goog	Falr	स म	poog	poog	Falr	Falr	Fair	Fair	Good	poog	Fair
COLOR		Clear	Pink				1 1 3 1 1 1 4 4 4 4 1							
PHASE		Liquid	Liquid	Liquid			1 1 1 1 1 1 1 1	Solld			Liquid		Solld	
FUNCTIONAL GROUP	Empty	Unknown	Unknown	Unknown	Empty		Water	Unknown	Емрту	Емрсу	Unknown	Unknown	Trash	Емрtу
NUMBER VOLUME EST. (ft3 OR GAL.)	0.00	1.00	0.50	40.00	0.00	00.00	165.00	4.00	0.00	0.00	10.00	7.35	7.35	00.0
NUMBER	m		1		-	ω	22		1	1	1	н	2	н
CONTAINER TYPE	Drum: Bung Top	Jar	Jar	Drum: Open Top	Tank	Tank	Drum: Bung Top	Other	5 Gallon Can	1 Gallon Can	Bottle	Drum: Open Top	Drum: Open Top	5 Gallon Can
ROOM	1ST	1ST	1ST	151	M/0		102	102	<	<	<	∢	<	<
BUILDING #	201	201	201	201	201A	202	301	301	301	301	301	301	301	301
CONTAINER I.D. #	CC-2288-0587	CC-2288-0587	CC-2288-0587	CC-2288-0587	CC-2285-0587	CC-2224-0587	CC-2287-0587	CC-2287-0587	CC-2292-0587	CC-2292-0587	CC-2292-0587	CC-2293-0587	CC-2294-0587	CC-2294-0587

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Copperis	55-GALLON DRUMS IN COCOON #13. SOME RADIOLOGICALLY CONTAMINATED. ASSUME LIQUID CONTENTS.	YELLOW PAINT.	1/2-GALLON OF TEXACO THERMATEX, EP-1.	8-03. OUNCES OF "INSTANT START".	EMPTY 55-GALLON DRUM	30 GAL. "TASIL 104W PATCH".		55-GALLON DRUMS, 1 EMPTY, 2 UNKNOWN. ASSUME LIQUID CONTENTS.	55-GALLON, OVERTURNED.	EMPTY, 3'D X 6'H.		"ROOFING TAR".	8 OZ. CAN, "INSTANT START."	55-GALLON LABELED "MG".	30-GALLON LABELED "MG".	55-CALLON, RUSIED, SOME EMPTY. MAY CONTAIN MAGNESIUM SHAVINGS.
PID READINGS	0	0	0	0	0	0	0	o	0	0	0	0	0	0	0	0
INTEGRITY		म् स	Fair	Fair	Falr	Falr	Fair	Falr	Falr	Fair	Poor	Falr	Falr	Fair	Fair	Fair
COLOR																
PHASE		Liquid	Liquid	Sas		Liquid						Liquid	Gas	So11d	Solld	Solid
FUNCTIONAL GROUP	Unknown	Paint	Lubricant	Gas	Empty	Tar	Empty	Unknown	Empty	Empty	Empty	Tar	Gas	Magneslum	Magnestum	Unknown
VOLUME EST. (ft3 OR GAL.)	1375.00	0.50	0.50	0.01	0.00	30.00	0.00	110.00	00.0	0.00	0.00	5.00	0.01	51.40	4.00	22.00
NUMBER	25	-	1	.	H		m	m	1	T.	7	Ħ	1	,	Ħ	90
CONTAINER TYPE	Drum: Open Top	1 Gallon Can	1 Gallon Can	High Pressure	Drum: Open Top	Drum: Open Top	5 Gallon Gan	Drum: Open Top	Drum: Open Top	Tank	Drum: Open Top	5 Gallon Can	High Pressure	Drum: Open Top	Drum: Open Top	Drum: Open Top
ROOM	⋖	υ	v	ပ	۵	۵	۵	۵	N/0	N/0	N/0	N/0	N/0	æ	æ	æ
BUILDING #	301	301	301	301	301	301	301	301	301	301	301	301	301	302	302	302
CONTAINER I.D. #	CC-2295-0587	CC-2289-0587	CC-2289-0587	CC-2289-0587	CC-2290-0587	CC-2290-0587	CC-2290-0587	CC-2291-0587	CC-2296-0587	CC-2296-0587	CC-2297-0587	CC-2297-0587	CC-2297-0587	CC-2158-0587	CC-2158-0587	CC-2159-0587

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COMMENTS	30-CALLON, RUSTED, SOME EMPTY. MAY CONTAIN MACNESIUM SHAVINGS.	3 STEEL HOPPERS, 15'H X 8'D.	40-GALLON, EMPTY	"ALL SURFACE CLEANER".	EMPTY.	55-GALLON CONTAINING WHITE POWDER.	55-GALLON DRUM FILLED WITH EMPTY PLASTIC CONTAINERS AND TRASH.	CAP LABELED "METAL LUBRICANTS CO.", FULL AND RUSTED. FULL, NEVER OPENED. FLASHPOINT = 35C.	SB INDUSTRIAL TYPE BATTERIES ON EAST WALL, 14 ON NORTH WALL.	EMPTY 3'D X 8'H TANK 10' ABOVE GROUND.	STEEL TANK ABOVE GROUND, 1.5'D X 3'H.	HALF FULL 80-POUND BAG OF UNKNOWN MATERIAL.	2 1.5'D X 7.5'H TANKS IN PARALLEL. PROBABLY EMPTY.	80-GALLON TANK ON CART, PROBABLY EMPTY.
PID READINGS	0	0	0	0	0	0	0	006	0	0	0	0	0	0
INTEGRITY	Falr	Good	Fair	Fair	Falr	Fair	Fair	Poor	poog		Good	Poor	Fair	
COLOR						White		Amber						
PHASE	Solid	Solid	Liquid	Liquid		Solld	Solid	Llquld	Solid		Llquld	Solid	Liquid	Liquid
FUNCTIONAL GROUP	Unknown	Unknown	Unknown	Solvent	Empty	Empty	Trash	Lubricant	Battery	Empty	Unknown	Unknown	Unknown	Unknown
NUMBER VOLUME EST. (ft3 OR GAL.)	64.00	00.00	00.0	1.00	00.00	7.35	7.35	55.00	20.00	0.00	10.00	0.40	0.00	0.00
NUMBER	16	m	ч	#	1	-	Ħ	ī	72	1	Ħ	-	2	-
CONTAINER TYPE	Drum: Open Top	Hopper	Drum: Bung Top	1 Gallon Can	Drum: Open Top	Drum: Open Top	Drum: Open Top	Drum: Bung Top	Other .	Tank	Tank		Tank	Tank
ROOM	æ	æ	υ	U	E/0	E/0	E/0	1ST	151	157	15T	1ST	1ST	151
BUILDING #	302	302	302	302	302	302	302	401	401	401	401	401	401	401
CONTAINER I.D. #	CC-2159-0587	CC-2161-0587	CC-2162-0587	CC-2162-0587	CC-2157-0587	CC-2157-0587	CC-2157-0587	CC-2194-0587	CC-2195-0587	CC-2196-0587	CC-2197-0587	CC-2198-0587	CC-2199-0587	CC-2200-0587

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COMPLENTS	3'D X 7'H TANK, PROBABLY EMPTY.	2 1.5'D X 6'H TANKS, PROBABLY EMPTY.	6 5'D X 6'H TANKS LABELED "HYDROGEN SOFTENER", PROBABLY EMPTY.	8'D X 15'H TANK LABELED "DEGASIFIER", PROBABLY EMPTY,	(2204A £ AA) 2 55-GALLON DRUMS, RUSTED. ONE OPENED, ONE SEALED. DRUM SAMPLE HAD TWO PHASES, AMBER AND BLACK SLUDGE. FLASHPOINT (2204A) = 63C, FLASHPOINT (2204AA) = 32C.	2 5-GALLON CONTAINERS LABELED "B & B GREASE".	1 10-GALLON CONTAINERS LABELED BONDING MORTAR.	OPEN AND SPILLED BAG,	4'H X 3'D TANK OF UNKNOWN, PROBABLY EMPIY.	2'D X 7'H TANK OF UNKNOWN, PROBABLY EMPIY.	2 80-POUND BAGS, OPEN, HALF FULL, APPEARING TO BE SALT.	1'D X 4'H EMPTY TANK.	3'D X 4'H STAINLESS STEEL TANK, EMPTY.
PID READINGS	0	c	o	0	200	0	0	•	0	0	0	0	0
INTEGRITY	Falr	Poog			Poor			Poor	Fair	F F F	Poor		Falr
COLOR					Amber								
PHASE	Liquid	Liquid	Llquld	Liquid	Liquid	Liquid	Solid	Solid	Llquid	Liquid	Solid		
FUNCT IONAL GROUP	Unknown	Unknown	Unknown	Unknown	Unknown	Lubricant		Unknown	Unknosn	Unknown		Empty	Empty
NUMBER VOLUME EST. (fe3 OR GAL.)	0.00	0.00	0.00	0.00	110.00	10.00	1.50	0.50	0.00	00.00	1.00	00.0	00.0
NUMBER	1	8	vo	Ħ	8	8		#	#		Ħ	4	Ħ
CONTAINER IYPE	Tank	Tenk	Tank	Tank	Drum: Bung Top	5 Gallon Can	Jar		Tank	Tenk	·	Tank	Tank
ROOM	151	151	151	1ST	ist	1ST	151	1ST	151	151	1ST	181	1ST
BUILDING #	401	401	401	401	401	401	401	401	401	401	401	401	401
CONTAINER I.D. #	CC-2201-0587	CC-2201-0587	CC-2202-0587	CC-2203-0587	CC-2204-0587	CC-2204-0587	CC-2204-0587	CC-2204-0587	CC-2205-0587	CG-2206-0587	CC-2207-0587	CC-2208-0587	CC-2209-0587

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Chemical Plant Containerized Chemical Inventory
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Comments	2 EMPTY AND RUSTED 55-GALLON DRUMS.	10-POUNDS OF POWDER,	1-PINT OF RED-ORANGE OIL.	2 1-QUART METAL CONTAINERS.		2 1-GALLON BOITLES OF NITRIC ACID.	2 1-PINT JARS OF UNKNOWN LIQUID, 1 EMPTY CONTAINER.	500 GALLON WATER TANK, PROBABLY EMPTY.	1 GALLON OF OIL.	DRUMS LABELED FILTER AID. FINE POWDER. FLASHPOINT > 110C.	(2305A) 3 EMPTY 55-CALLON DRUMS. DRUMS ON WEST WALL LABELED SODIUM SULFITE (6 DRUMS), HEAVY FINE POWDER, 3 UNKNOWNS. FLASHPOINT > 110G.	(2305B) DRUM SAMPLED WAS 1/2 FULL AND COLLAPSED, FLASHPOINT > 110C,	BICARBONATE OF SODA.	14 BAGS OF SILICA.	EMPTY 6'D X 18'H TANK LABELED "KOH CAUSTIC POTASH."
PID READINGS	0	0	0	0	0	0	0		0	0	0	o	0	0	0
INTEGRITY	Poor	Fair				poog			Fair		Fair	Poor	Poor	Poor	 1 1 1 1 1
COLOR		White								White	White	Pink			; ; ; ; ; ;
PHASE		Solld	Liquid	Liquid		Liquid		Liquid	Liquid	Solid	Solid	Solld	Solid	Solid	: : : : : : : : :
FUNCTIONAL	Empty		Lubricant	Unknown	Empty	Acid	Unknown	Water	Lubricant	Unknown	Chemicals	Unknown	Chemicals		Empty
VOLUME EST. (ft3 OR GAL.)	0.00	0.25	0.10	0.50	0.00	2.00	0.25	00.00	1.00	58.80	66.00	2.00	10.00	14.00	0.00
NUMBER	2	-	=	7	п	8	m	1	1	œ	12	H	10	14	-
CONTAINER TYPE	Drum: Bung Top	Вох	Jar	Can	5 Gallon Can	Bottle	Jar	High Pressure	1 Gallon Can	Drum: Open Iop	Drum: Open Top	Fiberboard			Tank
ROOM	0/8	<	<	<	. <	ပ	ပ	۵	Ħ	ស	ы	ы	м	Ħ	N/0
BUILDING #	401	403	403	403	403	403	403	403	403	, 403	403	403	403	403	403
CONTAINER I.D. #	CC-2212-0587	GC-2002-0587	CC-2300-0587	CC-2300-0587	CC-2301-0587	CC-2301-0587	CC-2301-0587	CC-2302-0587	CC-2304-0587	CC-2304-0587	CC-2305-0587	CC-2305-0587	CC-2306-0587	CC-2306-0587	CC-2298-0587

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	COMMENTS	BLENDER TANK. PROBABLY EMPTY.		5-GALLON FUEL CONTAINER.	1 16-02 JAR OF WHITE MATERIAL.	1-GALLON RUSTED CAN.	2 80-POUND BAGS OF "DIATOMITE".	OIL TANK, 1'D X 4'H.	6 300-POUND FIBERBOARD CONTAINERS OF LITHIUM CHLORIDE.	10 400-POUND FIBERBOARD CONTAINERS OF BARIUM FLUORIDE.	3 100-POUND FIBERBOARD CONTAINERS OF CEMENT.	2 1-GALLON CANS, UNKNOWN.	1 2-GALLON CAN OF CARBON GRAPHITE.	MISC. BOTILES OF UNKNOWN.	1 400-POUND FIBERBOARD DRUM OF BARIUM FLUORIDE.	1 300-POUND FIBERBOARD DRUM OF LITHIUM CHLORIDE.
	PID READINGS	0	0	0		0	6	0	0	o	• •	0	0	0	o	0
	INTEGRITY		Falr		Fair	Poor	Poor		Poor	Poor	Poor	Falr	Fair		Felt	ਜ ਵ ਜ
	COLOR				White											
	PHASE	Llquld	Liquid	Liquid	Solld		Solid	Liquid	Solid	Solid	Solid	Llquld	Solid	Liquid	Solid	Solid
•	FUNCTIONAL GROUP	Unknown	Unknown	Solvent	Unknown	Unknown		Lubricant				Unknown		Unknown		
	VOLUME EST. (ft3 OR GAL.)	00.0	5.00	5.00	0.04	1.00	2.00	25.00	24.00	40.00	4.00	2.00	0.27	0.25	4.00	00.4
	NUMBER VOLUME (ft3 OR		7	ч	п		8	Ħ	vo	10	en	8	ı	7		Ħ
	CONTAINER TYPE	Other	5 Gallon Can	5 Gallon Can	Jar	1 Gallon Can		Tank	Fiberboard	Fiberboard	Fiberboard	1 Gallon Can	Jar	Bottle	Fiberboard	Fiberboard
	ROOM	2ND	<	<	<	<	<	<	<	<	<	∢	<	<	<	∢
	BUILDING #	404	404	404	404	404	404	404	404	404	707	707	404	404	404	404
	CONTAINER I.D. #	CC-2235-0587	CC-2225-0587	CC-2225-0587	CC-2225-0587	CC-2225-0587	CC-2225-0587	CC-2229-0587	CC-2231-0587	CC-2231-0587	CC-2231-0587	CC-2231-0587	CC-2231-0587	CC-2231-0587	CC-2232-0587	CC-2232-0587

COMMENTS	HYDRAULIC AIR TANK.	REACTION VESSEL, 6'H X 5'D. PROBABLY EMPIY.	HEATER TANK, 1.5'D X 4'H. PROBABLY EMPTY.	UNKNOWN, BLUE 55-CALLON DRUM, PCB CONTAMINATED @ 3 PPM. FLASHPOINT < 0C.	RUSTED HOPPER.	SMALL STAINLESS STEEL HOPPER.	10 MISC. JARS AND CANS (HYDRAULIC FLUID, UNKNOWNS, CUTTING FLUID, CLEANERS, SAMPLE MATERIAL, OIL, ETC.)	2 CARBAGE CANS LABELED SODA ASH.	2 10-GALLON BUCKETS LABELED "NAK SPILLAGE ONLY".	1 GAL BUCKET OF BLACK UNKNOWN LIQUID.	OPEN 55-GALLON DRUM, HALP FULL OF BROWN MATERIAL.	EMPTY, RUSTED 55-GALLON DRUM.	BLUE TANK, 3'D X 9'H. PROBABLY EMPTY.	OPEN, EMPTY, GRAY TANK.
PID READINGS	0	0	o	0	0	0	0	0	•	0	o	0	0	0
INTEGRITY		Falr	e H	poog	Poor			Fair	Falr	Fair	Good	Poor	Fair	Good
COLOR				Yellow				White		Black				
PHASE	Se s	Liquid	Liquid	Liquid	Sol 1d	Solid	Liquid	Solid	Solid	Liquid	Solld		Liquid	
FUNCTIONAL GROUP	Gas	Unknown	Water	Lubricant	Unknown	Unknown	Mixture	Das se		Unknown	Unknown	Empty	Unknown	Empty
VOLUME EST. (ft3 OR GAL.)	0.00	0.00	0.00	55.00	00.0	0.00	10.00	5.00	2.00	1.00	3.00	0.00	00.0	0.00
NUMBER VOLUME (ft3 OR	.	1		F.	-	0	10	7	2	FI.	~	Ħ		
CONTAINER TYPE	Tank	Other	Other	Drum: Bung Top	Hopper	Hopper	nai.	Other	Other	Other	Drum: Open Top	Drum: Open Top	Tank	Tank
ROOM	∢	∢	<	< -	pt)	U	Ω	м	ы	ស	E/0	N/0	HST	WST
BUILDING #	404	404	404	704	404	404	404	404	. 404	404	404	404	404	404
CONTAINER I.D. #	CC-2232-0587	CC-2234-0587	CC-2236-0587	CC-2237-0587	CC-2227-0587	CC-2228-0587	CG-2230-0587	CC-2233-0587	CC-2233-0587	CC-2233-0587	CC-2239-0587	CC-2018-0587	CC-2238-0587	CC-2238-0587

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CONTAINER I.D. #	BUILDING #	ROOM	CONTAINER TYPE	NUMBER	NUMBER VOLUME EST. (ft3 OR GAL.)	FUNCTIONAL	PHASE	COLOR	INTEGRITY	PID READINGS	сомментѕ
CC-2240-0587	405	N/0	.Drum: Open Top	-	0.00	Empty	! ! ! ! ! ! ! !	1 1 1 1 1 1	Good	0	EMPIY RADIOACTIVE 55-GALLON DRUM. 150 K CPM BETA. GAMMA.
CC-2176-0587	904	<	5 Gallon Can	1	5.00	Tar	Liquid		Fair	0	1 5-GALLON CAN OF POLYCLAD.
CC-2176-0587	904	<	·	6	9.00	ង ១.s.c.	Solid		Fair	0	9 BAGS OF TYPE "S" LIME.
CC-2176-0587	904	< <	Bottle	п	1.00	Solvent	Liquid		booð	0	1-GALLON BOTILE OF "ALCOHOL".
CC-2178-0587	907	æ	Drum: Open Top	8	14.70	Trash	Solid		Good	0	55-GALLON DRUMS FILLED WITH SAMPLE MATERIAL OR IRASH.
CC-2179-0587	907	æ	5 Gallon Can	47	10.00	i e I	Liquid	1	Fi Fi	0	FLINTKOTE "LEVEL-KOTE",
CC-2303-0587	407	6.5	5 Gallon Can	Ħ	0.27	Unknown	Llquid	Yellow	Good	E	2-CALLONS OF OPAQUE YELLOW FLUID IN A 5-GALLON STEEL OVERPACK. TAKEN FROM HOOD #8. RADIOACTIVELY CONTAMINATED.
CC-2280-0587	407	88	Gas Cylinder	8	00.0	Gas	Sas		म म	o	2 LECTURE SIZE GAS CYLINDERS.
CC-2284-0587	407	0/8	Other	ĸ	00.0	Empty				O	8 5-GALLON EMPTY BUCKETS.
CC-2284-0587	407	0/8	Bottle	37	37.00	Acid	Liquid		Good	o	37 1-GALLON BOTTLES OF 70% NITRIC ACID.
CC-2284-0587	407	0/8	80 ब स्र	М	3.00	Unknovn	Solid	White	Poor	o	3 SPILLED 100-POUND BAGS OF UNKNOWN MAIERIAL.
CC-2279-0587	407A	61	Gas Cylinder	7	0.00	Gas	Gas			0	2 GAS CYLINDERS, PROBABLY EMPTY.
CC-2279-0587	407A	61	Вож	7	00.0	Battery	Solld		Fair	0	7 BOXES (12 EACH) OF "C" BATTERIES.
CC-2279-0587	407A	61	Bottle	vo	00.00	Unknown			Fair	0	6 MISC GLASS BOITLES, PROBABLY EMPIY.
CC-2280-0587	407A	89	Jar	Ħ	0.10	Lubricant	Liquid		poog	0	EYER-DROPPER OF RED, OILY LIQUID.

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COMMENTS	ONE EMPTY, ONE FULL OF UNKNOWN MATERIAL.	1-GALLON FUEL CAN.	1-GALLON OF WHITE OIL.	EMPTY ALUMINUM TANKS OF DISTILLED WATER, 1400 GALLONS EACH.	ASPHALT COATING.	MISC.; 2 1-PINT CANS OF CEMENT AND CREASE, 1-GAL OF THINNER, 1-GAL OF BITUMINOUS PAINT, BOX OF CHEM-E-PUCK ASBESTOS PACKNG, AND 1-GAL OF THINNER.	COMPRESSOR TANK, 3.5'D X 5'H	SQUARE TANK, 2' X 3' X 9".	55-GALLON DRUM CONTAINING LIGHT BULBS.	EMPTY FIVE GALLON CAN.	EMPTY ONE GALLON CAN.	UNKNOWN CONTENTS.	20 1-QUART PLASTIC BOTTLES OF HYDROFLUORIC ACID.	2-POUNDS OF "PEERAMID H-19".	5-GALLON CONTAINERS: UNKNOWNS, METHLY ALCOHOL, MAGNESIUM OXIDE, ETC.
PID READINGS	0	0	0	0	0	0	o	o	0		0	0	0	0	0
INTEGRITY		Fair	Falr	Falr	Fair	Ps 1.	Falr	Falr	Fair	Falr	Falr	14 14 14	Falr	Falr	Felr
COLOR			White												
PHASE	Liquid	Liquid	Liquid		Liquid	Liquid		Liquid	Solid			Liquid	Liquid	Liquid	Llquid
FUNCTIONAL GROUP	Unknovn	Solvent	Lubricant	Empty	T n	Solvent	Gas		Trash	Емрту	Empty	Unknown	Acid	Unknown	Mixture
NUMBER VOLUME EST. (ft3 OR GAL.)	5.00	1.00	1.00	0.00	5.00	4.00	00.00	30.00	7.35	0.00	00.00	1.00	5.00	0.01	55.00
NUMBER	2	4	Ħ	6	-	v	ī	#	1	Ħ	1		20	#4	#
CONTAINER TYPE	5 Gallon Can	1 Gallon Can	1 Gallon Can	Tank	5 Gallon Can	Other	Tank	Tank	Drum: Open Top	5 Gallon Can	1 Gallon Can	1 Gallon Can	Bottle	Bottle	5 Gallon Can
ROOM	E/R	E/R	E/R	E/R	E/R	E/R	E/R	E/R	W/R	W/R	W/R	89	17	11	11
BUILDING #	407A	407A	V 01 V	407A	407A	407A	¥07	407A	407A	407A	407A	4078	407D	407D	407D
CONTAINER I.D. #	CC-2276-0587	CC-2276-0587	CC-2276-0587	CC-2276-0587	CC-2277-0587	CC-2277-0587	CC-2278-0587	CC-2278-0587	CC-2275-0587	CC-2275-0587	CC-2275-0587	CC-2280-0587	CC-2282-0587	CC-2282-0587	CC-2282-0587

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COMMENTS	1-40-GALLON FIBERBOARD CONTAINER "LIQUID HEAT".	100-POUND, 30-GALLON CONTAINER OF BARIUM FLUORIDE.	55-GALLON DRUM OF CALCIUM METAL.	30-GALLON AND 40-GALLON CONTAINERS OF UNKNOWN MATERIAL.	5-QUART CONTAINER OF MERCURY, 3 FULL, 2 HALF FULL. 1-20Z BOTILE.	2'D X 6'H, POSSIBLY ASBESTOS LINED STEAM TANK. PROBABLY EMPTY.	4 100-POUND BAGS OF ACTIVATED CARBON.	1 PINT OF 3M ACTIVATOR FLUID, 1/2 PINT OF FINOL HOUSEHOLD OIL.	SMALL PROPANE GAS CYLINDER.	CONTAINERS PARTIALLY FULL, UNKNOWN LIQUIDS, SOME SOLIDS.	2-30 GAL. DRUMS LABELED LUBRIPLATE.	WHITE POWDER.	LABELED WESSON OIL.	LABELED CEMENT ADHESIVE.
PID READINGS	0	0	0	0	0	0	0	0	0	0	0	0	0	0
INTEGRITY	Fair	Falr	Fair	Falr	Falr	e ta	Fair	Good	poog		Good			Poor
COLOR												White		
PHASE	Solid	Sol1d	Solid	Solld	Liquid	Liquid	Solid	Liquid	s a g	Liquid	Llquld	Solid	Liquid	Solid
FUNCTIONAL GROUP				Unknown	Chemicals	Water		Solvent	Gas	Unknown	Lubricant	Unknown	Lubricant	Tar
NUMBER VOLUME EST. (ft3 OR GAL.)	5.35	4.00	7.35	9.36	1.50	0.00	4.00	10.00	0.10	15.00	60.00	4.00	5.00	0.67
NUMBER	н	Ħ	Ħ	8	v	r.	-4	7	#	36	8	T	1	1
CONTAINER TYPE	Fiberboard	Fiberboard	Drum: Open Top	Fiberboard	Other	Tank	80 80 80	Bottle	Gas Cylinder	Other	Drum: Open Top	Fiberboard	5 Gallon Can	5 Gallon Can
ROOM	11	11	11	17	11	34	34	34	⋖	<	∢	<	<	<
BUILDING #	407D	4070	407D	407D	4070	407D	407D	4070	, 408A	408A	408A	408A	408A	408A
CONTAINER I.D. #	CC-2282-0587	CC-2282-0587	CC-2282-0587	CC-2282-0587	CC-2282-0587	CC-2281-0587	CC-2281-0587	CC-2281-0587	CC-2096-0587	CC-2096-0587	CC-2097-0587	CC-2097-0587	CC-2097-0587	CC-2097-0587





COMMENTS	55 GAL DRUM IS EMPTY LABELED "KEROAINE".	QUART JAR LABELED "ISOTOX."	OIL BATHS. RADIOLOGICALLY CONTAMINATED. FLASHPOINT > 110C.	1 QT. GLASS JAR LABELED "MOTOR OIL."	10 GALLON CONTAINERS ARE COLLAPSED AND SPILLING. CONTAIN ASPHALT.	LABELED "LEVEL-KOTE".	MISC. INCLUDING PROPANE, INK, HONING OIL AND "ROSINESBALINE".	5 UNKNOWN 1 QUARI BOTTLES	WALL TILE, 1 GAL CAN & 1 QT BOTTLE ETHYLENE DICHLORIDE, 1 QT BOTTLE ETHYLENE CHLORIDE, AND SMALL CAN GLAZING COMPOUND.	SPRAY PAINT CAN.	2-80 POUND BAGS.	ANTI-RADIOACTIVE CLEANER.	Anti-Freeze.	8 MISCELLANEOUS GLASS BOTTLES AND CANS LABELED "ELECTROPLATES", "SODIUM CYANIDE", "SUPER-SOLV", "WELDING FLUX" AND "DYKEM SIEEL BLUE".
PID READINGS	0	0	6	0	0	0	o	o	o	0	0	0	0	0
INTEGRITY		Poog	Falr	goog	Poor	Poor			poog			Good		Cood
COLOR			Black		Black	Black						Yellow		
PHASE		Liquid	Llquld	Liquid	Solid	Solid	Liquid	Liquid	Liquid		Solld	Liquid		Liquid
FUNCTIONAL GROUP	Empty			Lubricant	Tar	Tar	Mixture	Mixture	Chemicals	Empty	Unknown	Solvent		Mixture
NUMBER VOLUME EST. (ft3 OR GAL.)	00.0	0.25	25.00	0.25	5.35	10.00	1.00	1.25	2.00	00.00	1.00	1.00	1.00	5.00
NUMBER		1	м	1	∢*	7	4	٧	47	Ħ	7	г	ī	∞
CONTAINER TYPE	Drum: Open Top	Bottle	Other	Jar	Fiberboard	5 Gallon Can	Other	Other	Other	High Pressure	13 pg	Bottle	Bottle	Other
ROOM #	<	<	<	æ	æ	æ	æ	æ	£ C) .	m	m	æ	ρQ	ပ
BUILDING #	408A	V 807	408 A	408 A	4 08 A	408A	408 A	V807	408A	408 A	408A	408 A	408 A	408 A
CONTAINER I.D. #	CC-2098-0587	CC-2099-0587	CC-2307-0587	CC-2099-0587	CC-2100-0587	CC-2100-0587	CC-2106-0587	CC-2106-0587	CC-2110-0587	CC-2110-0587	CC-2121-0587	CC-2121-0587	CC-2121-0587	CC-2103-0587

COMMENTS	MISCELLANEOUS JARS & BOTTLES INCLUDING ISOPROPYL, PROPANE, SILVER PLATING POWDER, LEAD SEAL AND CLEANING COMPOUNDS.	ELECTROLYTE	FUEL CAN. POSSIBLY EMPTY.	LABELED "B B".	PLASTIC CONTAINERS	DRUM WITH SPICOT ON CART LABELED "LIQUID CAUSTIC POTASH".	FUEL CAN	30 GAL, DRUM LABELED "EXIDE NICKEL ALKALINE - BATTERY ELECTROLITE",	PAINT CAN.	MISC. CANS AND JARS.		PLASTIC 1 CALLON BOTTLE.	PROTECTIVE COATING/THINNER/CEMENT.	S GALLON SQUARE.	PAINT CANS/THINNERS.
PID	o	0	o	0	0	0	0	0	0		0	0	0	0	0
INTEGRITY	poog	Good				Good	·	Poor				Poog			
COLOR															
PHASE	Llquid	Liquid	Liquid	Llquld		Liquid	Liquid	Liquid	Llquld	Liquid			Liquid	Llquld	Liquid
FUNCTIONAL GROUP	Mixture		Solvent	Lubricant	Empty	Base	Solvent		Paint	Mixture	Емрту	Empty	Solvent	Solvent	Solvent
NUMBER VOLUME EST. (ft3 OR GAL.)	2.00	30.00	5.00	10.00	00.0	55.00	5.00	30.00	1.00	2.00	0.00	0.00	10.00	5.00	7.00
NUMBER	6	1	1	8	2	7	1	Ħ	Ħ	15	1	+ 1	е	-4	,
CONTAINER TYPE	. Other	Drum: Open Top	5 Gallon Can	5 Gallon Can	Other	Drum: Bung Top	5 Gallon Can	Drum: Bung Top	1 Gallon Can	Other	5 Gallon Can	Bottle	5 Gallon Can	5 Gallon Can	1 Gallon Can
ROOM	U	υ	Q	Q	Q	Q	۵	Q	ស	M	БĪ	ы	ы	ш	ស
BUILDING #	408A	¥08¥	408A	408A	408A	¥807	408A	408 A	408A	408A	408A	408A	408A	V 807	408A
CONTAINER I.D. #	CC-2104-0587	CC-2105-0587	CC-2101-0587	CC-2101-0587	CC-2101-0587	CC-2102-0587	CC-2102-0587	CC-2102-0587	CC-2107-0587	CC-2107-0587	CC-2107-0587	CC-2107-0587	CC-2108-0587	CC-2108-0587	CC-2108-0587



COMMENTS	SALT TABLETS. 100 TABLETS TO BOX.	PROTECTIVE COATING.	FUEL CAN	1/2 PINT OF ADHESIVE.	POLYCLAD THINNER.	TRASH.	FLOOR SWEEPING COMPOUND.	WHITE POWDER. FOUND TO BE CORROSIVE. FLASHPOINT OF 110C.	MISC. CANS AND BOTTLES.	PAINT CANS.	. 5 GAL FUEL CAN	TRASH.		(2114A) "SPECIAL SAFETY SOLVENT" NAVY BRAND PRODUCTS. FLASHPOINT < 0C.	(2114B) UNLABELED. FLASHPOINT - 78C.	
PID READINGS	0	0	0		0	0	o	Đ	0	6	0	0	0	0.4	80	0
INTEGRITY	Good													Good	Fair	
COLOR								White						Yellow	Amber	
PHASE	Solld	Liquid	Liquid	Liquid	Liquid	Sol 1d	Solid	Solid	Liquid	Liquid	Liquid	Solid		Liquid	Liquid	
FUNCTIONAL GROUP	Base	Tar	Solvent	Tar	Solvent	Trash	Unknown	Unknown	Unknown	Paint	Solvent	Trash	Empty	Solvent	Solvent	Empty
VOLUME EST. (ft3 OR GAL.)	00.00	5.00	1.00	0.02	5.00	5.35	7.35	7.35	2.00	1.00	5.00	7.35	0.00	55.00	55.00	00.0
NUMBER VOLUME (ft3 OR	4	4	Ħ	-		₽	-	Ħ	^	1	Ħ	Ħ			1	7
CONTAINER TYPE	Other	5 Gallon Can	Other	Other	5 Gallon Can	Other	Fiberboard	Drum: Open Top	Other	1 Gallon Can	5 Gallon Can	Drum: Open Top	Drum: Bung Top	Drum: Bung Top	Drum: Bung Top	5 Gallon Can
ROOM ₩	м	84	<u>Bu</u>	£u,	Es,	ၒ	o	ಅ	ပ	ပ	ტ	ဖ	ဖ	æ	Ħ	æ
BUILDING	408A	408A	408A	408A	408A	408A	¥08¥	408A	408A	408A	4084	408 A	408 A	408A	408A	¥807
CONTAINER I.D. #	CC-2108-0587	CC-2109-0587	CC-2109-0587	CC-2109-0587	CC-2109-0587	CC-2111-0587	CC-2111-0587	CC-2111-0587	CC-2112-0587	CC-2112-0587	CC-2112-0587	CC-2113-0587	CC-2113-0587	CC-2114-0587	CC-2114-0587	CC-2115-0587

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COMMENTS	4-802 JARS OF SUPER-SOLV.	MISC. CANS AND PLASTIC CONTAINERS.		FUEL CAN.	ACOUSTIC CEMENT.	"MOBILE" OIL.	"SUNISO".	MISC. 1 QT PLASTIC CONTAINERS.	DOWFLAKE 77-80% CALCIUM CHLORIDE.	5 GALLON OIL TANK (1D X 2H).	AMOLUBE.	FUEL CONTAINER.	LUBRICANT	2 AUTOMOBILE 12V BATTERIES.	1 QUART CONTAINERS OF OIL, SOLVENTS, AND LUBRICANTS.	ONOX SKIN TOUGHENER.	CENTURY SPRAY UNIT.
PID READINGS	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0
INTEGRITY		poog			poog	Good	Good	Good		Falr							
COLOR	White																
PHASE	Liquid		Liquid	Liquid		Liquid	Llquid	Liquid	Solid	Liquid	Liquid		Liquid	Solid	Liquid	Liquid	
FUNCTIONAL	Solvent	Empty	Unknown	Solvent	Tar	Lubricant	Lubricant	Solvent		Lubricant	Lubricant	Empty	Lubricant	Battery	Solvent	Base	Empty
NUMBER VOLUME EST. (ft3 OR GAL.)	0.10	0.00	55.00	1.00	5.00	5.00	5.00	1.00	3.00	5.00	5.00	0.00	5.00	0.00	5.00	0.13	00.00
NUMBER	4	9	н	н		#	1	vo	vo	н	8	0		8	25	П	Ħ
CONTAINER TYPE	Other	Other	Drum: Bung Top	1 Gallon Gan	5 Gallon Can	5 Gallon Can	5 Gallon Can	Other	Bag	Other	5 Gallon Can	5 Gallon Can	5 Gallon Can	Other	Other	1 Gallon Gan	Drum: Bung Top
ROOM	æ	æ	0/M	'n	ה	ר	ט	ט	٠,	ט	×	×	×	×	×	×	×
BUILDING #	408A	V 807	¥807	4088	4088	408B	4088	408B	4083	4088	, 4088	408B	4088	4088	408B	4088	4088
CONTAINER I.D. #	CC-2115-0587	CC-2115-0587	CC-2145-0587	CC-2116-0587	CC-2116-0587	CC-2116-0587	CC-2116-0587	CC-2116-0587	CC-2117-0587	CC-2118-0587	CC-2119-0587	CC-2119-0587	CC-2119-0587	CC-2119-0587	CC-2119-0587	CC-2119-0587	CC-2120-0587



PID COMMENTS READINGS	0 10-GAL CONTAINER "OLIN - HTH", CHLORINE.	0 NITROGEN CYLINDERS, UN 1002, 4 1/2 PT. POR ACTIVE USE.	O SOME RUSTED OIL AND WATER - "FOAM".	54 ON CART WITH A SPIGOT. DRUM ON SIDE ON A RACK. FLASHPOINT (2139A) = 80C. FLASHPOINT (2139AA) = 74C.	0 FUEL, SOLVENT CANS.	0 LECITE BINDER.	O 6-POUND BAGS OF "ADHESO" DRY PASTE.	O INK, SPRAY CAN, ETC.	0 4 5-GAL. SQUARE CONTAINERS.	0 . "SHERLOCK 5-SECOND LEAK DETECTOR."	O UNKNOHN.	0 2-80Z SUPER-SOLV.	0 LIQUID ROOF.	0	0 80 POUND BACS OF CALGON GLASSY SODIUM PHOSPHATE.	0 100 POUND BAG OF CALCIUM CHLORIDE.
				v 1												
INTEGRITY				Poog												Poor
COLOR				Brown												
PHASE	Solid	Gas	Liquid	Liquid	Liquid	Liquid	Solid	Liquid	Liquid	Liquid		Liquid	Liquid		Sol1d	Solid
FUNCTIONAL GROUP	Chemicals	G as a	Solvent	Unknown	Solvent	Unknown		Mixture	Solvent	Unknown	Unknown	Solvent	Tar	Empty		
VOLUME EST. ft3 OR GAL.)	1.34	0.00	2.00	55.00	10.00	5.00	1.00	0.10	10.00	1.00	1.00	0.25	5.00	7.35	2.50	0.50
NUMBER VOLUME (ft3 OR	-	^	×١	T.	e	1	ю	•		- 4	ч	8	Ħ	Ħ	'n	
CONTAINER TYPE	Other	Gas Cylinder	5 Gallon Can	Drum: Bung Top	5 Gallon Can	5 Gallon Can		Other	Other	1 Gallon Can	1 Gallon Can	Other	5 Gallon Can	Drum: Bung Top		80 82
ROOM	×	0/8	0/14	Ε/0	<u>p</u>	e.	Ď,	e,	<u>α</u> ,	д	<u>Α</u> ,	£ι	А	ы	Д	ρı
BUILDING #	4088	408B	4088	780 7	4080	4080	408C	2807	408C	. 780 <i>t</i>	408C	408C	7807	708C	408C	4080
CONTAINER I.D. #	CC-2120-0587	CC-2146-0587	CC-2147-0587	CC-2139-0587	CC-2133-0587	CC-2133-0587	CC-2133-0587	CC-2133-0587	CC-2133-0587	CC-2133-0587	CC-2133-0587	CC-2133-0587	CC-2133-0587	CC-2133-0587	CC-2134-0587	CC-2135-0587

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COMMENTS	UNKNOWN 100 POUND BAGS.	CEMWAX THEMO CONDUCTING CEMENT BY CHEMAX.	NUCLEAR CUTIING OIL.	STENCIL INK.	CONTAINS BRICKS.	CUTTING AND SCARFING POWDER BY OXWELD.	60 POUND GUNNY SACKS CONTAINING BROWN POWDER.	MISC: 2-BATTERIES, 6-QTS OF SULFURIC ACID ELECTROLYTE, 5-GAL OF FUEL, 5 5-GAL OF OIL AND LUBRICANT, 2 1-GAL OF LUBRICANT, GAS TANKS FOR TRACTOR AND FORKLIFT, PORTABLE GREASER, AND 11 CANS AND JARS.	EMPTY 55-GALLON DRUM ON HOLDER.	ANTIFREEZE.	30-GALLON DRUM OF MULTI-LUBE.	40-GALLON DRUM OF RAGS.	EMPTY.	RUSTED, BULGING 55-GALLON DRUM.	10-GALLON FUEL CONTAINER.
PID READINGS	0	0	0	o	0	0	0	0		0	0	0	0	0	0
INTEGRITY	Poor				Poor					Falr	Fair	Falr		Poor	
COLOR							Brown								
PHASE	Solid	Liquid	Liquid	Liquid	Solld	Solid	Solid	Liquid		Liquid	Liquid	Solid			
FUNCTIONAL GROUP	Unknown	Tar r	Lubricant	Solvent		Unknovn	Unknown	Mixture	Empty	Solvent	Lubricant	Trash	Емрту	Емрtу	Empty
NUMBER VOLUME EST. (ft3 OR GAL.)	1.00	5.00	5.00	2.00	00.0	0.67	0.50	10.00	0.00	55.00	4.00	0.53	0.00	00.0	1.30
NUMBER	8	∗	H	Ħ	7	п	8	25	ed .	-	T	н	ਜ	4	
CONTAINER TYPE	88 88	5 Gallon Can	5 Gallon Can	5 Gallon Can	Fiberboard	5 Gallon Can	80 81 90	Other	Drum: Bung Top	Drum: Open Top	Drum: Open Top	Drum: Open Top	Tank	Drum: Bung Top	Other
ROOM	p.	ρ,	<u>α</u> ,	e,	<u>A</u>	ē.	£,	ρ.,	D.	ē.	Ω.	A	e.	p,	ρι
BUILDING #	408C	2807	408C	7807	408C	7807	4080	408C	7807	7807	4080	4080	408C	7807	780 ₇
CONTAINER I.D. #	CC-2135-0587	CC-2136-0587	CC-2136-0587	CC-2136-0587	CC-2136-0587	CC-2136-0587	CC-2136-0587	CC-2165-0587	CC-2166-0587	CC-2167-0587	CC-2167-0587	CC-2167-0587	CC-2168-0587	CC-2169-0587	CC-2169-0587



COMMENTS	EMPTY 80-GALLON OVERPACK DRUM ON CART.	5 GAL OF INDUSTRIAL SOAP.	LECTURE PROPANE BOTILE.	MISC. JARS: LIQUID SNAKE SINK CLEANERS, GLYCRIN, RED CACE OIL, ADHESIVE, PIPE JOINI COMPOUND, ACETONE, CHLOROFORM, LEAD LUBRICANI SEAL, BUFFERS, ETC.	2'H PROPANE, CYLINDER.	2'H GAS CYLINDERS.	BATIERY ELECTROLYTE.	SMALL 1.5V DRY CELL BATTERIES FOR LAB EQUIPMENT.	SPRAY PAINT CAN.	10-GAL. GREASE PUMPS.	INTERNATIONAL HY-TRAN FLUID.	MISC. CONTAINERS OF POSSIBLE OIL RESIDUES.	MOTOR OIL.	HYDRAULIC OIL	AUTOMATIC TRANSMISSION FLUID.
PID READINGS	0	0	0	o	0	0	0	0	0		0	0	0	6	0
INTEGRITY							Poor								Good
COLOR															
PRASE		Liquid	Gas	Liquid	623	Gas	Liquid	Solid	Gas	Liquid	Liquid	Liquid	Liquid	Liquid	Liquid
FUNCTIONAL			Gas	Mixture	Gas	Gas		Battery	Gas	Lubricant	Lubricant	Lubricant	Lubricant	Lubricant	Lubricant
NUMBER VOLUME EST. (ft3 OR GAL.)	0.00	5.00	0.00	5.00	0.00	0.00	3.00	0.20	0.00	40.00	5.00	1.00	110.00	55.00	55.00
NUMBER	H	1	Ħ	50	4	e	2	20	1	7	1	'n	8	1	Ħ
CONTAINER TYPE	Drum: Bung Top	Bottle	Gas Cylinder	Other	Gas Cylinder	Gas Cylinder	Вож	Other	High Pressure	Other	5 Gallon Gan	Other	Drum: Bung Top	Drum: Bung Top	Drum: Bung Top
#COM	<u>r</u>	ø	o	Ø	٥	0	æ	œ	æ	œ	æ	æ	œ	œ	æ
BUILDING #	408C	408C	408C	408C	¥08C	408C	408C	408C	. 2807	4080	\$08C	408C	408C	408C	4080
CONTAINER I.D. #	CC-2170-0587	CC-2137-0587	CC-2137-0587	CC-2137-0587	CC-2138-0587	CC-2138-0587	CC-2140-0587	CC-2140-0587	CC-2140-0587	CC-2140-0587	CC-2141-0587	CC-2141-0587	CC-2141-0587	CC-2141-0587	CC-2141-0587

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COMMENTS	B/B LUBRICATING, HYDRAULIC, OR MOTOR OIL.	20-GAL. OF MULTI-PURPOSE GREASE.	(2142A) DRUM LABEL ISOPROPYL ALCOHOL AND 3/4 FULL. FLASHPOINT < 0C.	MISC. GREASE, OIL AND PAINT.	UNKNOWN.	FUEL CAN.	UNKNOWN, DRUM MODIFIED.	1-GAL. FUEL CANS, FULL.	SMALL 4 02. BOTILE CONTAINING MERCURY.	5-GAL. FUEL CAN, FULL.	MISC. JARS OF OILS, GREASE, AND INK.	POLYCLAD.	JARS OF CLEANING COMPOUND.	SMALL 6-VOLT BATTERIES.	DRUM OF RAGS.	AUTOMOTIVE BAITERY.
PID READINGS	0	0	1000	0	0	0	0	0	0		0	0	0	0	0	0
INTEGRITY	goog		Fair													
COLOR			Clear													
PHASE	Liquid	Liquid	Liquid	Llquld			Liquid	Liquid	Liquid	Liquid	Liquid	Liquid	Liquid	Solid	Solid	Solid
FUNCTIONAL GROUP	Lubricant	Lubricant	Solvent	Mixture	Empty	Empty	Unknown	Solvent	Chemicals	Solvent	Lubricant	Solvent	Solvent	Battery	Trash	Battery
NUMBER VOLUME EST. (ft3 OR GAL.)	5.00	20.00	40.00	10.00	0.67	0.13	55.00	1.00	0.10	5.00	1.00	7.00	1.00	0.10	7.35	0.00
NUMBER	-	7	#	20	e	Ħ	Ħ	4	п	ᆏ	19	8	4	7	Ŧ	r.
CONTAINER TYPE	5 Gallon Can	Other	Drum: Open Top	Other	5 Gallon Can	1 Gallon Can	Drum: Bung Top	1 Gallon Can	Other	5 Gallon Can	Other	5 Gallon Can	Other	Other ·	Drum: Open Top	Other
ROOM	œ	æ	æ	æ	æ	œ	œ	w	ω	W	w	w	w	w	w	W/0
BUILDING #	408C	408C	7807	7080	7807	7807	40BC	408C	7807	408C	7807	408C	7807	408C	708C	7080
CONTAINER I.D. #	CC-2142-0587	CC-2142-0587	CC-2142-0587	CC-2142-0587	CC-2142-0587	CC-2142-0587	CC-2143-0587	CC-2144-0587	CC-2144-0587	CC-2144-0587	CC-2144-0587	CC-2144-0587	CC-2144-0587	CC-2144-0587	CC-2144-0587	CC-2147-0587

COMMENTS is	EMPTY PUEL CANS.	GLASS CARBOY IN WOOD BOX.	EMPTY 1 GALLON JAR.	CHLORINATED HYDROCARBONS, MONSANTO, INERTEEN. FLASHPOINT = 102C.	2 OPEN - POSSIBLY OVERPACK - WHITE POWDER.	RUSTED OPEN.	WITH SPIGOT ON CART. "ISOPROPYL ALCOHOL".	WITH SPIGOT, INDUSTRIAL SOAP CO., "JANITOR SUPPLIES".	BATTERY IN TRUCK PARKED'OUTSIDE. E-06339.	DEHYDRATING AGENT IN 20-GAL. DRUM.	4-80Z SUPER-SOLV JARS.	HONING OIL	INK, DIP-II CLEANER.	ANTIRADIOACTIVE CLEANING COMPOUND.	FREON, 15 POUNDS.
PID READINGS	0	0	0	100	o	0	0	0	0	0	0	0	6	0	0
INTEGRITY				Fair	Poog						Good				Poog
COLOR				Clear	White										
PHASE				Liquid	Solid	Liquid	Liquid	Liquid	Solid	Liquid	Liquid	Liquid	Liquid	Liquid	ກ ແ
FUNCTIONAL GROUP	Empty	Empty	Empty	Solvent	Unknown	Unknown	Solvent	9 8 8	Battery	Unknown	Solvent	Lubricant	Solvent	Solvent	Sa s
NUMBER VOLUME EST. (ft3 OR GAL.)	00.0	00.0	00.0	55.00	14.70	55.00	55.00	22.06	0.00	20.00	0.50	1.00	0.10	1.00	00.00
NUMBER	8	н	.	Ħ	7	п	-	m	1	e	4	Ħ	ю	ਜ	e
CONTAINER TYPE	5 Gallon Can	Bottle	Bottle	Drum: Bung Top	Drum: Open Top	Drum: Open Top	Drum: Open Top	Drum: Bung Top	Other	Other	Jar	1 Gallon Can	Jar	Bottle	Gas Cylinder
ROOM	z	z	z	z	z	z	z	×	N/0	0	•	0	o	o	0
BUILDING #	408D	4080	4080	408D	4080	408D	4080	4080	408D	4080	408D	408D	408D	408D	4080
CONTAINER I.D. #	CC-2122-0587	CC-2122-0587	CC-2122-0587	CC-2123-0587	CC-2124-0587	CC-2124-0587	CC-2124-0587	CC-2124-0587	CC-2218-0587	CC-2121-0587	CC-2125-0587	CC-2125-0587	CC-2125-0587	CC-2125-0587	CC-2126-0587

COMENTS	UNKNOWN GAS CYLINDERS.	5 POUND AMMONIA CYLINDER.	SPRAY PAINT CAN.	UNKNOWN 1-GAL.	1-GAL. UNKNOWN, PROBABLY EMPTY.	EMTY.	10-GAL. CONTAINER WITH MISC. CONTENTS (DRY POWDER, EIC.).	ANSUL MET-L-X DRY POWDER FOR FIRE EXTINGUISHERS. HEAVY POWDER. FLASHPOINT = 32C.	ANSUL FORAY DRY CHEMICAL.	80 POUND ANSUL WHITE POWDER. FLASHPOINT = 71G.	"BIOMACHINE LAKESEAL" SODIUM SILICATE.	REDUCED IRON.	SILICONE SPRAY CANS.	BLACK 55-GALLON DRUM. APPEARS TO CONTAIN VERMICULITE AND ELECTRIC WIRES.	COMPOUND MR-2000.
PID READINGS	0	0	0	. •	6	0	0	0			0	0	o	0	0
INTEGRITY	Good	Poog						Fair		lu mi mi pla				Falr	
COLOR								White		White				Brown	
PHASE	Gas	Se s	8 8				Solld	Sol1d	Solid	Solid	Solld	Solid	Gas	Sol1d	Liquid
FUNCTIONAL GROUP	Gas	Gas	Gas	Unknown	Unknown	Empty	Unknown	Unknown	Unknown	Unknown			Gass		Solvent
NUMBER VOLUME EST. (ft3 OR GAL.)	0.00	00.0	00.00	1.00	00.00	00.0	1.34	14.70	1.34	13.00	7.00	1.34	0.00	65.00	55.00
NUMBER	7		ī	Ħ	#	1	1	22	8	25	Ħ	8	4		Ŧ.
CONTAINER TYPE	Gas Cylinder	Gas Cylinder	High Pressure	1 Gallon Can	1 Gallon Can	5 Gallon Can	Other	5 Gallon Can	5 Gallon Can	80 80	Fiberboard	5 Gallon Gan	Gas Cylinder	Drum: Open Top	Drum: Open Top
ROOM #	0	0	0	o	0	0	0	0	0	0	0	0	o	0	0
BUILDING #	4080	408D	408D	408D	408D	408D	408D	408D	408D	408D	408D	408D	408D	408D	408D
CONTAINER I.D. #	CC-2126-0587	CC-2126-0587	CC-2126-0587	CC-2127-0587	CC-2127-0587	CC-2127-0587	CC-2127-0587	CC-2128-0587	CC-2129-0587	CC-2130-0587	CC-2131-0587	CC-2131-0587	CC-2131-0587	CC-2131-0587	CC-2131-0587





TY PID COMMENTS READINGS	0 UNKNOWN 25-GAL, DRUM,	0 2-80Z SUPER-SOLV.	0 40-GAL. GENERAL ELECTRIC PYRANOL (PCBS).	0 25 POUND MAGNESIUM OXIDE CONTAINERS.	0 1 GALLON CONTAINERS OF UNKNOWN.	0 EMPTY ONE GAL. CONTAINERS.	0 MISCELLANEOUS MATERIALS. SMALL QUANTITY CANS, BOTTLES, JARS ETC.	O 1-0-1		O FILLED WITH TRASH.	0 1-GAL. PLASTIC CONTAINERS, FULL AND PARTIALLY FULL; INCLUDING HYPONEUTRALIZER, EKTRACHROME FILM, ETC.	0 1-GAL, GLASS JARS.	0 1-GAL, METAL CONTAINERS.	0 1/2 PINT AND PINT CONTAINERS OF FILM DEVELOPERS.	0 QUART BOTILES OF RAPID FIXER.	
INTEGRITY																
COLOR			Black				; ; ;									
PHASE			Liquid	Solid	Liquid		Liquid	Light		Solid	Liquid	Liquid	Liquid	Liquid	Liquid	
FUNCTIONAL		Solvent		Chemicals	Unknown	Empty	Mixture	Solvent		Trash	Chemicals	Chemicals	Unknown	Chemicals	Chemicals	- C
NUMBER VOLUME EST. (ft3 OR GAL.)	25.00	0.00	10.69	4.00	2.00	0.00	0.10	1.00		7.35	20.00	2.00	2.00	5.00	3.00	•
NUMBER	н	2	8	∢	е	8	9	~	ı		35	,	Ŋ	100	12	•
CONTAINER IYPE	Drum: Open Top	Jar	Drum: Open Top	Fiberboard	Bottle	Bottle	Other	1 Gallon Can		Drum: Open Top	1 Gallon Gan	Jar	1 Gallon Can	Bottle	Bottle	, + + + + + + + + + + + + + + + + + + +
ROOM	o	o	0	0	0	0	EQ I	<	:	æ	U	ပ	v	ပ	v	ť
BUILDING #	4080	408D	408D	408D	408D	408D	408E	607		409	. 607	607	409	607	607	604
CONTAINER I.D. #	CC-2131-0587	CC-2131-0587	CC-2131-0587	CC-2132-0587	CC-2215-0587	CC-2215-0587	CC-2108-0587	CC-2241-0587		CC-2242-0587	CC-2243-0587	CC-2243-0587	CC-2243-0587	CC-2243-0587	CC-2243-0587	1850-8965-00

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CONTAINER I.D. #	BUILDING	коом	CONTAINER IYPE	NUMBER	NUMBER VOLUME EST. (ft3 OR GAL.)	FUNCTIONAL GROUP	PHASE	COLOR	INTEGRITY	PID READINGS	COMMENTS
CC-2244-0587	409	Q	Other	#	0.00	Empty				0	EMPTY 25-GALLON AMMONIA CONTAINER.
CC-2245-0587	409	ы	1 Gallon Can	œ	8.00	Unknown	Llquld			0	1-GALLON PLASTIC CONTAINERS.
CC-2245-0587	409	W	Drum: Open Top	e	0.00	Empty				0	3 TRASH DRUMS - EMPTY.
CC-2245-0587	607	ស	1 Gallon Can	4	4.00	Unknown	Liquid	Blue		0	1-GALLON CONTAINERS FILLED WITH UNKNOWN BLUE-GREEN LIQUID,
CC-2245-0587	604	ы.	Other	80	2.00	Unknown	Liquid	; ; ;		0	UNKNOWN 1 QUART CONTAINERS.
CC-2265-0587	410	•	Other	200	0.00	Chemicals	Solid			0	200 METAL CONTAINERS, PROBABLY EMPTY.
CC-2246-0587	410	<	5 Gallon Can	8	0.27		Sol 1d			0	PASTE WAX.
CC-2246-0587	410	≺	1 Gallon Can	ed	0.50	Water	Liquid			0	1 GAL PLASTIC BOTTLE WINDSHIELD CLEANER.
CC-2246-0587	410	<	Other	#	0.10	Days on the second of the seco	Liquid			0	1 QT PLASTIC BOITLE SODIUM HYDROXIDE
CC-2246-0587	410	∢ .	1 Gallon Can		1.00	Solvent	Liquid				FLOOR POLISH REMOVER.
CC-2246-0587	410	∢	Вож	٧٢	0.00		Solid			0	SPACKLING COMPOUND.
CC-2246-0587	410	∢	Bottle	m	2.00	Solvent	Liquid			0	3H ACTIVATOR PLUID FOR COPIERS (ONE EMPIY).
CC-2271-0587	410	*	Jar	m	0.50	Chemicals	Liquid			0	1 QT GLASS BOTTLES CONTAINING DEXTROSE.
CC-2271-0587	410	¥	Jar	12	3.00	Chemicals	Liquid		Pood	0	12 1-QT JARS OF CITROCARBONATE.
CC-2271-0587	410	¥	නි ස	ø	0.50	es a g	Solid		Poor	0	6 1-POUND BAGS OF SODIUM BICARBONATE.
CC-2271-0587	410	*	1 Gallon Can	7	0.05	Base	Solid			0	1-POUND CAN OF EPSOM SALT.

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COMMENTS	CLINITEST FOR URINE SUGAR, "POISON".	LAXATIVES.	1-GAL JARS OF MINERAL OIL, CALAMINE LOTION, ETC.	MISC.: ACIDS, MEDICAL COMPOUNDS.	12 1-PINT, UNKNOWN AND CORRODED.	KAMFOLENE SOLUTION. 40Z JAR.	MISC. JARS AND BOXES OF MEDICAL SUPPLIES.	CLEANERS AND 1/2 PT. OF PAINT.	WIII-0-GREEN CLEANER DISINFECTANT.	EMPTY SS CAN.	ISOPROPYL ALCOHOLS,, CLEANING FLUID, HAND CLEANERS.	3 2-02 OF INK AND 1 1-02 OF ERASING FLUID.	LIQUID WRENCH. 80Z CAN.	16 OZ OF SUPER SOLV	QUART CAN OF UNKNOWN CONTENTS.	
PID	0	0	•	o	0	0	0	0	0	0		0	0	0	0	
INTEGRITY					Poor											
COLOR																
PHASE	Liquid	Liquid	Liquid	Liquid	Liquid	Liquid	Liquid	Liquid	Liquid		Liquid	Llquid	Liquid	Liquid	Liquid	
FUNCT IONAL GROUP	Chemicals	Chemicals	Chemicals	Chemicals	Chemicals	Chemicals	Chemicals	Solvent	Solvent	Empty	Mixture	Solvent	Solvent	Solvent	Unknown	
VOLUME EST. (ft3 OR GAL.)	0.01	5.00	0.00	3.00	1.50	0.01	2.00	2.00	5.00	0.00	1.00	0.05	0.01	0.25	0.25 U	
NUMBER	4	٠	10	12	12	12	30	٠	1		-4	4	-	1	1	
CONTAINER TYPE .	Other	Вох	Teb.	Jar	Jar	ra L	Other	Other	5 Gallon Can	5 Gallon Can	Other	Jar	Other	na c	hat	
ROOM	*	¥	¥	\$	¥	¥	*	*	EQ.	æ	æ	8 0	33	93	BB	
BUILDING #	410	410	410	410	410	410	410	410	410	410	410	410	410	410	410	
CONTAINER I.D. #	CC-2271-0587	CC-2271-0587	GC-2271-0587	CC-2271-0587	CC-2271-0587	CC-2271-0587	CC-2271-0587	CC-2271-0587	CC-2247-0587	CC-2247-0587	CC-2247-0587	CC-2272-0587	CC-2272-0587	CC-2272-0587	CC-2272-0587	

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COMMENTS	MISC.: CLEANERS, ADHESIVES, ETC.	UNKNOWN CONTENTS.	INDUSTRIAL BATTERY.	1-GALLON PLASTIC CONTAINER OF ALCOHOL	1-GALLON OF "POSTER PERFECT" YELLOW PAINT	5-GAL, FUEL CAN.	CASE (~24 CONTAINERS) OF SALT TABLETS.	UNKNOWN HEAVY YELLOW LIQUID	2 POUND OF NITRIC ACID.	MISC: ALCOHOL, CLEANING COMP., MERCUROCHROME, ETC.	2 POUND BOXES OF "SBS SKIN CLEANER".	3 AEROSOL PAINT CANS.	SMALL JAR OF MERCURY.	16 OZ OF "SUPER-SOLV".	DEWITT INDUSTRIAL MAINTENANCE CLEANER.	MISC.: GREASE, RUBBER ADHESIVE, BLACK INK, ETC.
PID READINGS	•	0	0	0	0	0	0	6	0	O	0	0	0	0	0	0
INTEGRITY						Fair							Good			·
COLOR					Yellow			Yellow								
PHASE	Liquid	Liquid	Solid	Liquid	Liquid		Solid	Liquid	Liquid	Liquid	Solid		Liquid	Liquid	Liquid	Liquid
FUNCTIONAL GROUP	Solvent	Unknown	Battery	Solvent	Paint	Empty		Unknown	Acid	Chemicals		Empty	Chemicals	Solvent	Solvent	Solvent
VOLUME EST. (ft3 OR GAL.)	0.05	1.00	00.00	1.00	1.00	0.67	0.50	05.0	2.00	1.00	1.00	0.00	0.10	0.25	2.00	0.50
NUMBER	4	H	н	1	T	1	r.			10	8	Ħ	Ħ	1	н	٠
CONTAINER TYPE	Other	Bottle	Other	Bottle	1 Gallon Can	5 Gallon Can	Вож	Bottle	Bottle	Other	Вож	High Pressure	nap.	Jar .	5 Gallon Can	Other
ROOM	88	ပ	U	Q		۵	Ω	Q	Ω	۵	£Ω	м	ы	ы	ы	ы
BUILDING #	410	410	410	410	410	410	410	410	410	410	410	410	410	410	410	410
CONTAINER I.D. #	CC-2272-0587	CC-2248-0587	CC-2248-0587	CC-2249-0587	CC-2249-0587	CC-2249-0587	CC-2249-0587	CC-2249-0587	CC-2249-0587	CC-2249-0587	CC-2250-0587	CC-2250-0587	CC-2250-0587	CC-2250-0587	CC-2250-0587	CC-2250-0587



COMMENTS	SMALL 16 02. PROPANE BOTTLE.	1-POUND SODA ACID CHARGE FOR FIRE EXTINGUISHERS.	MET-L-X DRY POWDER FOR FIRE EXTINGUISHERS, FULL & RUSTED.	1-GAL OF SAFETY SOLV.	16 OZ OF "SUPER-SOLV".		OXYGEN-GENERATION CANISTERS FOR MSA RESPIRATORS.	1-GAL OF INDUSTRIAL FINISH ENAMEL - RED.	1-GAL. METALLIC MORTOR BY CHEMTREE.	1 1/2 POUNDS OF IIDE.	BLACK LIQUID, POSSIBLY OIL, ONOX.	SUPER SOLV CLEANER CONTAINING DARK LIQUID.	2-GAL BUCKET CONTAINING DARK LIQUID - POSSIBLY STANDING WATER.	ASBESTOS TANK 2.5°D X 4°H. PROBABLY EMPTY.	BOX OF ASBESTOS ROPE.
PID READINGS	o	0	0	0	0	0	0	0	.	0	0		0	0	0
INTEGRITY			Falr				Falr	Pood		•				Cood	
COLOR								Red			Black		Black		
PHASE	Gas	Solid	Sol 1d	Liquid	Liquid		Solid	Liquid	Solid	Solid	Liquid	Liquid	Liquid	Liquid	Solid
FUNCTIONAL GROUP	Gas			Solvent		Empty		Paint	Unknown	Base	Lubricant	Solvent	Water	Water	
NUMBER VOLUME EST. (ft3 OR GAL.)	0.00	0.50	3,35	1.00	0.25	00.00	3.00	1.00	0.27	0.01	0.50	0.10	2.00	00.00	00.00
NUMBER	-	8	σ.	1		Ħ	20		7	н	ч	- 4	н		1
CONTAINER TYPE	Gas Cylinder	Other	5 Gallon Can	1 Gallon Can	Jat	Drum: Open Top	Other	1 Gallon Can	1 Gallon Can	Вож	1 Gallon Can	Jar	Other	Tank	Вож
ROOM #	E/0	B4	£s.	De	įε.	ĵz,	₿u.	v	ပ	. 22	m	Ħ	Ħ	H	н
BUILDING #	410	410	410	410	410	410	410	410	410	410	410	410	410	410	410
CONTAINER I.D. #	CC-2299-0587	CC-2251-0587	CC-2251-0587	CC-2251-0587	CC-2251-0587	CC-2251-0587	CC-2251-0587	CC-2252-0587	. CC-2252-0587	CC-2253-0587	CC-2253-0587	CC-2253-0587	CC-2253-0587	CC-2254-0587	CC-2254-0587

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COMMENTS	YELLOW DRUM LABELED "TESTED 9/77 DOT R1117".	COFFEE CAN WITH OILY LIQUID.	THICK BLACK LIQUID.	FEND-X HAND CREAM.	ASBESTOS LINED TANKS, 6'D X 12'H. PROBABLY EMPIY.	ETHYLENE OXIDE NONIONIC (MONYL PHENOL) WITH SPIGOT.	FUEL CANS.	SALT SHAKER OF UNKNOWN CONTENTS.	PAIL OF OILY LIQUID.	HEAVY DUTY DRY RINSE.	4-02 GLASS OF WADE'S CAINFENAL SOLUTION - GLYCERINE AND WITCH HAZEL.	MISC.: CLEANERS, ETC.	ONOX SKIN TOUGHENER	TRANSMISSION FLUID.	INSECTICIDE.	MISC.: SOLVENT, CLEANERS, ENAMEL, INK, SALT TABLETS, ETC.
PID READINGS	0	0	6	0	0	0	0	0	0	6	0	0	0	0	0	o
INTEGRITY	Poor				Poog	Poop	Good				poog		Good			
COLOR			Black						•							
PHASE	Liquid	Liquid	Liquid	Liquid	Liquid	Liquid	Liquid	Solid	Liquid	Solid	Liquid	Liquid	Liquid	Liquid	Liquid	Liquid
FUNCTIONAL GROUP	Unknown	Lubricant	Lubricant	Solvent	Water	Solvent	Solvent		Lubricant		Chemicals	Solvent	Base	Lubricant	Unknown	Mixture
VOLUME EST. (ft3 OR GAL.)	55.00	1.00	5.00	0.10	0.00	55.00	2.00	0.00	5.00	0.67	0.10	0.10	1.00	1.00	1.00	1.00
NUMBER VOLUME (ft3 OR	-	Ŧ	1	1	6	↔			r.	1	-	en.	1	1	1	15
CONTAINER TYPE	Drum: Open Top	1 Gallon Can	5 Gallon Can	Other	Tank	Drum: Bung Top	1 Gallon Can	Other	5 Gallon Can	5 Gallon Can	Bottle	Other	Bottle	1 Gallon Can	1 Gallon Can	Other
ROOM	ה	٠,	×	×	×	×	×	ĸ	×	ı	H	1	×	Σ	Σ	Σ
BUILDING #	410	410	410	410	410	410	410	410	410	410	410	410	410	410	410	410
CONTAINER I.D. #	CC-2257-0587	CC-2257-0587	CC-2258-0587	CC-2258-0587	CC-2258-0587	CC-2258-0587	CC-2258-0587	CC-2258-0587	CC-2258-0587	CC-2259-0587	CC-2259-0587	CC-2259-0587	CC-2260-0587	CC-2260-0587	CC-2260-0587	CC-2260-0587

Comments	JOHNSON'S,	CONTAINS 1-GAL CANS AND MISC. CANS OF STERNO.	2 EMPTY, 1 FULL. CORRODED AND SPILLED.	RUSTED FUEL TANK, 5'D X 12'H. FLASHPOINT = 63C.	EMPTY AND RUSTED THROUGH ON SIDE.	EMPTY 5-GAL CARBOYS.	EMPTY 1-GAL BOTTLE.	QUART OF BROWNISH LIQUID.	1-GAL GLASS JAR OF MAGNESIUM SULFAIE.	PLASTIC BOTTLE OF SOAP.	BOILLE OF INK.	TETANUS TOXIOD ALUM.	MISC BOTTLES OF MEDICINE: COUCH SYRUP, ASPIRIN, LISTERINE, EYE WASH, CHLORASEPTIC, ETC.	PLASTIC BOTTLE OF KEROSENE.	UNKNOWN CLEAR LIQUID	EMPTY 1-GAL PLASTIC JARS.
PID READINGS	0	0	0	75	0	6	0	0	0	0	•	0	6	0	0	0
INTECRITY			Poor	Falr												
COLOR				Brown												
PHASE	Liquid	Llquid	Liquid	Liquid				Liquid	Solid	Liquid	Liquid	Liquid	Liquid	Liquid	Llquid	
FUNCTIONAL GROUP	Solvent	Solvent	Unknown	Solvent	Empty	Empty	Empty	Unknown	Chemicals	Base	Solvent	Chemicals	Mixture	Solvent	Unknown	Empty
VOLUME EST. (ft3 OR GAL.)	55.00	5.00	5.00	200.00	0.00	0.00	0.00	0.25	0.13	1.00	0.10	0.10	0.50	1.00	0.10	0.00
NUMBER VOLUME (ft3 OR	#	1	m	H	Ħ	2	r.			1	н	20	52		н	2
CONTAINER TYPE	Drum: Open Top	Drum: Open Top	5 Gallon Can	Tank	Drum: Open Top	5 Gallon Can	Bottle	Bottle	Jer	Bottle	Other	Other	Bottle	Bottle	Jar	1 Gallon Can
ROOM	×	z	z	N/0	0/N	<u>α</u> ,	ρι	e,	٥	٥	o,	o,	ø	w	w	W
BUILDING #	410	410	410	410	410	410	410	410	410	410	410	410	410	410	410	410
CONTAINER I.D. #	CC-2261-0587	CC-2261-0587	CC-2261-0587	CC-2255-0587	CC-2274-0587	CC-2262-0587	CC-2262-0587	CC-2262-0587	CC-2263-0587	CC-2263-0587	CC-2263-0587	CC-2263-0587	CC-2263-0587	CC-2264-0587	CC-2264-0587	CC-2264-0587

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COMMENTS	50 ML BOTTLES OF UNKNOWN.	1-PINT BOTTLES OF GENERAL PURPOSE INK.	UNKNOWN CLEAR LIQUID.	METAL CUTTINGS ON BLACK MATERIAL.	5-17 POUND BOXES OF SODIUM CARBONATE.	ONE EMPTY, 2 POSSIBLY EMPTY.	PARTICALLY FULL 10 GALLON CARBOY.	UNKNOWN,	2 GAL. SODIUM HYDROXIDE.	NITRIG ACID	MISC.: LAB CHEMICALS, LAB REACENTS, ACID, ETC.	2-GAL OF CALCIUM CHLORIDE.	1-GAL OF ACETONE.	HISC.: CHEMICALS.	EMPIY 5-GAL CARBOYS.	2-GAL CARBOY	UNKNOWN, HALF CORRODED AND SPILLED.
PID READINGS	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0
INTEGRITY													Good				Poor
COLOR																	
PHASE	Liquid	Liquid	Liquid	Solid	Solid		Liquid	Liquid	Liquid	Liquid	Liquid	Solid	Liquid	Liquid		Liquid	
FUNCTIONAL GROUP	Chemicals	Solvent	Unknown	Unknown	Chemicals	Empty	Unknown	Unknown	Ваѕе	Acid	Chemicals	Chemicals	Solvent	Chemicals	Empty	Unknovn	Unknown
NUMBER VOLUME EST. (ft3 OR GAL.)	0.10	0.50	0.50	0.01	1.00	00.0	2.00	60.00	2.00	1.00	2.00	0.27	1.00	2.00	0.00	0.27	2.00
NUMBER	e	'n	H	1	'n	m	ਜ	85	8	1	25	т	#	15	2	н	H
CONTAINER	Bottle	Bottle	Bottle	Bottle	Вох	Drum: Open Top	Bottle	1 Gallon Can	Bottle	Bottle	Jar	Jar	Bottle	Other	Bottle	Bottle	5 Gallon Can
ROOM	w	ហ	w	w	S	0/8	Ð	ū	Ð	Ð	n	>	>	>	3	3	3:
BUILDING #	410	410	410	410	410	410	410	410	410	. 410	410	410	410	410	410	410	410
CONTAINER I.D. #	CC-2264-0587	CC-2264-0587	CC-2264-0587	CC-2264-0587	CC-2268-0587	CC-2256-0587	CC-2265-0587	CC-2265-0587	CC-2265-0587	CC-2265-0587	CC-2265-0587	CC-2266-0587	CC-2266-0587	CC-2266-0587	CC-2267-0587	CC-2267-0587	CC-2267-0587



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COMMENTS	CARDBOARD SHIPPING TUBE LABELED "RADIOACTIVE MATERIAL".	2-GAL OF CONCENTRATED SODIUM THIOSULFAIE.	16 02 BOTILE CONTENTS UNKNOWN.	16-02 SQUEEZE BOTTLE. CONTENTS UNKNOWN.	12 16-02 JARS OF SODIUM FLUORIDE.	JAR OF ALUMINUM NITRATE.	8-02 JAR LABEL "KOI" WITH CRYSTALS ON THE BOTTOM.	SULFURIC AND ACETIC ACID.	PHOSPHORIC ACID.	16 OZ BOTTLE, LABELED KEL-F 300.	EYE DROPPER WITH UNKNOWN LIQUID.	2 1-QT OF ISOPROPYL ALCOHOL.	10-GAL CARBOY OF UNKNOWN LIQUID	AEROSOL CONTAINERS.	MISC.: NITRIC ACID, LEAD OXIDE, SULFURIC ACID, ETHYL ALCOHOL, SULFONIC ACID.
PID READINGS	0	0		0	0	0	0	6	6		0	0	0	0	0
INTEGRITY														Pood	
COLOR															
PHASE	Solld	Liquid	Liquid	Liquid	Solid	Solid	Solid	Liquid	Liquid	Liquid	Liquid	Liquid	Liquid	Q 88	Liquid
FUNCTIONAL GROUP	Unknown	Chemicals	Unknown	Unknown	Chemicals	Chemicals	Chemicals	Acid	Acid	Unknown	Unknown	Solvent	Unknown	Gas	Mixture
NUMBER VOLUME EST. (ft3 OR GAL.)	0.00	2.00	0.25	0.25	0.20	0.01	0.01	2.00	1.00	1.00	0.10	0.50	10.00	0.13	5.00
NUMBER	F	.		1	12			10	.	×۸	T.	2	.	10	20
CONTAINER TYPE	Other	Bottle	Bottle	Bottle	Jar	다 as P	Bottle	Jar	Jar	Bottle	Other	Bottle	Bottle	Other	Other
ROOM	3:	×	×	×	×	>	>	>	>	×	>	>	>	>	> -
BUILDING	410,	410	410	410	410	410	410	410	410	410	410	410	410	410	410
CONTAINER I.D. #	CC-2267-0587	CC-2268-0587	CC-2268-0587	CC-2268-0587	CC-2269-0587	CC-2269-0587	CC-2269-0587	CC-2269-0587	CC-2269-0587	CC-2269-0587	CC-2269-0587	CC-2269-0587	CC-2269-0587	CC-2270-0587	CC-2270-0587

COMMENTS	HEXANE	CARBON TETRACHLORIDE	16 OZ. JARS	AEROSOL CANS - RUSTED. LABELED "FINISH COAI",	10-GALLON FUEL CAN.	10-GALLON TRASH CAN.	INDUSTRIAL BATTERIES.	EMPIY QUAKER MAID CAN	EMPIY 4'D X 7'H TANK.	EMPTY 18'D X 25'H TANK.	EMPTY 5'D X 20'H TANK.	LOCOMOTIVE WITH GAS TANK AND BATTERY.	5'D X 8'H MODIFIED, FUEL TANK WITH 4 INCHES OF FLUID IN BOTTOM. SAMPLE WAS GRITTY. FLASHPOINT > 110C.	55-GALLON DRUM, CONTENTS UNKNOWN.	RUSTED 55-GALLON DRUM.
PID READINGS	0	o	6	0	0	0	0	0	0	0	0	0	80	0	0
Integrity		Good	Falr			म	Falt				 	Fair	Pair	Fair	Poor
COLOR		1 1 1 1 1	Black										Black		
PHASE	Liquid	Liquid	Liquid			Liquid	Solid					Liquid	Liquid	Liquid	Liquid
FUNCTIONAL GROUP	Solvent	Solvent		Empty	Empty	Trash	Battery	Empty	Empty	Empty	Empty	Solvent	Solvent	Unknown	Solvent
VOLUME EST. (ft3 OR GAL.)	5.00	0.50	0.50	0.00	00.0	1.34	0.00	0.00	00.00	00.0	0.00	00.00	50.00	55.00	55.00
NUMBER VOLUME (ft3 OR	1	1	2	m	н	-	58		=	-	-	8	.	0	Ħ
CONTAINER TYPE	5 Gallon Can	Bottle	na C	High Pressure	Can	Other	Other	5 Gallon Can	Tank	Tank	Tank	Other	Tank	Drum: Bung Top	Drum: Bung Top
ROOM	2	2	∢	< -	<	<	m	<	4	E/0	0/8	E/0	s/o	<	∢
BUILDING #	410	410	412	412	412	412	412	413	413	413	413	414	414	417	417
CONTAINER I.D. #	CC-2270-0587	CC-2270-0587	CC-2175-0587	CC-2175-0587	CC-2175-0587	CC-2175-0587	CC-2174-0587	CC-2213-0587	CC-2213-0587	CC-2214-0587	CC-2215-0587	CC-2171-0587	CC-2172-0587	CC-2064-0587	CC-2065-0587

Comments	4 30-CALLON DRUMS OF METALUBE. ALL BADLY STAINED, ONE LEAKING.	55-GALLON DRUM OF UNKNOWN.	2-55 GAL, 1-30 GAL, "LUBRIPLATE" (?).	55-GALLON DRUM ON CART WITH LEAKING SPIGOT.	MOTOR OIL CONTAINER, BULGED, OPENED, SAME CONTENTS INSIDE.	55-GALLON WITH SPIGOT ON CART.	20/20 WEIGHT HOTOR OIL (TEXACO) WITH SPIGOT.	3 55-GALLON, ONE WITH SPIGOT, EMPTY.	HITH SPICOT ON CART.		MISC. CONTAINERS OF UNKNOWNS AND PAINT PRODUCTS.	INCLUDES: POLYCLAD, RUSTBOND PRIMER, CORROSIVE PROTECTIVE COATINGS, EPOXY, AND CATALYSIS.	"COOK'S COROVEL LATEX FINISH".	BOILED INEDIBLE LINSEED OIL.
PID READINGS	o	0	0	0	0	0	6	0	0		0	0	0	0
INTEGRITY	Poor	Fair	Falr	Poor	Poor	Pair	Fair	Fair	Falr	Pair		Fair	Fair	
COLOR										,				
PHASE	Liquid	Liquid	Liquid	Liquid	Liquid	Liquid	Liquid		Liquid	Liquid	Solid	Liquid	Liquid	Liquid
FUNCT I ONAL GROUP	Lubricant	Unknown	Lubricant	Solvent	Lubricant	Lubricant	Lubricant	Solvent	Solvent	Unknown	Paint	Paint	Paint	Solvent
VOLUME EST. (ft3 OR GAL.)	70.00	55.00	140.00	10.00	50.00	40.00	55.00	55.00	55.00	2.00	5.00	150.00	30.00	10.00
NUMBER VOLUME (ft3 OR	4	п	m				-	m	1	ო	290	09	6	4
CONTAINER	Drum: Bung Top	Drum: Bung Top	Drum: Bung Top	Drum: Bung Top	Drum: Bung Top	Drum: Bung Top	Drum: Bung Top	Drum: Bung Top	Drum: Bung Top	5 Gallon Can	Other	5 Gallon Can	5 Gallon Can	5 Gallon Can
ж ₩	∢	<	<	< -	<	<	∢	<	<	∢	æ	s a	æ	æ
BUILDING #	417	417	417	417	417	417	417	417	417	417	417	417	417	417
CONTAINER I.D. #	CC-2066-0587	CC-2067-0587	CC-2068-0587	CC-2069-0587	CC-2070-0587	CC-2071-0587	CC-2072-0587	CC-2073-0587	CC-2074-0587	CC-2075-0587	CC-2076-0587	CC-2077-0587	CC-2078-0587	CC-2079-0587

COMMENTS	SOLVENT.	SPRAY PAINT BOTILE, 12 02.	PAINT AND THINNERS, RUSTED.	55-GALLON. TOP RUSTED. POSSIBLY EMPTY. ONE OPEN, ONE CLOSED. FLASHPOINT 14C.	55-GALLON OF METHYL ISOBUTYL KETONE. DRUM CLOSED. FLASHPOINI = 15C.	6 SPRAY PAINT CANS, 4 1-GAL PAINT CANS, 5 1-PT PAINT REDUCER, 8 1-QT PAINT CANS.	"PHENOLINE THINNER",	1-QT CONTAINERS, "CATALYST".	1/2 GAL OF EPOXY COMPOUND.	55-CALLON CONTAINER CONTAINS SOLID MATERIAL RESEMBLING DRIED PAINT.	PAINT CONTAINERS.	PAINT THINNER.	MISC.: MATERIALS INCLUDING BENZENE, ACIDS, AND CATALYSTS.	55-GALLONS, TRASH.	PRIMER WITH PRESSURE APPLICATOR.
PID READINGS	0	0	0	120	1600	0	0	0	0		0	0	0	0	0
INTEGRITY		Falt	Poor	Poor	Fair	E E	Falr	Fair	Fair	Falr	Fair	Fair		Fair	Good
COLOR				Clear	Clear					Green					
PHASE	Liquid	Gas	Liquid	Liquid	Liquid	Liquid	Liquid	Liquid	Liquid	Solld	Liquid	Liquid	Liquid	Sol 1d	Liquid
FUNCTIONAL GROUP	Solvent	Se S	Solvent	Solvent	Solvent	Paint	Solvent	Solvent	Paint	Paint	Paint	Solvent	Solvent	Trash	Paint
NUMBER VOLUME EST. (ft3 OR GAL.)	1.00	0.00	50.00	55.00	55.00	5.00	2.00	1.00	2.00	1.00	10.00	20.00	5.00	3.50	00.00
NUMBER	-	1	85	8	-	23	4	m	50	ı	15	'n	15	п	
CONTAINER IYPE	1 Gallon Can	High Pressure	1 Gallon Can	Drum: Bung Top	Drum: Bung Top	Other	1 Gallon Can	Other	Other	Drum: Open Top	1 Gallon Can	5 Gallon Can	Bottle	Drum: Open Top	5 Gallon Can
₩ ₩	æ	æ	æ	æ	#4	æ	ø	μ	£	æ	ρQ	ø	æ	æ	ø
BUILDING #	417	417	417	417	417	417	417	417	417	417	417	417	417	417	417
CONTAINER I.D. #	CC-2079-0587	CC-2079-0587	CC-2080-0587	CC-2081-0587	CC-2082-0587	CC-2083-0587	CC-2084-0587	CC-2084-0587	CC-2084-0587	CC-2085-0587	CC-2086-0587	CC-2086-0587	CC-2086-0587	CC-2087-0587	CC-2087-0587

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	COMMENTS	1 POSSIBLY EMPTY. DRUM SAMPLED WAS HALF FULL. FLASHPOINT > 110C.	55-GALLON DRUM WITH SPIGOT, RUSTED.	TAR	SOLVENT.	MISC, PAINTS.	40-GALLON DRUMS.		LUBE GUNS.	OIL CANS.	1-GALLON LUBE GUNS.	5-GALLON FUEL CAN.	5-GALLON METAL CONTAINERS OF LUBRICANT.	5-CALLON CONTAINERS OF HOT DIE LUBE BY FISKE.	"SUNISU".	LABELED "MOTOR OIL" AND "METHYLENE GLYCOL". DRUM HAS SPIGOT.	UNKNOWN CONTENTS. FLASHPOINT = 12C.
	PID READINGS	0	0	•	0	0	0	0	0	0	0		0	0	0	0	540
	INTEGRITY	Falr		Falr	Falr	Falr	Fair	Fair	Fair	Falr	में स	ो स स स	Fair	Fair	Fair	Falr	Fair
	COLOR	Clear															Brown
	PHASE	Liquid	Liquid	Liquid	Liquid	Liquid	Liquid	Liquid	Liquid	Liquid	Liquid	Liquid	Liquid	Liquid	Liquid	Liquid	Liquid
•	FUNCTIONAL GROUP	Solvent	Solvent	Tar	Solvent	Paint	Solvent	Solvent	Lubricant	Lubricant	Lubricant	Solvent	Lubricant	Lubricant	Unknown	Lubricant	Solvent
	NUMBER VOLUME EST. (ft3 OR GAL.)	80.00	50.00	5.00	5.00	40.00	150.00	5.00	2.00	2.00	5.00	3.00	50.00	10.00	20.00	55.00	55.00
	NUMBER (ю	1	1	1	20	v	'n	'n	æ	7		14	8	'n	#1	н
	CONTAINER TYPE	Drum: Bung Top	Drum: Bung Top	5 Gallon Can	5 Gallon Can	1 Gallon Can	Drum: Open Top	5 Gallon Can	Other	Other	Other	5 Gallon Can		5 Gallon Can	5 Gallon Can	Drum: Bung Top	Drum: Bung Top
	ROOM	υ	O	U	υ	U	υ	υ	U	υ	υ	υ	ပ	υ	U	υ	υ
	BUILDING #	417	417	417	417	417	417	417	417	417	417	417	417	417	417	417	417
	CONTAINER I.D. #	CC-2088-0587	CC-2089-0587	CC-2090-0587	CC-2090-0587	CC-2091-0587	CC-2092-0587	CC-2092-0587	CC-2092-0587	CC-2092-0587	CC-2092-0587	CC-2092-0587	CC~2093-0587	CC-2093-0587	CC-2093-0587	CC-2094-0587	CC-2095-0587

COMMENTS	2.5-GALLON FUEL CAN.	2-55 GALLON DRUMS, 1 WITH MODIFIED TOP, BOTH PROBABLY EMPTY.	55-GALLON DRUMS WITH MODIFIED TOPS, PROBABLY EMPTY.	3 CART BATTERIES.	4 5-GALLON GASOLINE TANKS ON VEHICLE.	4 40-GALLON GASOLINE TANKS. 2 ON EACH TRUCK. PROBABLY EMPTY.	2 20-GALLON GASOLINE TANK ON FORKLIFT. PROBABLY EMPTY.	2 30-GALLON GAS TANKS ON VEHICLE. PROBABLY EMPTY.	55-CALLON DRUMS: ONE HODIFIED, ONE BULGED ON THE BOTTOM.	RUSTED WITH SPIGOT.	RUSTED, DENTED 55-GALLON DRUM	CHEMICAL HAULER/TANKER, 1500 GALLON CAPACITY.	10-GALLON GAS CONTAINER ON CEMENT MIXER.	BATTERY	EMPTY AIR COMPRESSOR IANK.
PID READINGS	0	0	o	0	0	0	0	0	· •	0	0	o	o	0	0
INTEGRITY	Falr			म् इ	Fair	Falt	स व ग	Fair	Poor	Poor	Poor			Fair	Fair
COLOR															
PHASE	Liquid			Solid	Liquid	Liquid	Liquid	Liquid	Liquid	Liquid	Liquid			Solid	Sa s
FUNCTIONAL		Емрtу		Battery	Solvent	Solvent	Solvent	Solvent	Unknown	Solvent	Unknown	Empty	Empty	Battery	Empty
NUMBER VOLUME EST. (ft3 OR GAL.)	2.00	0.00	0.00	0.00	20.00	0.00	0.00	0.00	55.00	20.00	0.00	00.0	0.00	00.00	00.00
NUMBER	7	8	8	E	4	4	8	7	8	1	1	T.	F		1
CONTAINER TYPE	Other	Drum: Bung Top	Drum: Bung Top	Other	Other	Other	Other	Other	Drum: Open Top	Drum: Bung Top	Drum: Bung Top	Other	Other	Other	Other
ROOM ₩	υ	E/0	E/0	Ε/0	E/0	E/0	Ε/0	E/0	E/0	E/0	w	ĸ	w	0/s	s/o
BUILDING #	417	417	417	417	417	417	417	417	417 .	417	417	417	417	417	417
CONTAINER I.D. #	CC-2097-0587	CC-2053-0587	CC-2063-0587	CC-2148-0587	CC-2148-0587	CC-2148-0587	CC-2149-0587	CC-2150-0587	CC-2150-0587	CC-2150-0587	CC-2155-0587	CC-2155-0587	CC-2155-0587	CC-2151-0587	CC-2151-0587

COMMENTS	10-GALLON GAS TANKS. PROBABLY EMPTY.	4 BATTERIES ON PALLET.	BATTERY ON FORKLIFT.	10-CALLON GAS TANKS ON FORKLIFTS.	5 55-GALLON DRUMS ARE EMPTY, OTHER ARE UNKNOWN, DRUM SAMPLED HAD A 2-PHASE LIQUID, LABELED TRIBUTYL PHOSPHATE. FLASHPOINT > 110C.	BATTERIES STACKED ON PALLETS.	2 PROPANE PRESSURE TANKS. EMPTY, VALVES OPEN ON BOTTOM. 29,950 GALLON EACH. 50'H X 10'D.	БНРТҮ.	CONTENTS UNKNOWN.	4'D X 5'H.	16 OZ PROPANE CAN.	55-GALLON, EMPTY AND RUSTED.	16-OUNCE, HALF FULL OF BLACK LIQUID.	MODIFED EMPTY 55-GALLON DRUMS LABELED "WASTE OIL".
PID READINGS	0	0	0	0	21	0	c	0		0	0		0	o
INTEGRITY	Fair	Fair	Fair	Falr	Poor	Falr	Falr				다 다 다 다 다	Poor	Good	स ब भ
COLOR					Brown	1	1				; 1 1 1 1 1		Black	
PHASE	Liquid	Solid	Solid	Liquid	Liquid	Solid			Liquid		Gas		Liquid	Liquid
FUNCTIONAL GROUP	Solvent	Battery	Battery	Solvent	Unknown	Battery	Empty	Empty	Unknown	Empty	Gas	Empty	Unknown	Lubricant
NUMBER VOLUME EST. (ft3 OR GAL.)	00.00	00.0	00.0	00.0	100.00	6.00	0.00	00.0	1.00	62.80	0.00	00.00	0.25	100.00
NUMBER	2	4	1	7	12	80	8	8	ᆏ	H	.	1	-	22
CONTAINER TYPE	Other	Other	Other	Other	Drum: Bung Top	Other	Other	5 Gallon Can	1 Gallon Can	Tank	Gas Cylinder	Drum: Bung Top	Tail Tail	Drum: Open Top
ROOM	0/8	8/0	0/8	0/8	o/s	0/s	s/0					E/0	E/0	<
BUILDING #	417	417	417	417	417	417	428	431	431	431	431	432	432	433
CONTAINER I.D. #	CC-2151-0587	CC-2152-0587	CC-2152-0587	CC-2152-0587	CC-2153-0587	CC-2154-0587	CC-2182-0587	CC-22	CC-2283-0587	CC-2283-0587	CC-2283-0587	CC-2163-0587	CC-2163-0587	CC-2048-0587

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Comments	10-GALLONS DRUMS OF POWDER.	20-GALLON DRUM OF POWDER.	MSA CHEM-OX BREATHING APPARATUS.	BATTERY	OXYGEN-CENERATION CANNISTERS FOR MSA RESPIRATORS,	OIL FILL CONTAINER.	1-GALLON PAINT CAN.	1-GALLON GASOLINE CAN.		ONE PAINTING MACHINE.	INSECTICIDE SPRAYER.			PAINTING MACHINE INCLUDING A PRESSURE TANK, GAS TANK AND PAINT CAN.	GAS TANKS ON VEHICLES. PROBABLY EMPTY.	BATIERY ON VEHICLE.
PID READINGS	0	0	0	.	0	0	0	o	o	0		0	0	0	0	0
INTEGRITY	Falr	Fair	Poor	मुख्या	Fair	Fair	Fair	Falr	Falr	Fair			Falr	Fair	Fair	Falr
COLOR	White	White														
PHASE	Solid	Solid	Solld	Solld	Sol1d	Liquid	Liquid	Liquid	Liquid	Solid					Liquid	
FUNCT IONAL GROUP	Unknown	Unknown		Battery		Lubricant	Paint	Solvent	Solvent	Paint	Empty	Емрту	Емрту	Empty	Solvent	Battery
VOLUME EST. (ft3 OR GAL.)	2.67	2.67	2.00	00.0	0.20	1.00	1.00	1.00	5.00	00.0	0.00	0.00	0.00	00.00	0.00	0.00
NUMBER VOLUME (ft3 OR	7		4	1	п	1	Ħ	-	#	-	н	₩	#	Ħ	m	1
CONTAINER IYPE	Drum: Open Top	Drum: Open Top	Other	Other	Вож	1 Gallon Can	1 Gallon Can	1 Gallon Can	5 Gallon Can	Other .	Other	5 Gallon Can	1 Gallon Can	Other	Other	Other
ROOM	≺	≺	∢	∢	<	<	<	<	<	<	<	∢	<	<	<	<
BUILDING #	433	433	433	433	433	433	433	433	433	433	433	433	433	433	433	433
CONTAINER I.D. #	CC-2049-0587	CC-2049-0587	CC-2050-0587	CC-2050-0587	CC-2050-0587	CC-2050-0587	CC-2050~0587	CC-2050-0587	CC-2050-0587	CC-2050-0587	CC-2050-0587	CC-2051-0587	CC-2051-0587	CC-2051-0587	CC-2052-0587	CC-2052-0587

COMMENTS	POSSIBLY DRY CHEMICAL FIRE EXTINGUISHER RECHARGE.	STEEL BUCKET FULL OF MISC. SMALL ITEMS.	4 MISC. CONTAINERS APPEAR EMPTY.	55-GALLONS OF HYDRAULIC FLUID.	LECITE BINDER.	EMPTY 30-GALLON DRUM.	BRAMBLCIDE,	OLD GAS CANS.	30-GALLON DRUMS LABELED "SORBEAD" BY MOBIL. FLASHPOINT > 110C.	25-GALLLON CARDBOARD CONTAINER CONTAINING POWDER.	10-GALLON PRESSURE VESSEL, POSSIBLY PAINT.		55-GALLON DRUM OF TRASH.	1-GALLON AND 5-GALLON GAS CAN.	BAITERY.	20-GALLON DRUM, MFG. B/B.
PID READINGS	0	0	6	0	0	0	0	0	1	0	0	0	o	0	0	0
INTEGRITY	Falr			Fair	Pair		Fair		Poor	Falr	년 연 년	Palr	Falr	Fatr	Fair	Fair
COLOR										Black						
PHASE	Solid			Liquid	Liquid		Liquid		Solid	Solid	Solid	Liquid	Solid	Liquid	Solld	Liquid
FUNCTIONAL GROUP	Unknown		Емрtу	Lubricant	Solvent	Empty	Unknown	Empty	Unknown	Unknown	Paint	Unknown	Trash	Solvent	Battery	Lubricant
NUMBER VOLUME EST. (ft3 OR GAL.)	0.00	0.50	0.00	55.00	5.00	0.00	0.13	0.00	28.07	3.34	1.34	50.00	7.35	5.00	0.00	20.00
NUMBER	m	Ħ	4		1	#1	Ħ	7	,	#4 ,		7	1	7	1	20
CONTAINER IYPE	Other	Other	Other	Drum: Bung Top	5 Gallon Can	Drum: Bung Top	Other	Other	Drum: Bung Top	Other	Other	Drum: Bung Top	Drum: Open Top	Other	Other	Other
ROOM	<	<	<	<	. ⋖	m	ø	K A	υ ·	Q	۵	Ω	Q	Q	۵	Q
BUILDING #	433	433	433	433	433	433	433	433	433		433	433	433	433	433	433
CONTAINER I.D. #	CC-2054-0587	CC-2055-0587	CC-2055-0587	CC-2056-0587	CC-2060-0587	CC-2055-0587	CC-2055-0587	CC-2055-0587	CC-2057-0587	CC-2058-0587	CC-2059-0587	CC-2060-0587	CC-2060-0587	CC-2060-0587	CC-2060-0587	CC-2060-0587

COMMENTS	9 MISC. CANS OF PAINT OR VARNISH, 1-QT.	TRACTOR FUEL TANK, CAPACITY UNKNOWN. PROBABLY EMPTY.	10-GALLON PRESSURE VESSEL.	10-GALLON FUEL CONTAINER.	EMPTY 5-GALLON CONTAINER.	EMPTY TRASH PAILS.	EMPTY 25-CALLON WATER CONTAINER.	THERMOS.	ON PALLET, 1/3 FULL. PCB ANALYSIS < 2 PPM. FLASHPOINT (2003A) = 37C. FLASHPOINT (2003AA) <0C.		HALF FULL OF CLEAR, LIGHT LIQUID, 3/4 FULL. PCB ANALYSIS = 28,000 PPM. FLASHPOINT < 0C.	DARK LIQUID, 3/4 AND 1/4 FULL DRUMS. 55-GALLON. PCB'S < 2 PPM. FLASHPOINT > 110G.	55-GALLON DRUM WITH DRIP PAN.	
PID READINGS	0	0	0	0	0	0	0	0	m		9	20	o	0
INTEGRITY	Fair	Falt	Fair	Falr	Fair	Falr		Fair	Falr	is T	Poog	F a r	म म	म क म
COLOR					1 1 1 1 1				Yellow		Yellow	Brown		
PHASE	Liquid		Liquid	Liquid	; ; ; ; ; ; ; ;				Liquid	Liquid	Liquid	Liquid	Liquid	Liquid
FUNCTIONAL	Paint	Empty	Unknown	Solvent	Empty	Empty	Empty	Empty	Unknovn	Unknown	Lubricant	Lubricant	Lubricant	Unknown
VOLUME EST. (ft3 OR GAL.)	1.00	00.00	10.00	10.00	0.00	0.00	00.0	0.00	15.00	55.00	45.00	55.00	55.00	1.00
NUMBER VOLUME (ft3 OR	6	Ħ	1	Ħ	F	8	Ħ	~	8	Ħ		8	Ħ	0
CONTAINER TYPE	Other	Other	Other	Other	5 Gallon Can	Other	Other	Other	Drum: Bung Top	Drum: Bung Top	Drum: Open Top	Drum: Open Top	Drum: Bung Top	1 Gallon Can
ROOM	۵	Ω	M	ы	м	<	<	<	<	<	. ◀	∢	<	∢
BUILDING #	433	433	433	433	433	435	435	435	435	435	435	435	435	435
CONTAINER I.D. #	CC-2060-0587	CC-2062-0587	CC-2061-0587	CC-2061-0587	CC-2061-0587	CC-2001-0587	CC-2001-0587	CC-2002-0587	CC-2003-0587	CC-2004-0587	CC-2006-0587	CC-2007-0587	CC-2008-0587	CC-2200-0587

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	Comments	SODIUM FLUORIDE. 55-GALLON. REACTIVE HATERIAL.	(2009A) HOUGHTON'S LIQUID HEAT. 55-GALLON DRUMS. POWDER WITH CRYSTALS. FLASHPOINT > 74G.	(2009B) BETZ WATER CONDITIONER. CONTAINS SOLUABLE CHROMATE SALTS. FLASHPOINT > 110G.	(2009C) POLYPHASE FIRESIDE CHEMICAL (RX613). FLASHPOINT > 110C.	55-GALLON CRYSTALLINE POWDER. FLASHPOINT > 110C.	50-POUNDS OF MET-L-X POWDER.	55-GALLON DRUMS.	70 POUND BAG OF LIGHT SODA ASH.	(2011B) 70-POUND BAGS OF PARAFORMALDEHYDE FLAKE. MADE BY CELANESE. FLASHPOINT = 80C.	(2011A) 100-POUND BAGS OF KENITE. FLASHPOINT > 110G.	(2011C) HYPLO-SUPERCEL. DIATOMACEOUS SILICA PRODUCT FROM JOHNS MANVILLE. FLASHPOINT > 110C.	EMPTY CRYOGENIC CONTAINERS.	EMPIY CRYOGENIC CONTAINERS.
	PID READINGS	o	0	0	o	0	0	6	0		o	o .	0	0
	INTEGRITY	Fair	Falt	Pair	Fair	T at	Fair	Fair	Poor	Fair	Falr	14 14 14 14 14 14 14 14 14 14 14 14 14 1	Fair	
	COLOR		White	Orange	Brown	Blue				White		White		
	PHASE	Solid	Solid	Solld	Solid	Solid	Solld	Solid	Solid	Solid	Solid	Solid		
may 1987	FUNCTIONAL GROUP	Unknown	Unknown	Unknovn	Unknown	Unknown	Unknown	Unknown	Base	Unknown	Unknown	Unknown		Empty
	VOLUME EST. (ft3 OR GAL.)	7.35	147.00	147.00	66.00	14.70	7.00	169.00	29.00	25.00	500.00	3.00	0.00	0.00
	NUMBER	-	20	20	٥	2	Ħ	23	29	25	500	m	6	6
i	CONTAINER TYPE	Fiberboard	Fiberboard	Fiberboard	Fiberboard	Drum: Open Top	Drum: Open Top	Drum: Open Top	ВяВ	80 81 82	88 88 80 88 88 88 88 88 88 88 88 88 88 88 88 8	ಕು ಜ ಛ	Other	Other
	ROOM #	m	æ	#A	m	æ	ø	æ	m	e a	æ	pi.	м	ы
	BUILDING #	435	435	435	435	435	435	435	435	435	435	435	435	435
	CONTAINER I.D. #	CC-2009-0587	CC-2009-0587	CC-2009-0587	CC-2009-0587	CC-2010-0587	CC-2010-0587	CC-2010-0587	CC-2011-0587	CC-2011-0587	CC-2011-0587	CC-2011-0587	CC-2012-0587	CC-2101-0587

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COMMENTS	REACTION VESSELS, 2 EMPTY, 1 SEALED.	PAINT CAN OF POWDER.	EMPTY 10-GALLON CONTAINER.	4 1/2" GAS CYLINDERS.	10-GALLON CONTAINERS OF KEROSENE.	30-GALLON TRASH CAN FILL WITH EMPTY BOITLES,	10-GALLON DRUM CONTAINING SMALL AMOUNT OF TRASH.	EMPTY CYROCENIC CONTAINERS.	55-GALLON DRUMS, OVER-PRESSURIZED, LEAKING AND OILY AROUND DRUM RIM. APPEARS TO BE LUBRICANI. FLASHPOINT > 110C.	55-GALLON DRUMS, TOP RUSTED OFF.	RUSTED. DRUM SAMPLED WAS 3/4 FULL. FLASHPOINT > 110C.	MODIFIED, 55-GALLON DRUM. RADIOACTIVE.	55-GALLON, MULTI-RIBBED AND RUSTED. FLASHPOINT = 24C.	
PID READINGS	0	0	0	0	0	0	o			0	100	o	11	0
INTEGRITY	Good	Falr		Fair	Fair	Er er Fr			Poor	Poor	Fair		Poor	
COLOR		White							Brown		Blue		Brown	
PHASE	Liquid	Solid		88 88	Liquid	Solld	Solid		Liquid		Liquid		Liquid	
FUNCTIONAL GROUP	Unknown	Paint	Empty	Gas	Solvent	Trash	Trash	Empty	Lubricant	Empty	Unknown	Empty	Solvent	Empty
VOLUME EST. (ft3 OR GAL.)	00.00	1.00	00.0	00.00	10.00	0.00	1.34	00.00	200.00	0.00	200.00	0.00	. 00.09	0.40
NUMBER VOLUME (ft3 OR	e			۲	8	H			'n	m	4	н	2	m
CONTAINER TYPE	Other	1 Gallon Can	Other	Gas Cylinder	Other	Other	Other	Other	Drum: Bung Top	Drum: Open Top	Drum: Bung Top	Drum: Bung Top	Drum: Bung Top	1 Gallon Can
ROOM	E/0	ĵs.,	S te	ဖ	O	ဖ	z	0/H	0/2	0/M	0/14	0/M	0/14	<
BUILDING #	435	435	435	435	435	435	435	435	435	435	435	435	435	436
CONTAINER I.D. #	CC-2017-0587	CC-2013-0587	CC-2013-0587	CC-2014-0587	CC-2015-0587	CC-2016-0587	CC-2001-0587	CC-2019-0587	CC-2020-0587	CC-2021-0587	CC-2022-0587	CC-2023-0587	CC-2024-0587	CC-2033-0587

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CONTAINER I.D. #	BUILDING #	ROOM	CONTAINER TYPE	NUMBER	NUMBER VOLUME EST. (ft3 OR GAL.)	FUNCTIONAL GROUP	PHASE	COLOR	INTEGRITY	PID READINGS	COMMENTS
CC-2033-0587	436	∢	Bottle	1	0.50	Acid	Liquid		Poog	0	1/2 GALLON BOTTLE OF SULFURIC ACID.
CC-2033-0587	436	<	Other	'n	1.00	Mixture	Liquid		Poor	0	ASSORIED BOTILES, CANS, BUCKEIS.
CC-2034-0587	436	v	5 Gallon Can	8	5.00	Tar	Liquid		Fair	0	PROTECTIVE COATING.
CC-2034-0587	436	υ	1 Gallon Can	7	0.27	Empty			Fair	0	
CC-2034-0587	436	U	Jar	∞	1.00	Solvent	Llquid		Poor	0	POSSIBLY GLUE,
CC-2034-0587	436	υ	Bottle	н	1.00	Lubricant	Liquid		Falr	0	TRANSHISSION FLUID.
CC-2034-0587	436	υ	Bottle	П	0.50	Solvent	Liquid		Fair	0	IURPENTINE.
CC-2025-0587	436	E/0	Other	п	0.00					0	TRANSFORMER.
CC-2026-0587	436	E/0	Other	0	0.00	Емрсу			Poor	0	EMPIY AND BROKEN GLASS CARBOYS.
CG-2027-0587		E/0	Drum: Open Top	19	100.00	Unknown	Llquld	Yellow	प्र व स		RADIOACTIVE, 12 APPEAR TO BE OVERPACK WITH STAINLESS STEEL CASKS IN THEM. BETA = 6,000 & 1100 CPM. GAPMA = 265K & 3K. KEG SAMPLED WAS 2/3 FULL.
CC-2028-0587	436	E/0	Other	,	00.00	Empty			Fair	0	EMPTY CYROGENIC CONTAINERS.
CC-2032-0587	436	0/N	Other	13	0.00	Battery	Solid		Fair	0	FORKLIFT BATTERY PACKS.
CC-2029-0587	436	0/1	Drum: Open Top	8	5.00	Unknown	Solid		Poor	0	55-GALLON DRUMS OF SOIL MATERIAL.
CC-2029-0587	436	0/8	Drum: Bung Top	7	5.00	Unknown	Solid		Poor	0	55-GALLON DRUMS WITH SANDY SOIL MATERIAL.
CC-2030-0587	436	0/8	Drum: Open Top	0 7	294.00	Unknown	Solid		Poor	0	DRUMS FILLED WITH 2" X 2" MAGNESIUM INGOTS. SEVERAL DRUMS NEAREST THE BUILDING CONTAIN TAR-LIKE MATERIAL.

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COMMENTS	HANY DRUMS RUSTED THROUGH. ALL DRUMS EHPTY EXCEPT FOR ONE. ONE DRUM APPEARS TO CONTAIN RAINWATER. FLASHPOINT (2041A £ 2041AA) > 110C.	OPENED WITH SURFACE RUSI.	55-GALLON.	EMPTY 55-GALLON,	OPEN, RUSTED 55-GALLON DRUM.	7 EMPTY, 2 FULL 55-CALLON DRUMS. 1 SAMPLED AND FOUND TO BE RADIOACTIVE.	PRESSURIZED CAN, SURFACE RUST.	GALLON OF NITRIC ACID.	40-GALLON DRUMS WITH SURFACE RUSI. FINE POWDER. FLASHPOINT > 110C.	"PROTECTITE", LID RUSTED THROUGH.	CANS LABELED "PROTEKTITE", MANUFACTURED BY THE STOVEY COMPANY.	SODIUM FLUORIDE.	RED-BROWN SOLID, CONTAINER RUPTURED, FLASHPOINT > 110C.	PAINT CONTAINERS.
PID READINGS	0	0	. 0	0	0	0	0	0	0	0	0	0	0	0
Integrity	Poor		Falr	Fair	Poor	Falt	Fair	Poog	Falr	Poor	Poor	Falr	Poor	Fair
COLOR	Clear						; ; ; ;	! ! ! ! !	White				Brown	
PHASE	Liquid		Liquid			Liquid	Solid	Liquid	Solld	Solld	Liquid	Solid	5011d	Liquid
FUNCTIONAL GROUP	Water	Empty	Unknown	Емрту	Empty	Unknown	Paint	Acid	Unknown	Tar	Tar Tar	Unknown	Unknown	Paint
NUMBER VOLUME EST. (ft3 OR GAL.)	20.00	00.00	55.00	7.35	00.00	110.00	0.67	1.00	80.20	2.67	20.00	12.00	12.00	2.00
NUMBER	18	и	7	1	#	σ.	#	1	15	4	4	7	7	8
CONTAINER TYPE	Drum: Open Top	Tank	Drum: Open Top	Drum: Open Top	Drum: Open Top	Drum: Bung Top	5 Gallon Gan	Bottle	Drum: Bung Top	5 Gallon Can	5 Gallon Can	Fiberboard	Fiberboard	1 Gallon Can
ROOM	0/m	0/1	0/1	0/11	0/11	N/0	0//	<	<	∢	∢	≺	<	≺
BUILDING #	436	436	436	436	436	436	436	437	. 438	438	438	438	438	438
CONTAINER I.D. #	CC-2041-0587	CC-2042-0587	CC-2043-0587	CC-2044-0587	CC-2045-0587	CC-2046-0587	CC-2047-0587	CC-2031-0587	CC-2035-0587	CC-2035-0587	CC-2036-0587	CC-2038-0587	CC-2039-0587	CC-2040-0587

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Comments	1QT PAINT CONTAINERS.	SOLVENTS		LABELED NITROGEN, 4.5'H RUSTED ROTTOMS		EMPIY AND RUSIED DRUMS: 2 55-GALLON, 3	40-CALLON.
PID READINGS	0	6		6		0	
INTEGRITY	Fair	Fair	1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1	Poor	***	Fair	
COLOR			1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1		1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1		
PHASE	Liquid	Liquid		Gas	: : : : : :		
EST. FUNCTIONAL	Paint	Solvent		Gas		Empty	
NUMBER VOLUME EST. FUNCTION (ft3 OR GAL.) GROUP	2.00	1.00	 	00.00		0.00	
NUMBER	18	m	 	13		٠	
CONTAINER TYPE	Other	CC-2040-0587 438 A 1 Gallon Can 3 1.00		CC-2173-0587 441 Gas Cylinder 13 0.00		Drum: Open Top	
ROOM	<	<		1		0	
BUILDING #	438	438		441		443	1
CONTAINER I.D. #	CC-2040-0587	CC-2040-0587		CC-2173-0587		CC-2180-0587	

ATTACHMENT 2

APPENDIX II Weldon Spring Site Remedial Action Project Chemical Plant Fire Extinguisher Inventory May 1987

																					; ; ; ; ; ; ; ; ; ; ; ; ;			
NUMBER	4	T	m	7	1	8	4	v	4	1	1	; ; ; ; ; ; ; ; ; ; ; ;	15	18	e	43	1	ı	1	1			1	၈
WEIGHT	15	15	15	15	15	15	15	15	15	15	15)	15	15	۲۰	15	150	10	15	15			15	10
TYPE	Dry Chemical, Red (AB or ABC)	Dry Chemical, Yellow (for metal fires)	Carbon Dioxide	Carbon Dloxide	Carbon Dioxide		Dry Chemical, Red (AB or ABC)	Dry Chemical, Yellow (for metal fires)	Carbon Dloxide	Carbon Dioxide	Carbon Dioxide	Carbon Dloxide	Carbon Dioxide	Carbon Dioxide	Drv Chemical. Red (AB or ABC)		Carbon Dloxide	Carbon Dloxide						
ROOM					٧	α	U						Д	£Q.	s a	Ω	æ.	WEST	EAST		SOUTH OUTSIDE		٧	EAST OUTSIDE
FLOOR	1ST	15T	IST	2ND	ZND	2ND	2ND	3RD	4TH	STH	6ТН								TOP	WEST STAIRS				
BUILDING	101	101	101	101	101	101	101	101	101	101	101		103	103	103	103	103	103	103	103	105	1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1	106	108

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				; ; ; ; ; ; ; ; ; ; ; ;										: ; ; ; ; ; ; ; ; ; ; ; ;									\			
NUMBER	v	52	2	1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1	ī	19	1	'n	Ħ	e	v	2	m		r	7	2			4	Ħ	m	1	2	E	2
WEIGHT	15	15	15	;	15	15	20	15	20	15	1.5	1.5	1.5		15	15	15		1.5	15	15	15		150	30	10
TYPE	Dry Chemical, Yellow (for metal fires)	Carbon Dloxide	Propellant Cartridge, Ansul		Dry Chemical, Yellow (for metal fires)	Carbon Dioxide	Dry Chemical, Yellow (for metal fires)	Carbon Dioxide	Dry Chemical, Yellow (for metal fires)	Carbon Dioxide	Carbon Dloxide	Carbon Dioxide	Carbon Dloxide		Dry Chemical, Yellow (for metal fires)	Garbon Dloxide	Carbon Dioxide		Garbon Dioxide	Garbon Dloxide	Carbon Dioxide	Carbon Dloxide		Carbon Dioxide .	Dry Chemical, Red (AB or ABC)	Carbon Dloxide
КООМ																			FEEDER HOUSE					EAST OUTSIDE	ROOM 3, SOUTH	ROOM E, SOUTH
FLOOR					151	1ST	2ND	2ND	3RD	3RD	4 T H	STH	6ТН			2ND .	ROOF			1ST	3RD	H17				
BUILDING	109	109	109		201	201	201	201	201	201	201	201	201		301	301	301		401	401	401	401		403	403	403

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BUILDING	FLOOR	ROOM	TYPE	WEIGHT	NUMBER
403	15T	x	Carbon Dloxide	15	
403	1ST	v	Carbon Dioxide	20	. 2
403	15T	۵	Carbon Dloxide	15	4
403	1ST	ROOM F	Dry Chemical, Red (AB or ABC)	30	4
403	2 1/2		Carbon Dioxide	15	H
403	2ND		Carbon Dioxide	15	-4
403	2ND	SOUTH	Dry Chemical, Red (AB or ABC)	30	m
403	3RD	SOUTH	Dry Chemical, Red (AB or ABC)	30	٣
403	OUTDOORS, 2 & 3		Other	20	4
				1	; ; ; ; ;
404		Y	Dry Chemical, Yellow (for metal fires)	15	ᆏ
404		Y	Carbon Dloxide	15	
707		Y	Other	15	2
404		٧	Dry Chemical, Red (AB or ABC)	10	
707		¥	Other		4
404		٧	Dry Chemical, Red (AB or ABG)	15	
707		٧	Dry Chemical, Yellow (for metal fires)	. 15	ᆏ
404		₹	Other		1
404		٧	Dry Chemical, Yellow (for metal fires)	15	7
404		V	Dry Chemical, Yellow (for metal fires)	15	2
404		A, W. CENTRAL AREA	Dry Chemical, Red (AB or ABC)	15	ᆏ
404		A, W. CENTRAL AREA	Dry Chemical, Yellow (for metal fires)	15	2
707		WEST	Dry Chemical, Yellow (for metal fires)	150	
404	2ND	NORTH	Dry Chemical, Yellow (for metal fires)	15	en
707	2ND	NORTH	Other	15	Ħ
707	2ND	WEST	Dry Chemical, Red (AB or ABC)	15	-

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BUILDING	FLOOR	ROOM	TYPE	WEIGHT	NUMBER
404	ZND	VEST	Dry Chemical, Yellow (for metal fires)	15	
3					
405		v	Dry Chemical, Red (AB or ABC)	15	2
405		٧	Dry Chemical, Yellow (for metal fires)	15	2
405		NORTH	Other	151	2
					1 d 1 B B B B B B B B B B B B B B B B B
904		٧	Dry Chemical, Red (AB or ABC)	350	T.
406		٧	Carbon Dioxide	15	-37
				· 8	1 1 4 5 5 7 7 7 8 6 8 8 8 8 8 1 1 1 1 1 1 1 1 1 1 1 1 1
407A		51	Dry Chemical, Yellow (for metal fires)	20	
407A	ROOF	EAST	Carbon Dloxide	1.5	3
407A	ROOF	WEST	Carbon Dloxide	15	1
4078		89	Carbon Dloxide	20	1
4078		75	Carbon Dloxide	15	1
4070		103	Carbon Dioxide	15	
408A		V	Carbon Dlowide		T.
408 A		æ	Carbon Dlox1de	15	e
408A		ы	Carbon Dloxide	15	F
408A		m	Carbon Dioxide	15	H
408C		£	Carbon Dioxide	15	2
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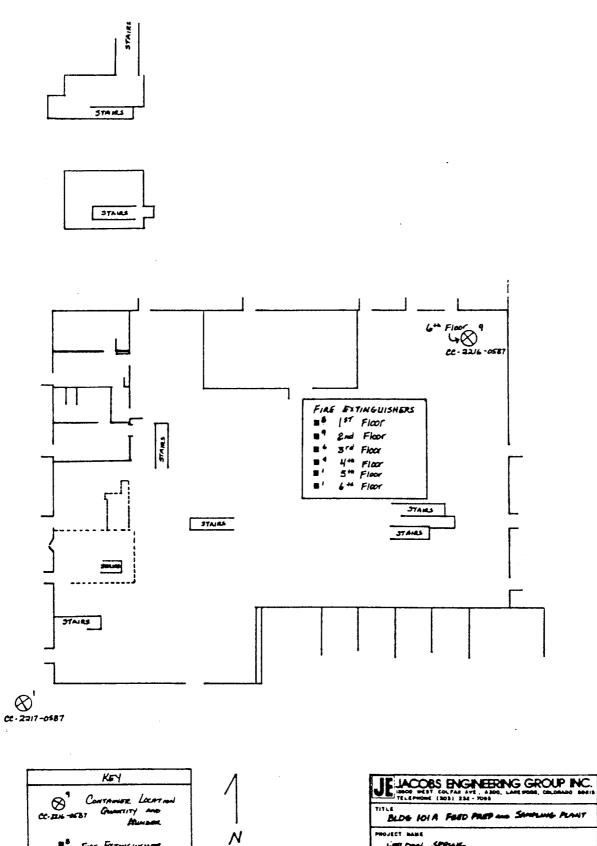
APPENDIX II Weldon Spring Site Remedial Action Project Chemical Plant Fire Extinguisher Inventory May 1987

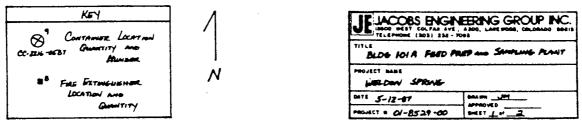
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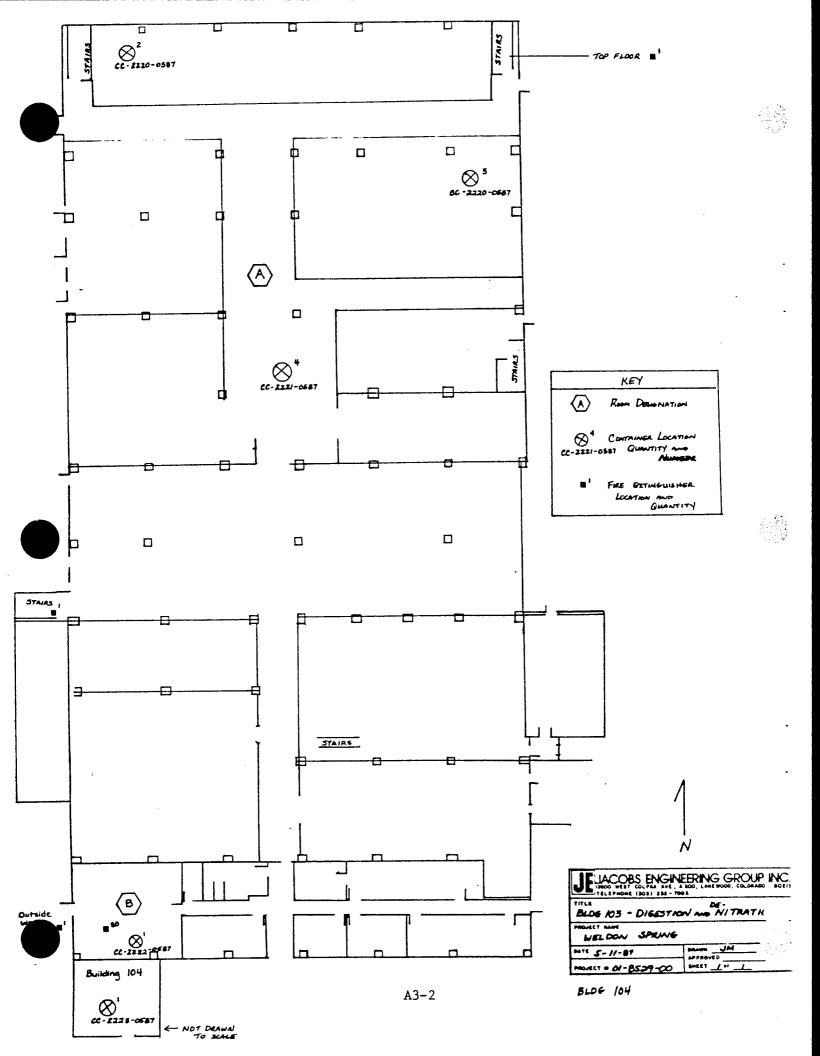
APPENDIX II Weldon Spring Site Remedial Action Project Chemical Plant Fire Extinguisher Inventory May 1987

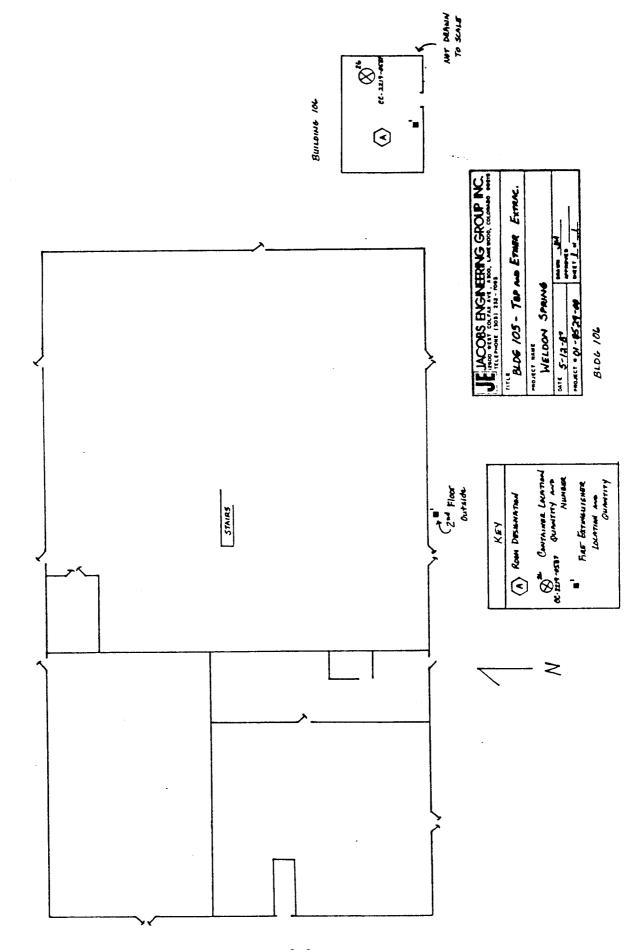
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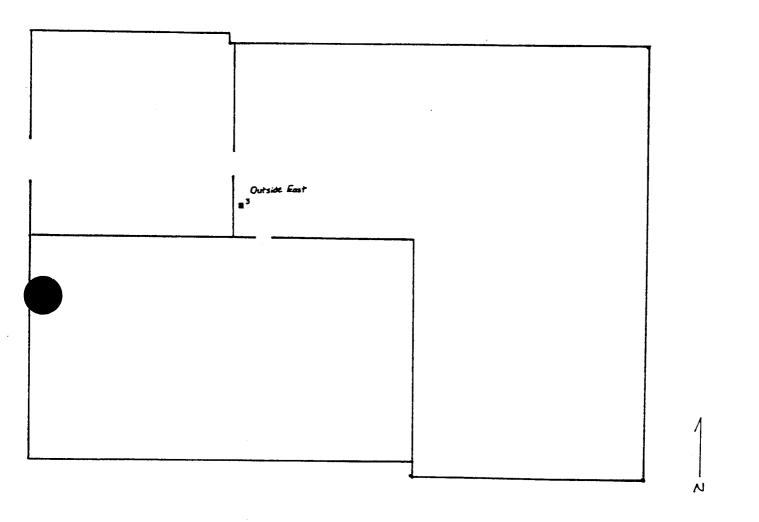
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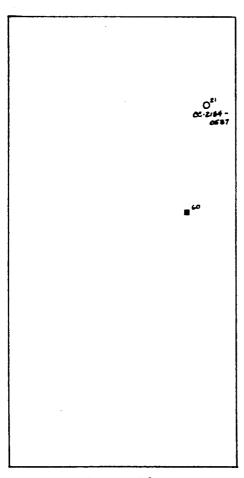
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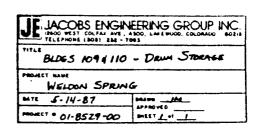
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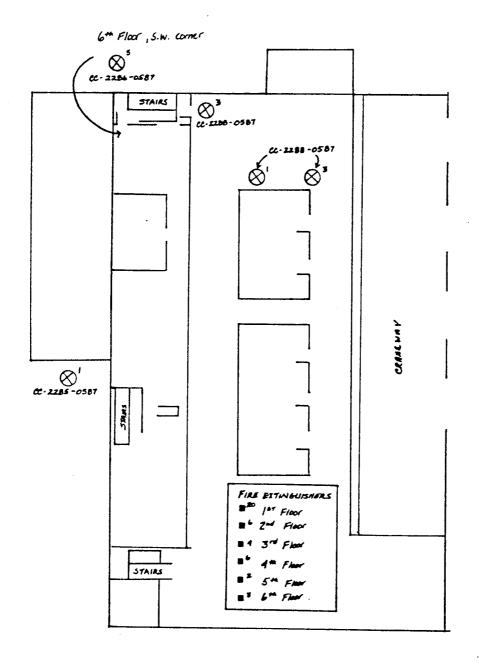
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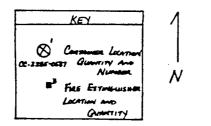
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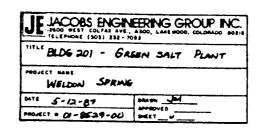
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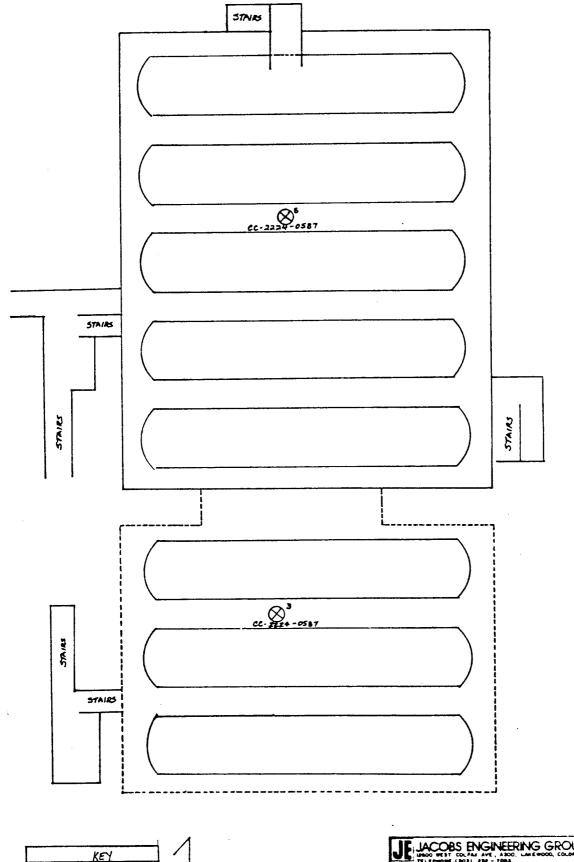
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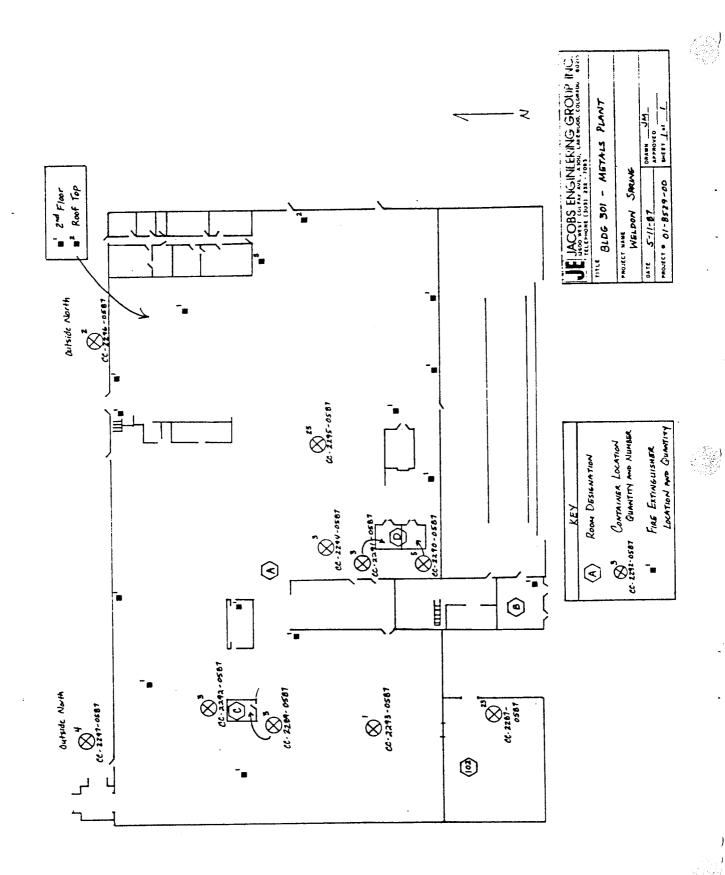
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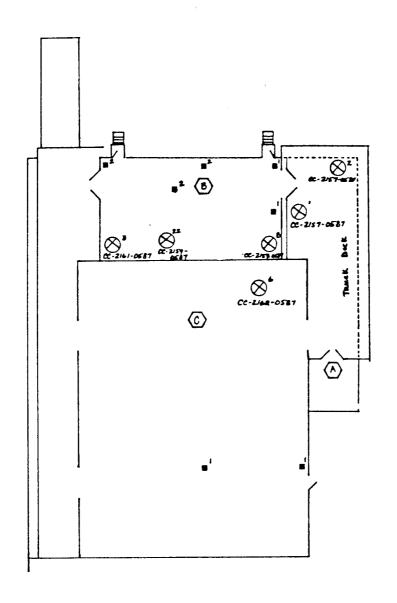
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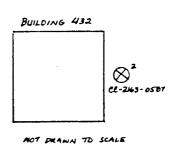


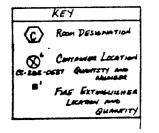
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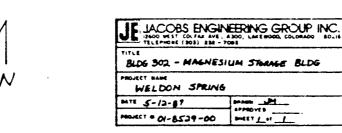
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5.	34	X	45	X	12
6.	18	X	7 .	X	12
7.	16	X	17	X	3
8.	21	X	13	X	5
9.	33	X	15	X	18
10.	17	X	15	X	18
11.	18	X	15	X	18
12.	15	X	8	X	12
13.	7	X	7	X	6
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15.	6	X	4	X	7
16.	12'	X	12'	X	6"
17.	12'	X	12'	X	6"
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19.	12'	X	12'	X	6"
20.	23	X	7	X	6
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27.	9	x	3	X	8
28.	85	x	38	X	10
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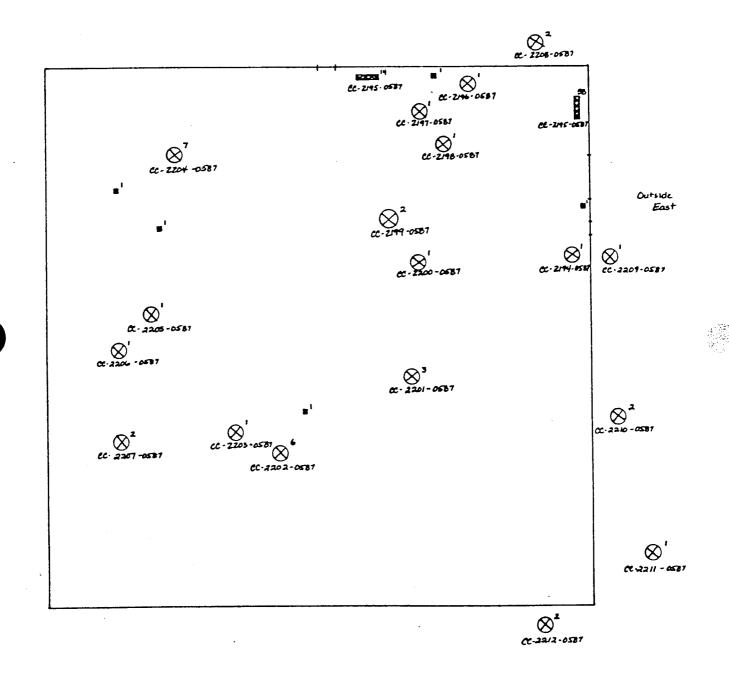




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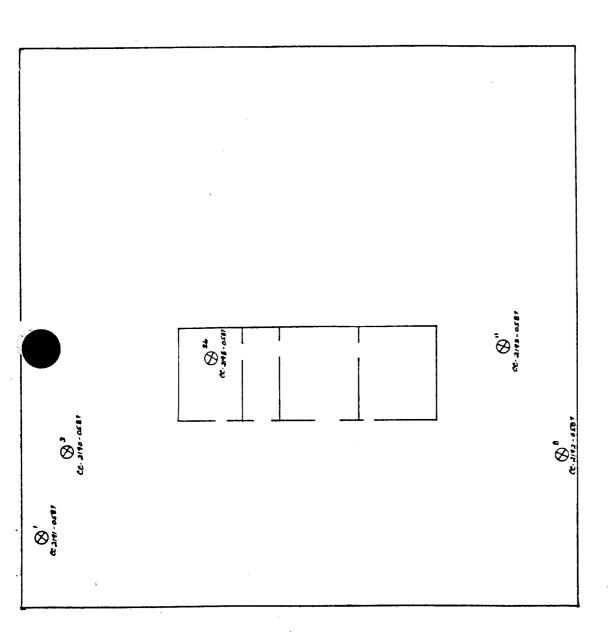
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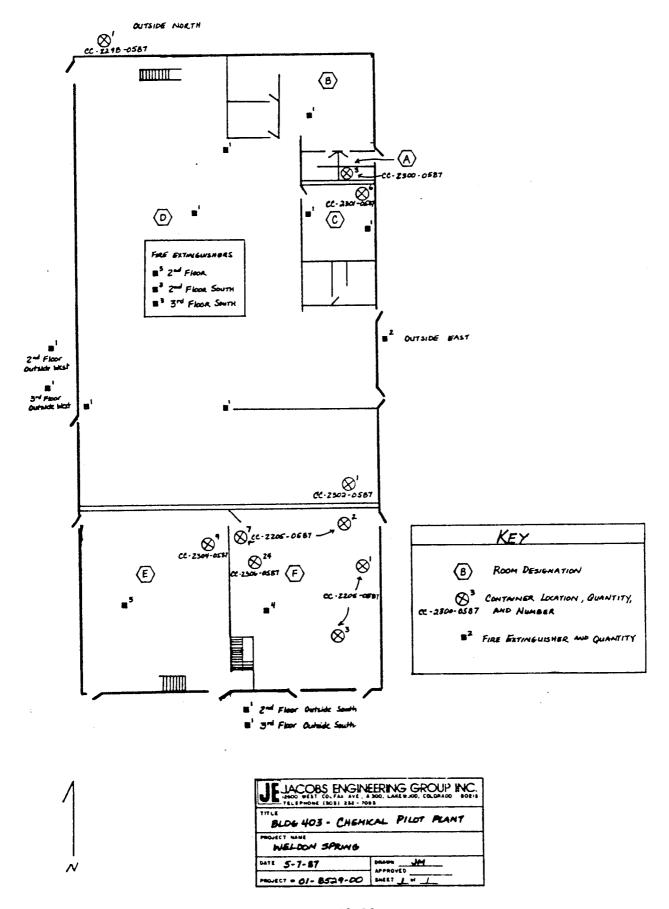
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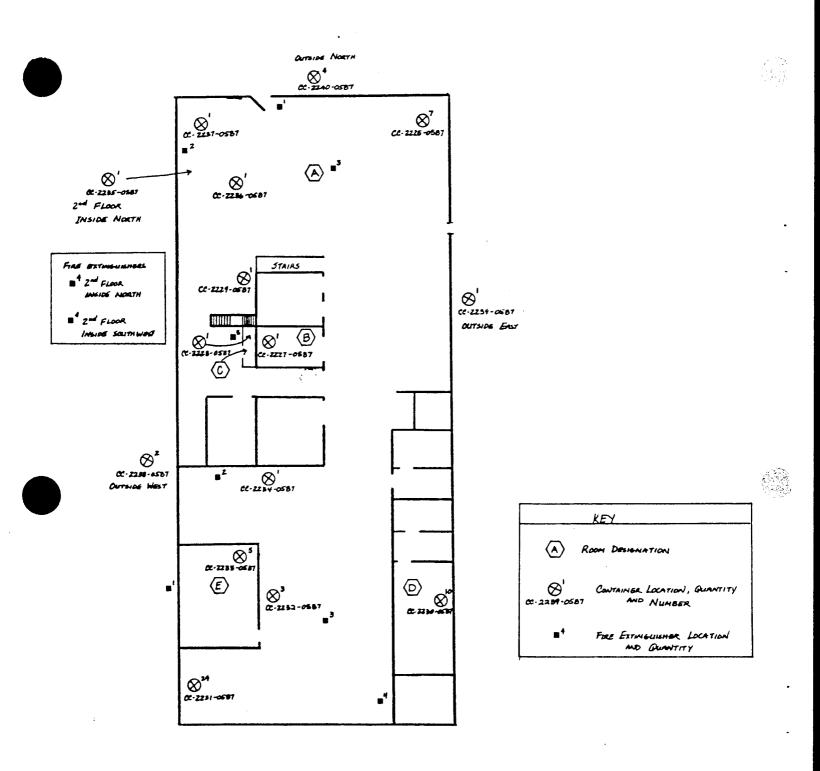
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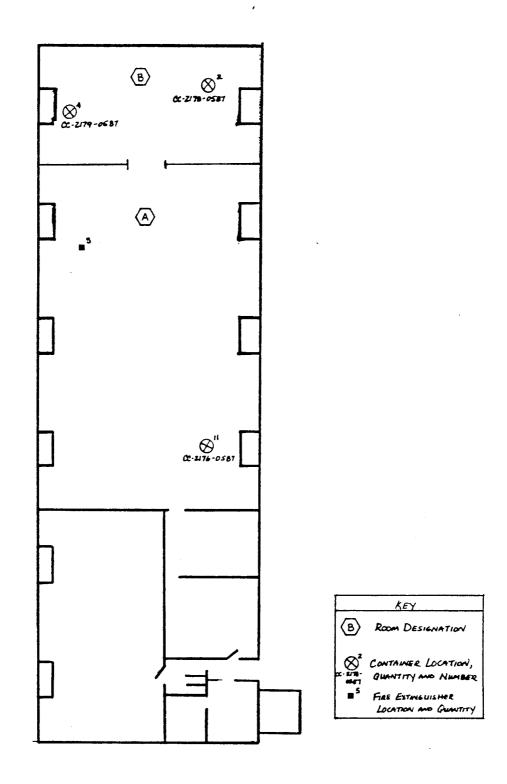
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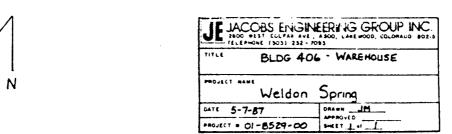
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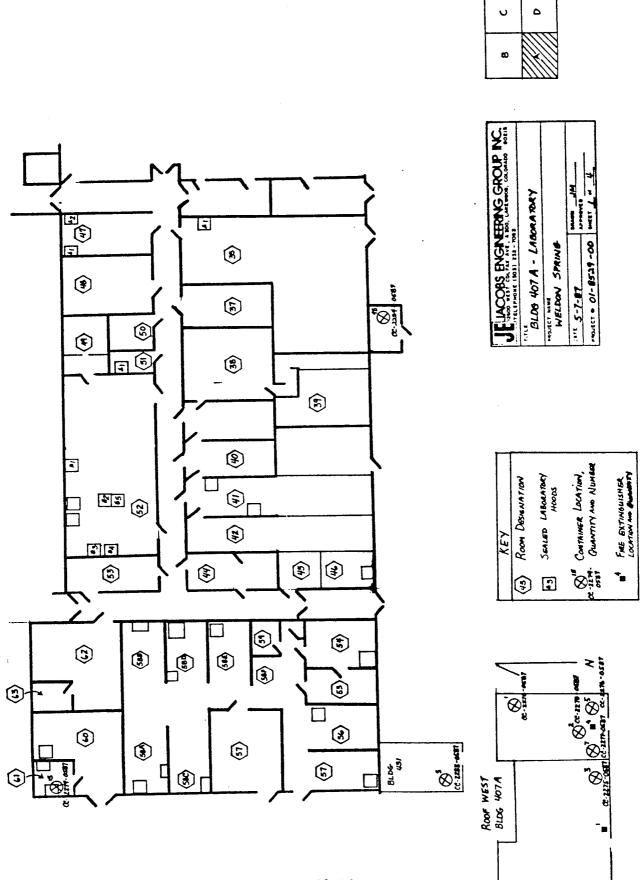
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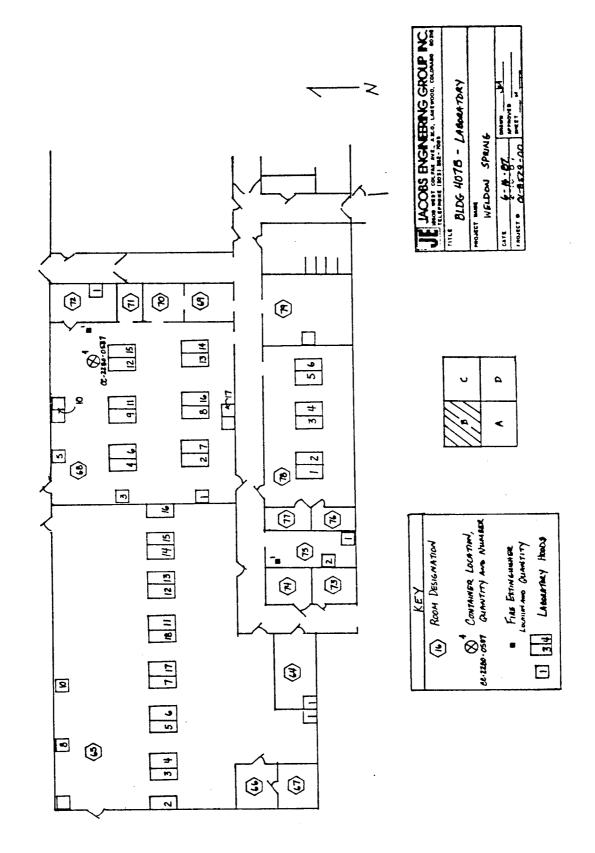
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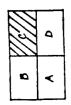
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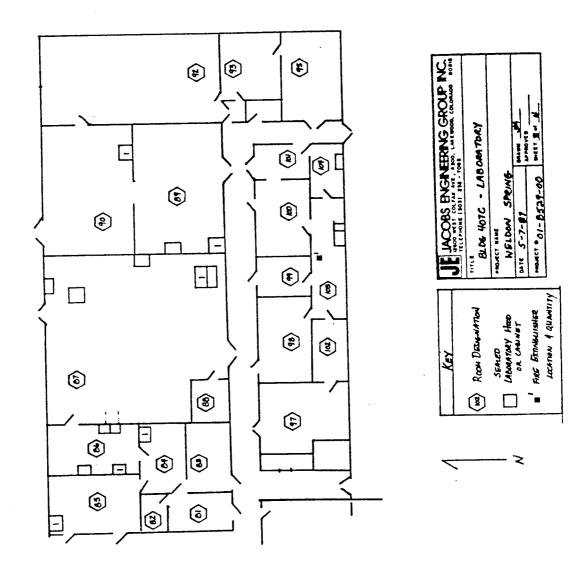


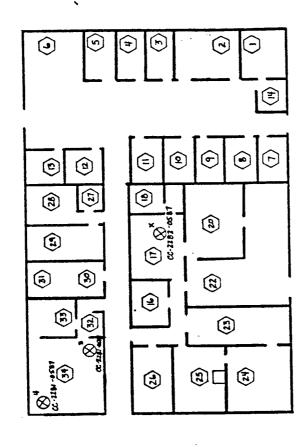


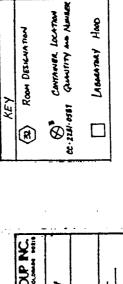


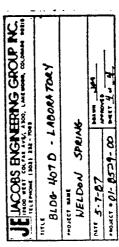


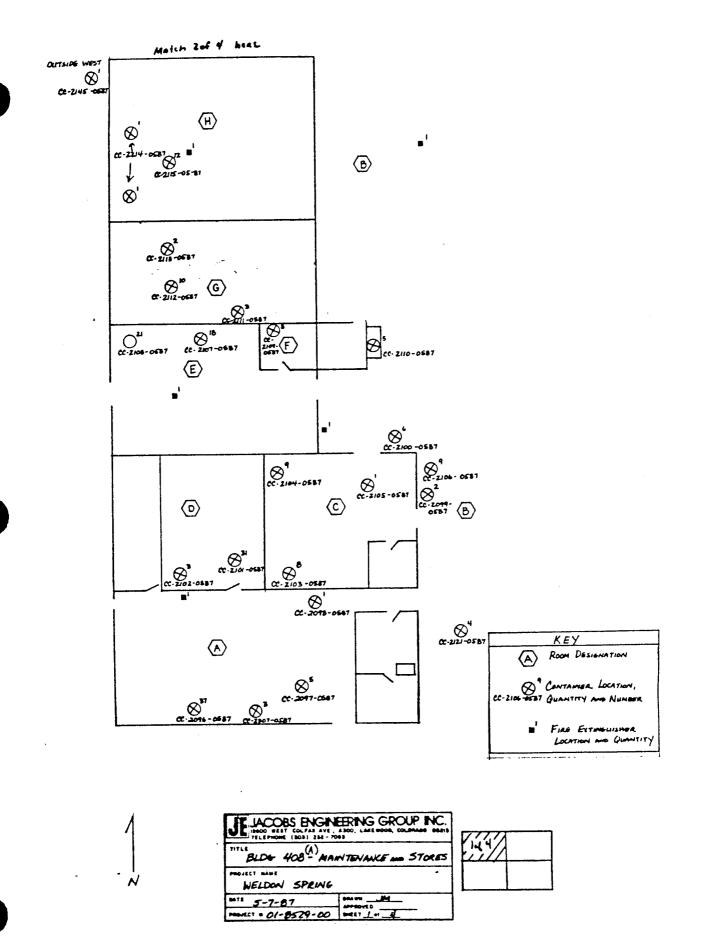


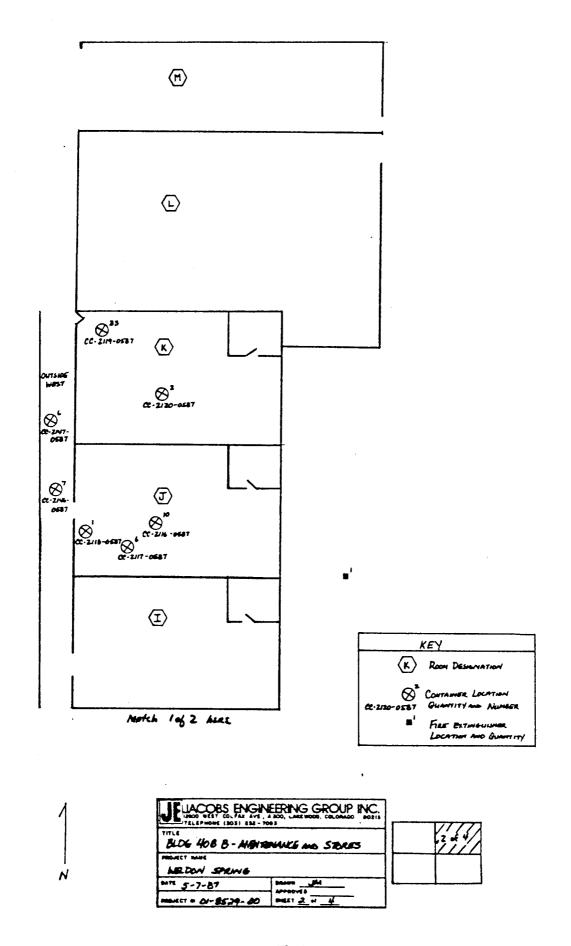


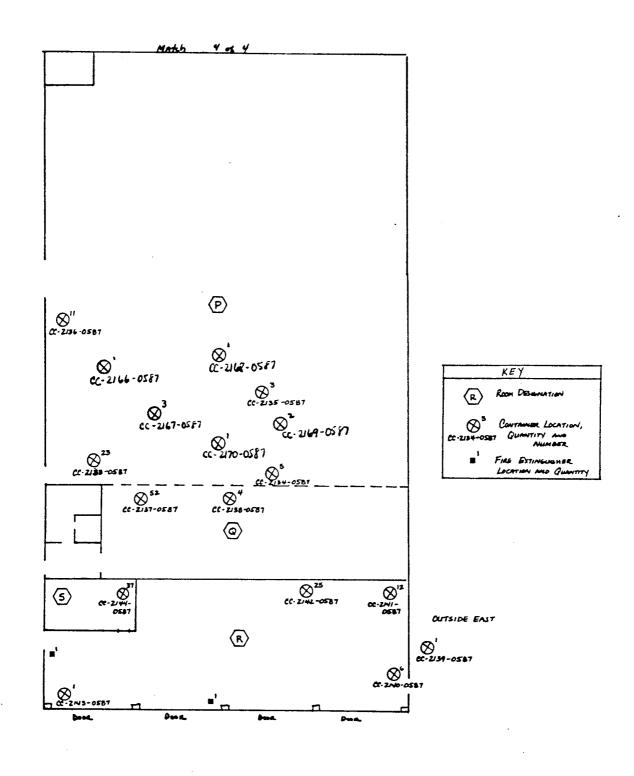












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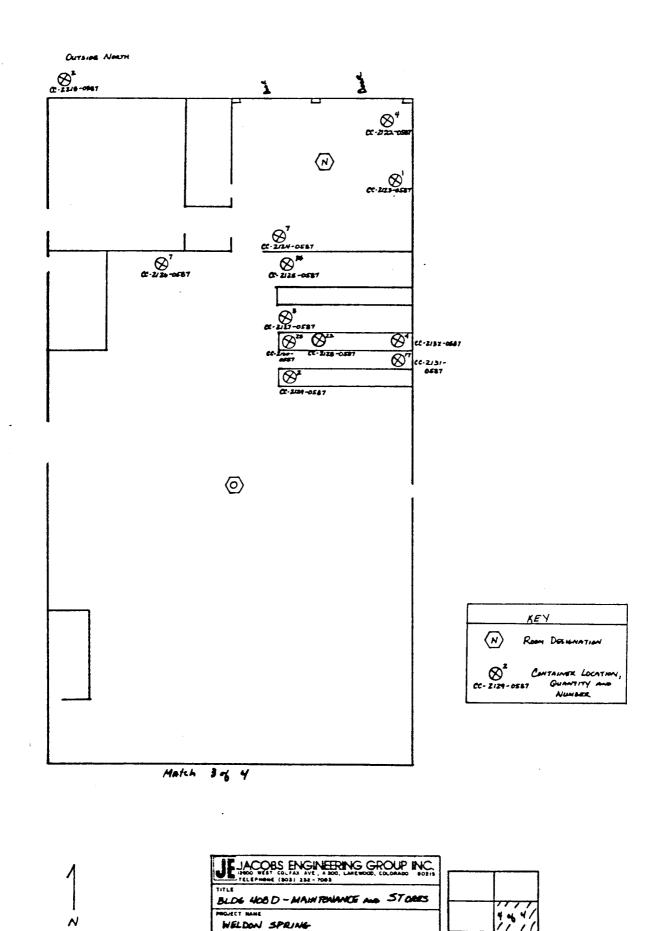
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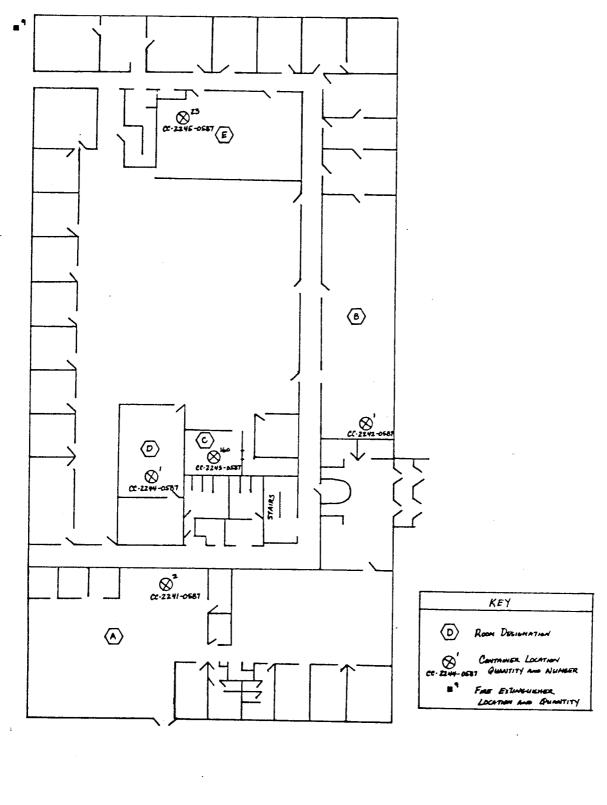
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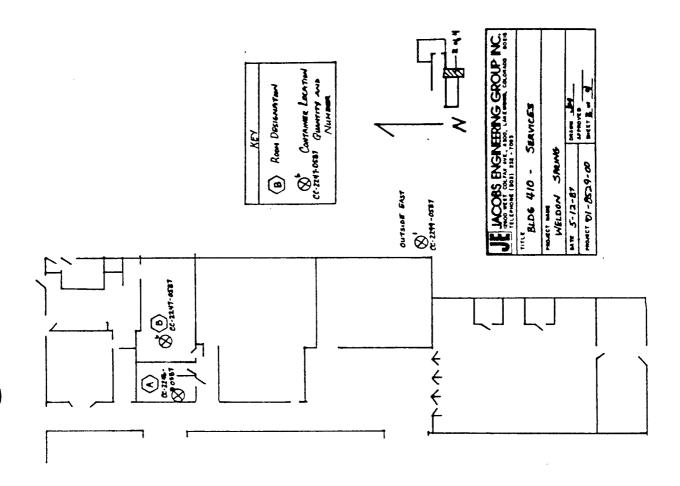
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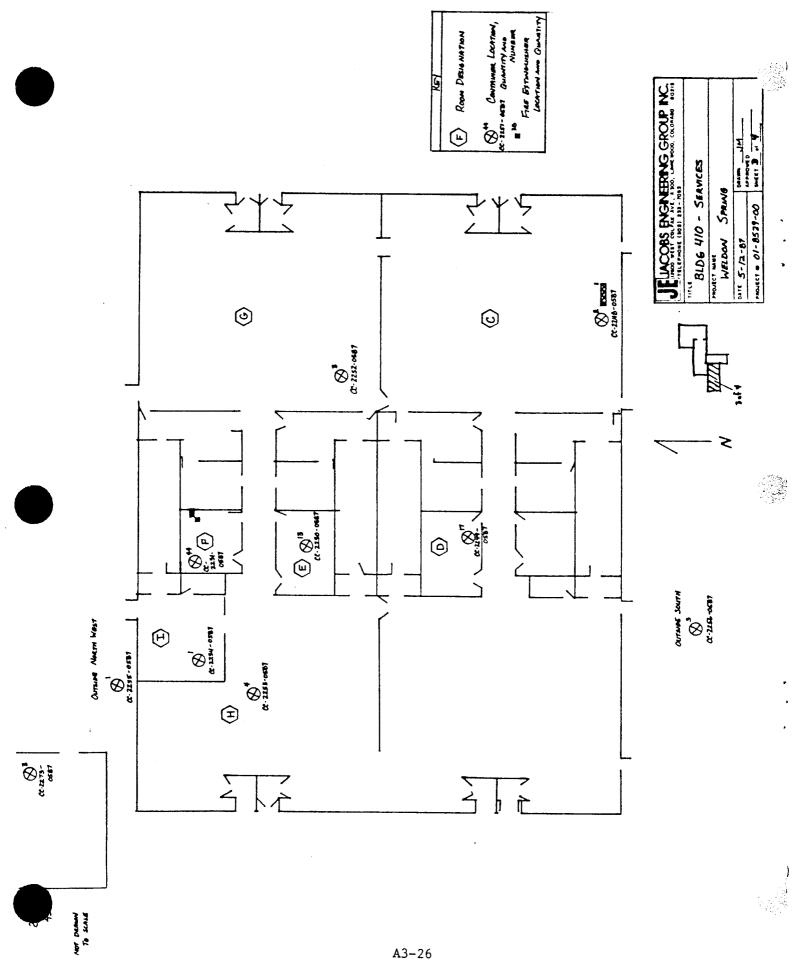
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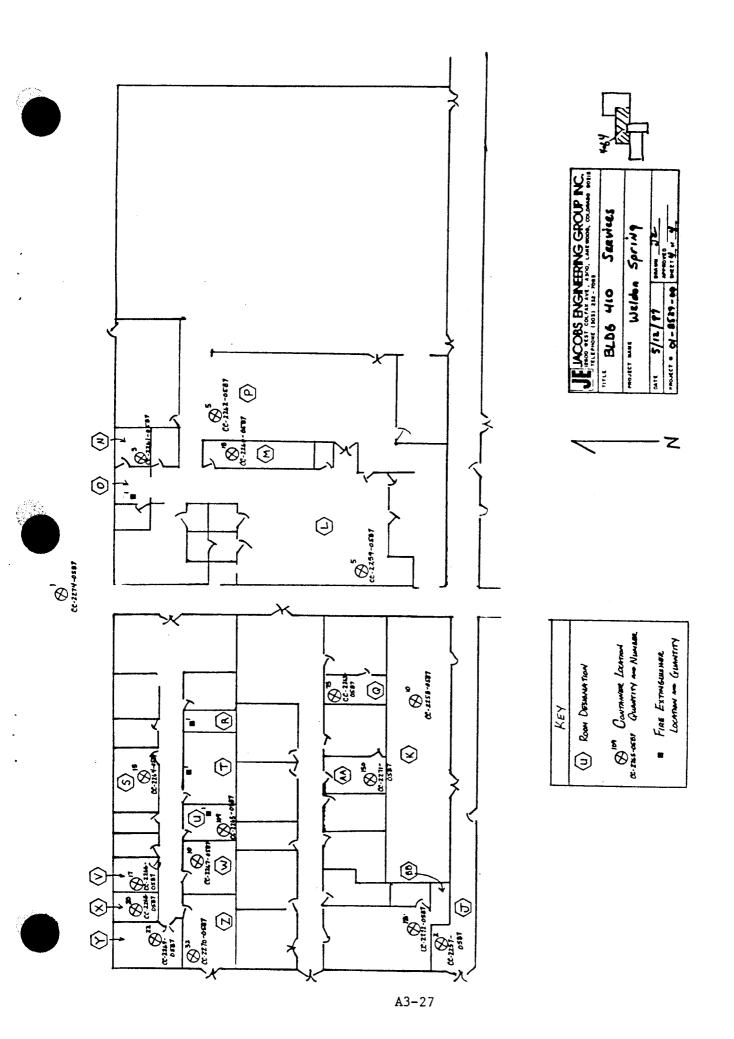


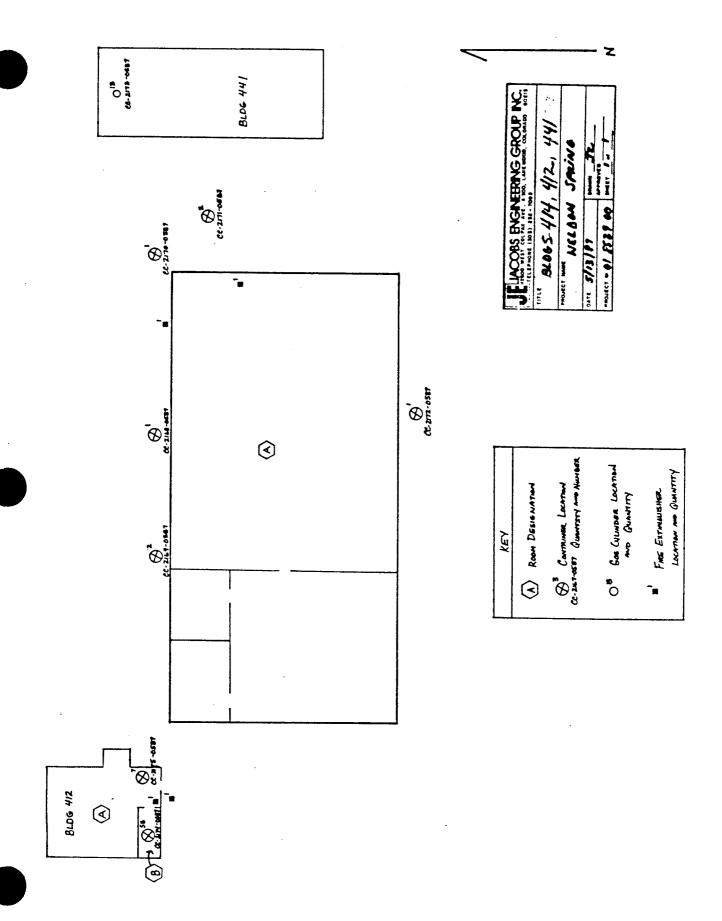
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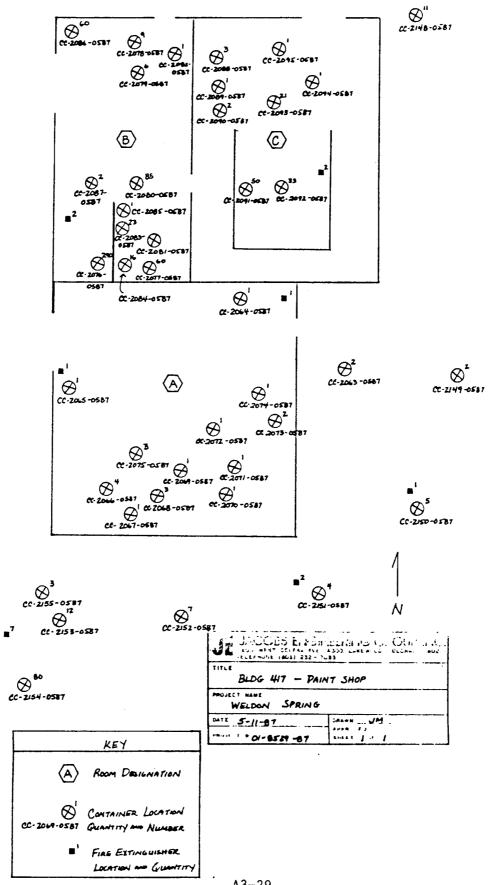
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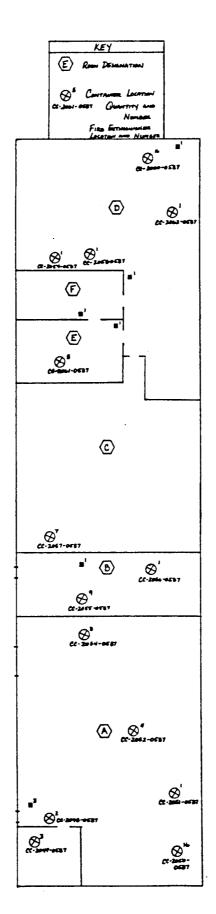






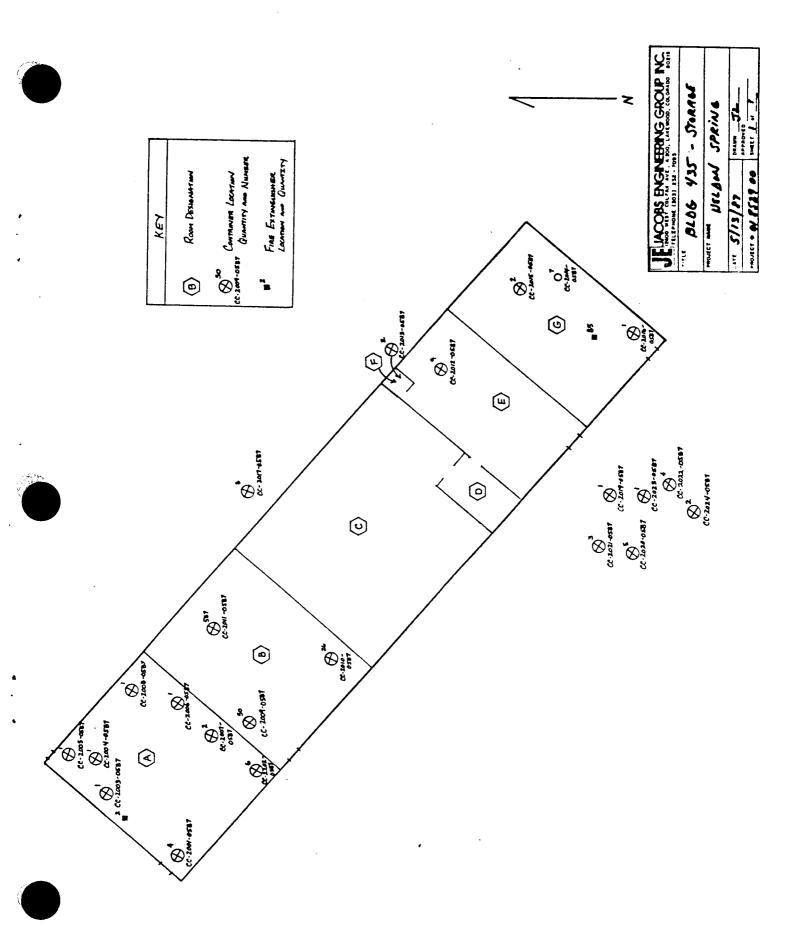


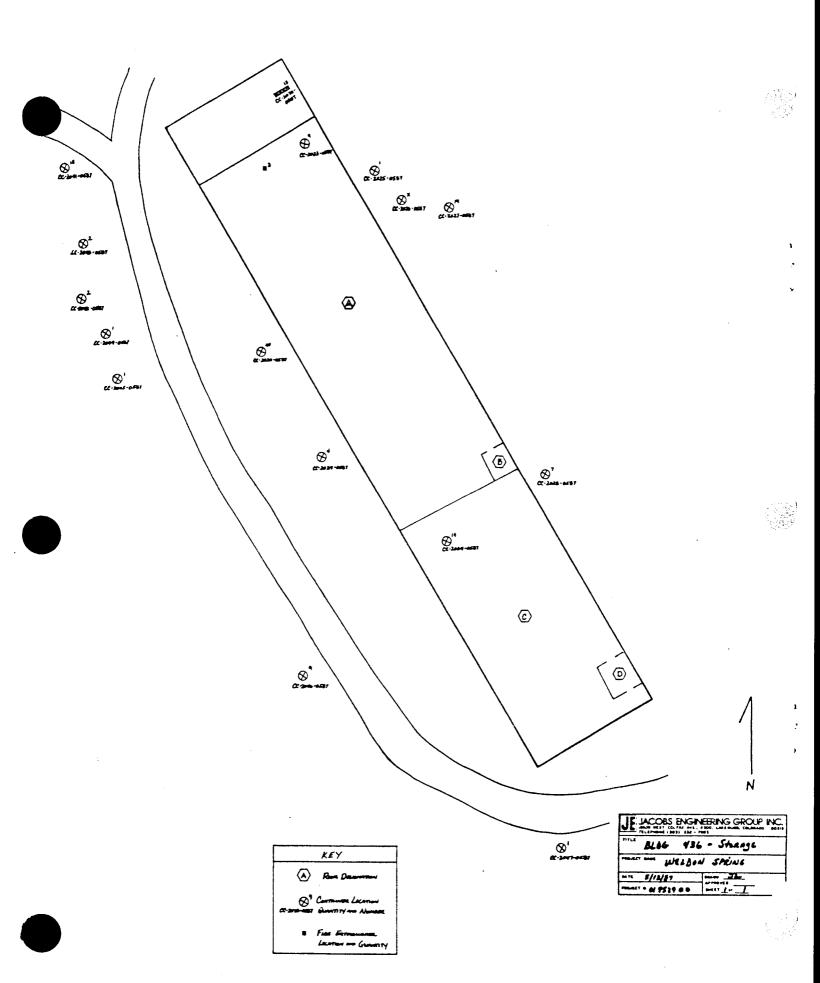
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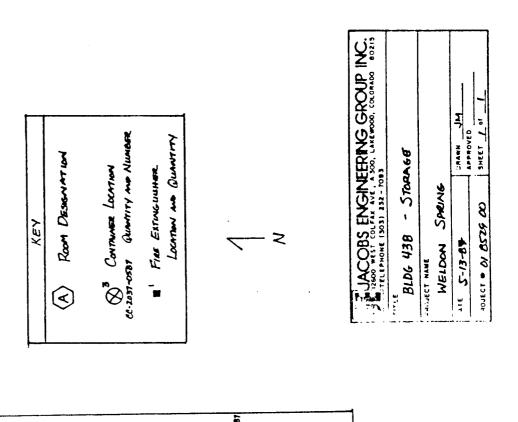


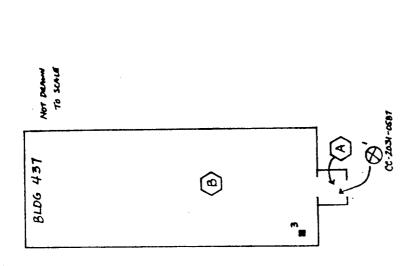
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TITLE BLOCK 438, 439 - 570 R.M.C.
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DATE 5-13-87

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SPECIAL CONDITIONS

MK-FERGUSON CONSTRUCTION SUBCONTRACTS

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SC-1 WORK TO BE ACCOMPLISHED

The work scope consists of furnishing all labor, supervision, equipment and materials (except as otherwise specified), and performing all work in strict accordance with Federal, State and Local laws, codes and regulations. The work in general consists of: Containerized waste Handling, Transfort, and

DISFOSAL

The above description of work is for general information only and in no way limits the responsibility of the Subcontractor for constructing the Work in strict accordance with the subcontract drawings and specifications.

SC-2 DEFINITIONS

- A. See GP-2 For General Definitions
- B. The term "Contractor's Inspector" shall mean the Duly Authorized Quality Assurance Representative of the Contractor.
- C. The term "Contractors Representative" shall mean the duly authorized "Construction Engineer" of the Contractor.

SC-3 LOCATION

The work to be accomplished is located at the abandoned Uranium Feed Materials Processing Plant at Weldon Spring, Missouri.

SC-4 HOLIDAYS

The following will be observed as holidays by the Contractor:

- * Christmas Day Columbus Day
- * New Year's Day Presidents' Day Memorial Day
- * Independence Day Labor Day Thanksgiving Day Day Following Thanksgiving
- * Holidays occurring on Saturday or Sunday will be on Friday or Monday.

Upon written notice from the Subcontractor 5 days in advance, the Contractor will adequately man the job on Contractor holidays not observed by the Subcontractor.

SC-5 SITE ACCESS

- A. Access to the site is via the main gate on Highway 94 South. The gate is normally open from 7 a.m. to 6 p.m. and the normal work day is from is 8 a.m. to 4:30 p.m., Monday thru Friday.
- B. Controls are established at the Weldon Spring site to govern personnel access. A guard station is located approximately 50 feet inside the main gate.
- C. All privately and Subcontractor owned vehicles shall stop at the guard station.
- D. The Subcontractor will have a specific identification symbol prominently displayed on each vehicle for entry to the site. A sample of this symbol will be given to MK-F prior to starting work.
- E. If no specific identification symbol is displayed on a vehicle, each person shall sign in and out with the guard before entry to the site when entering and leaving the site.

SC-6 TEMPORARY UTILITIES

A. Telephone Service

Telephone service will not be provided by the Contractor. The Subcontractor shall make his own arrangements for any telephone service desired.

B. <u>Electrical Power for Construction</u>

Electrical power for construction will not be provided by the Contractor. The Subcontractortor shall make his own arrangements for construction power.

C. Toilet and Washroom Facilities

Toilet and washroom facilities shall be provided by the Contractor. Emergency showers are available in Trailer 7, Full Trail France TRAILER ATTAL ROCESS CONTROL POINT.

D. Temporary Water Facilities

- 1. Drinking water shall be furnished by the Subcontractor.
- 2. If authorized by the Contractor, temporary supply lines, standpipes, and connections over and above those facilities presently existing and available shall be furnished and installed by the Subcontractor and shall be removed upon the completion of the need for such temporary facilities.

SC-7 <u>CONSTRUCTION RESTRAINTS</u>

- A. There are no known restraints at this time. The Subcontractor should be aware that more than one subcontractor may be working in the areas identified in the Work Scope. These areas are to be used jointly and the Subcontractor shall coordinate his work activities with other ongoing activities.
- B. See GC-19 For General Requirements

SC-8 TEMPORARY STORAGE OF CONSTRUCTION MATERIAL

The Contractor will provide space for storage of materials and equipment during performance of the Work Scope.

The Subcontractor is responsible for the security of his equipment and material.

SC-9 <u>CLEANUP AND WASTE DISPOSAL</u>

A. Cleanup

The Subcontractor shall maintain a neat and orderly work area. Cleanup of the construction work areas shall be required on a daily basis. Waste and debris shall not be allowed to accumulate in such quantities as to create an unsightly appearance, a

safety or fire hazard. All construction areas shall be thoroughly cleaned to the satisfaction of the Contractor prior to final acceptance of the completed project(s).

B. <u>Waste Disposal</u>

The Subcontractor shall provide suitable receptacles and dispose of all waste material such as paper, discarded containers, scrap lumber, scrap metals, etc., by removal from the work area to on-site disposal or storage areas.

SC-10 <u>CONSTRUCTION HEALTH AND SAFETY</u>

A. <u>Contractor Safety Program</u>

1. Subcontractors shall comply with all applicable Local, State, DOE/WSSRAP and Federal Safety Codes, Regulations, Standards and Special Procedures.

Subcontractor shall comply with the MK-Ferguson Construction Safety and Health Management Program. A Subcontractor may submit his own Safety Program to the Contractor for review. The Subcontractor's program must be equal to or more stringent than the WSSRAP Construction Safety and Health Program for acceptance and use. The Subcontractor's Safety and Health Program shall be an integral part of this Subcontract including mandatory implementation and compliance by the Subcontractor.

2. During the Pre-Bid Meeting a copy of the WSSRAP Construction Safety and Health Management Program will be made available for review per request. The successful bidder will be issued a copy with the Notice of Award by the Contractor's Subcontract Administrator. A Table of Contents for the WSSRAP Safety and Health Program is presented on Figure 1.

SAFETY AND HEALTH MANAGEMENT PROGRAM TABLE OF CONTENTS

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х	SUBCONTRACTOR RECORDS AND REPORTS	
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XII	SAFETY AND HEALTH TRAINING PROGRAM	
xiii	RADIOLOGICAL CONTROL AND MONITORING	
xıv	ENVIRONMENTAL CONTROL AND MONITORING	
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XVII	JOB SAFETY ANALYSIS	
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Appendix A	POSTERS, RECORDS AND REPORTING FORMS	
Appendix B	JOB SAFETY ANALYSIS	
Appendix C	PROJECT FORMS AND DISTRIBUTION TABLE	

B. <u>CONSTRUCTION SAFETY & HEALTH INITIAL INDOCTRINATION & TRAINING</u>

- 1. All Subcontractor personnel working on the site shall receive the basic Construction Safety and Health Initial Indoctrination and Training from the Construction Safety and Health Manager. The Indoctrination will be oral or written and will take approximately 30 minutes. No Subcontractor personnel will be permitted to work without having received this basic indoctrination.
- 2. If basic indoctrination is unavailable at the time of need, the Contractor will provide escorts until an indoctrination session can be coordinated.
- 3. Indoctrination requirements will not apply to routine delivery men who will be escorted while on site by Contractor personnel.

C. <u>Smoking, Drinking, Eating and Chewing</u> Restrictions

No smoking will be permitted in the immediate vicinity of any flammable liquids, gases or highly combustible material, or in any plant area posted as a non-smoking area. No smoking, eating or chewing will be permitted in any controlled area. Drinking within controlled areas is allowed only after hands are washed and frisked with appropriate radiation detection instrumentation.

SC-11 <u>HEALTH PHYSICS</u>

All work performed under this subcontract shall conform to the Construction Safety & Health Management Program-SEC XIII, SEC XIV, SEC XV.

The Subcontract work area (is located/is not located) within a radiologically controlled area. The Contractor will provide formal radiation training (2 hours in duration) and dosimetry requirements of bioassay and TLD prior to commencement of the work.

A. Work Conditions

- 1. Some areas beyond the control point have special Health Physics restrictions. These are controlled by ribbons, signs, and tags. Such restrictions shall be observed by the Subcontractor and applicable precautions taken.
- Radiation exposure on this subcontracting effort will be maintained well within allowable radiation exposures.
- 3. Personnel protection equipment: The level of protection could require coveralls, cotton gloves, and rubber over boots. These items will be provided by the Contractor unless otherwise specified.
- 4. Personnel Monitoring: All personnel will be monitored for radioactive contamination upon leaving any radiologically controlled area. Personnel will be instructed in self-monitoring procedures if necessary.
- 5. Vehicle, Tool and Equipment Monitoring:
 All vehicles, tools, & equipment, used
 inside a controlled area will be monitored
 for radioactive contamination by ES&H
 personnel before leaving the
 area.
- 6. ES&H Monitoring: The ES&H personnel will provide radiological surveillance over work activities and advise the Subcontractor on matters concerning radiation safety.
- B. Warning Signals: In emergency cases, the Subcontractor may depend on direct verbal information from the Contractor's personnel for warning signals. The Subcontractor's foreman and employees shall take action as directed. The Subcontractor's representative shall obtain the name, position, and agency of the messenger providing such direction.

- C. Disposition of Contaminated Equipment, Tools and Material:
 - 1. The Subcontractor shall use his own equipment in performing the required work under this Subcontract. All tools, vehicles, equipment and material will be inspected for radioactive contamination by Contractor personnel prior to removal from the construction area. Site experience has shown that no decontamination problems have occurred.
 - 2. Should the Subcontractor's tools, material, or equipment become contaminated, they will have to be decontaminated before removal from the area. If decontamination becomes necessary, the Contractor will provide instructions for decontamination which may consist of steam cleaning, dry brushing, or washing with appropriate liquids.

 Decontamination required beyond these described will be handled under Article 59, "CHANGES" of the General Provisions.
 - 3. If decontamination proves impracticable or impossible, the tools, material, or equipment in question will be retained and an equitable adjustment of the value of the equipment for same will be negotiated with the Subcontractor provided that:
 - a. There is no fault or negligence of the Subcontractor contributing to the contamination;
 - b. The Subcontractor has followed all the specific instructions of the authorized HP personnel who have surveillance over the worker;
 - c. Item or equipment confiscated from the Subcontractor is documented by a Confiscation Notice furnished to the Subcontract Administrator by the Contractor and signed by HP personnel and the Operations Manager or his designee.

- d. The Subcontractor allows reasonable time (a maximum of five (5) working days, excluding weekends and holidays) in which to attempt decontamination of the item(s) in question.
- 4. If reimbursement is required for tools, material or equipment, the following compensation will be made:
 - a. Tools valued less than \$300.00: 95% of replacement cost.
 - b. Tools/equipment \$300.00 and up:
 If less than one (1) year old or
 at top of depreciation schedule,
 at 75% of replacement cost; if at
 bottom of or off the depreciation
 schedule, at 50% of replacement
 cost.

SC-12 TEMPORARY HEAT AND COLD WEATHER ENCLOSURES

A. Temporary heating if needed shall be conducted in strict accordance with the applicable section of the Construction Safety and Health Management Program. The Subcontractor shall provide all temporary heat and/or heating equipment required for his use. The Subcontractor shall provide a watchman to check the heater when they are in operation and construction crews are not working at the job site. The use of open-type or oilpot salamanders, and gasoline-fueled heaters is prohibited. All products of combustion-type heaters shall be directly vented to the exterior of the structure and/or temporary enclosure.

SC-13 PARKING FACILITIES

A. Construction personnel shall park their vehicles in the designated parking area.

SC-14 <u>EATING FACILITIES</u>

A. No lunch room or cafeteria facilities are available.

SC-15 DAILY REPORT

- A. The Subcontractor shall submit a daily report to the Operations Department stating the following information:
 - 1. Work Accomplished
 - 2. Manpower
 - 3. Equipment
 - 4. Problems or delays

SC-16 INSURANCE (Reference GP-33)

A. Subcontractor shall procure and maintain, during the period that this order remains in force, insurance coverages and limits of not less than those set forth in this agreement.

Subcontractor will require all insurance companies issuing policies of insurance for Subcontractor, to certify to the Contractor in writing prior to commencement of any work, that such policies have been issued and are currently in effect.

Subcontractor agrees to waive any subrogation rights against Contractor.

Policies issued for Subcontractor shall be endorsed to include the following for the benefit of the Contractor.

- * A 30 day advance written notice in the event of cancellation, non-renewal or material change of any policy.
- * Contractor named as Additional Insured, as interests appear (Coverage B below).
- * Subcontractor's insurance is primary and any insurance maintained by Contractor is excess and non-contributory.
- * Cross liability or severability of interest clause (liability policies only).

INSURANCE COVERAGES

Type of Coverage: A

Worker's Compensation and, where an exposure exists Federal Employees Liability Act, U.S.

Longshoremen & Harbor Workers', Jones Act and Employer's Liability.

Policy Limits:

State and Federal Acts - Statutory Employer's Liability - \$1,000,000

Type of Coverage: B

Comprehensive General Liability including coverages for Independent Contractors, Products and Completed Operations (extending for at least 24 months after completion of operations) Blanket or Broad Form Contractual, Personal Injury Liability, Broad Form Property Damage, and where an exposure exists and explosion, collapse and underground (XCU) hazard exclusions deleted.

Policy Limits:

\$1,000,000 each occurrence Bodily Injury.

\$500,000 each occurrence Property Damage: or \$1,000,000 Combined Single Limit.

Type of Coverage: C

Comprehensive Automobile Liability including coverage for owned, non-owned and hired vehicles.

Policy Limits:

\$500,000 each person/\$1,000,000 each occurrence.

Bodily Injury/\$500,000 each occurrence Property Damage: Or \$1,000,000 Combined Single Limit.

B. In the event any work to be performed under this subcontract is sublet, the Subcontractor will require the same insurance coverage and limits from its lower tier Subcontractors or suppliers and will require said lower tier Subcontractors or suppliers to certify insurance coverage to the Contractor prior to the commencement of any sale or work.

- C. If Subcontractor shall fail to certify required insurance coverage to the Contractor as set forth above, before commencing work hereunder the Contractor may, at its option, place insurance of the character, nature and limits described above to cover the operations of the Subcontractor, paying the premiums for same and charging same to the Subcontractor.
- D. The Contractor, by requiring the foregoing minimum insurance coverages, will not be deemed to limit any of the other obligations or liabilities of the Subcontractor, deductibles if any, will be solely for the account of the Subcontractor.
- E. Should Contractor require additional insurance subsequent to the acceptance of this subcontract by Subcontractor, the net cost thereof shall be an addition to the Subcontract price. Such request must be made in writing to Subcontractor.
- F. Notwithstanding any trade practice or custom, neither the Subcontractor nor any lower tier subcontractor or supplier shall be entitled to the benefit of any insurance Contractor or its Customer have in effect of which either might have obtained.

SC-17 QUALITY ASSURANCE

All work performed under this Subcontract shall conform to the requirements of the Contractor's Quality Assurance Program Plan. The Subcontractors work will be subject to inspection and Quality Assurance audits by the Contractor.

A copy of the Quality Assurance Program Plan is available upon request from the contractor's office at the Weldon Spring Site.

SC-18 <u>VARIATIONS IN QUANTITIES</u>

In all cases (except cancellation of one (1) or more line items) where the quantity of a unit priced line item in the subcontract is an estimated quantity, and where the actual quantity of such line item varies by more than 10% above or 10% below the originally estimated quantity stated in the subcontract, any adjustment in the

unit rate shall be negotiated upon demand of either party for the quantities above or below the stated variation.

The adjustment shall be based on any increase or decrease in cost due solely to the variation above or below the originally estimated quantities.

SC-19 <u>SUPERINTENDENCE BY SUBCONTRACTOR</u>

- A. SEE GP-46 For General Requirements
- B. In accordance with Article GP-46, a (full-time working) (full-time non-working) supervisor shall be required.

SC-20 <u>DISPOSITION OF REMOVED EQUIPMENT, MATERIAL, AND SCRAP METAL</u>

All material or equipment removed from any existing system (plant site or quarry), or structure, shall be transported to a storage location as specified by the Construction Engineer, and turned over to MK-F for disposition for the Government.

All potentially reusable material or equipment shall be properly identified, tagged, and checked for contamination before transporting to a storage location identified by the Construction Engineer.

SC-21 SCHEDULE/SCHEDULE OF VALUES

- A. SEE GC-6 For General Requirements
- B. In General, schedule items should be limited to approximately __ weeks duration and limited to approximately __ percent of the total project value.

SC-22 <u>DISPOSITION OF EXCAVATED SOIL</u>

A. <u>Excess Soil</u>

Soil removed during excavation that is needed for backfill shall be stockpiled to allow the Contractor and others free and clear access (including road access) to all existing buildings and other operating facilities. In those cases where the stacking of soil near the excavation would

restrict vehicle traffic or prohibit access to the buildings and operating facilities, or create congestion of the work area, the Subcontractor shall temporarily stockpile the soil in an area identified by the Construction Engineer away from the excavation until backfill operations are designated. All excess soil removed from excavations (soil displaced in excess of that needed for backfill) shall be disposed of as directed by the Construction Engineer. All soil to be removed to the disposal site shall be free of non-soil materials such as concrete, wood, asphalt, metal, plastic, etc.

B. Radioactively Contaminated Soil

Radioactively contaminated soil (will/will not) be encountered during excavations required for this Subcontract. Disposal of said soil shall be at the direction of the Construction Engineer.

SC-23 TRAINING AND INDUSTRIAL HYGIENE (Clause 1)

A. <u>Training Requirements</u>

- 1. All personnel who will work in access controlled areas of this site must have undergone a health and safety training program in accordance with 29 CFR 1910.120. This training shall include appropriate respiratory protection training. The training shall be provided by the Subcontractor for its own personnel. Supervisory personnel shall receive additional training in health and safety in accordance with the same regulation.
- Documentation of completion of this training must be provided to the PMC prior to or upon the arrival of each employee at the site. This documentation must consist of, at a minimum, the dates of training, a description of the contents of the course, a copy of a letter or certificate signed by the person conducting the training course and any additional supporting information which the PMC may deem necessary. Personnel will not be allowed unescorted entry to the site until this documentation is received.

- 3. The Subcontractor shall provide additional training which addresses the major hazards at the WSS. The <u>Health and Safety</u>
 <u>Guidebook to the Weldon Spring Site</u>
 included with this Request for Proposal provides information on the known hazards at this site and their locations.
- 4. Once at the site, the PMC shall provide information and training on the known hazards at this site. This training will be included as a part of the initial orientation training session described in SC-11, Health Physics.

B. Medical Surveillance

- 1. All Subcontractor personnel working in access controlled areas of the Weldon Spring site meeting any of the following criteria shall participate in a medical surveillance program:
 - a. Personnel who are or are likely to be exposed to air contaminants in excess of the OSHA permissible exposure limits for 30 or more days per year.
 - b. Personnel who are or are likely to be exposed to asbestos fibers in excess of the OSHA action level for 30 or more days per year.
 - c. Personnel who will be required to wear respiratory protection.

This program shall be designed, at a minimum, to detect the effects of exposure to the major hazards listed below.

- 2. The medical surveillance program shall at a minimum include a baseline physical examination and an exit physical examination.
 - a. For the baseline physical examination, the Subcontractor shall provide documentation concerning the

fitness of each employee to work at a hazardous waste site at which the major hazards listed below are present. Where applicable, this shall include documentation that the employee is fit to wear a respirator and other required personal protective equipment. The above documentation must be received by the PMC upon arrival of the employee at the site. Documentation meeting the above criteria will be acceptable for physical examinations conducted up to three (3) months prior to the start of work at site.

b. Exit physical examination shall be conducted within two (2) weeks of the employee's last day of work at the site. Documentation of the exit physical examination, including a copy of the written opinion from the examining physician, relative to occupational exposure, shall be submitted to the PMC within two (2) weeks of the examination.

C. Work Conditions

1. Portions of the controlled access area in which this Subcontract is to be performed are known to be contaminated with the following materials:

Asbestos Uranium Chlorodiphenyls (PCBs) Nitroaromatics (TNT, 2,6-DNT, etc.) Mercury

The Subcontractor is responsible for providing adequate equipment and procedures for controlling exposure of its personnel to these materials and for controlling the spread of these materials from its

work area to other areas of the site. The Subcontractor shall be responsible for maintaining exposure levels below the established OSHA exposure limits specified in 29CFR 1926, Subpart D and 29CFR 1910, Subpart Z, as applicable.

- 2. Known contaminated areas and health hazards are posted by the PMC. Subcontractor personnel shall obey posted warnings.
- 3. Any health hazard(s) created by the Subcontractor shall be posted by the Subcontractor. The PMC shall be notified immediately of all such hazards.
- 4. Personal protection or engineering control equipment appropriate to the potential hazards to which personnel performing the work under this Subcontract shall be provided by the Subcontractor.
- 5. A heat stress hazard may exist at this site due to the warm, humid climate, the personal protective equipment requirements and the prohibition on eating and drinking in certain areas of the site. The Subcontractor is responsible for providing a heat stress management program for its personnel.
- 6. Biological hazards in the form of poison ivy, snakes, ticks, mosquitoes, and high airborne mold concentrations have been noted at this site.

SC-23 TRAINING AND INDUSTRIAL HYGIENE (Clause 2)

A. Training Requirements

 Subcontractor personnel who will work in access controlled access areas of this site must have undergone a health and safety training program in accordance with 29 CFR 1910.120. This training shall include appropriate respiratory protection training. The training shall be provided by the Subcontractor for its own personnel. Supervisory personnel shall receive additional training in health and safety in accordance with the same regulation.

- 2. Documentation of completion of this training must be provided to the PMC prior to or upon the arrival of each employee at the site. This documentation must consist of, at a minimum, the dates of training, a description of the contents of the course, a copy of a letter or certificate signed by the person conducting the training course and any additional supporting information which the PMC may deem necessary. Personnel will not be allowed unescorted entry to the site until this documentation is received.
- 3. The Subcontractor shall provide additional training which addresses the major hazards at the WSS. The <u>Health</u>
 and Safety Guidebook to the Weldon
 Spring Siteincluded with this Request for Proposal provides information on the known hazards at this site and their locations.
- 4. Once at the site, the PMC shall provide information and training on the known hazards at this site. This training will be included as a part of the initial orientation training session described in SC-11, Health Physics.

B. Work Conditions

Portions of the controlled access area in which this Subcontract is to be performed are known to be contaminated with the following materials:

> Asbestos Uranium Chlorodiphenyls (PCBs) Nitroaromatics (TNT, 2,6-DNT, etc.) Mercury

The Subcontractor is responsible for providing adequate equipment and procedures for controlling exposure of its personnel to these materials and for controlling the spread of these materials from its work area to other areas of the site.

The Subcontractor shall be responsible for maintaining exposure levels below the established OSHA exposure limits specified in 29CFR 1926, Subpart D and 29CFR 1910, Subpart Z, as applicable.

- 2. Known contaminated areas and health hazards are posted by the PMC. Subcontractor personnel shall obey posted warnings.
- 3. Any health hazard(s) created by the Subcontractor shall be posted by the Subcontractor. The PMC shall be notified immediately of all such hazards.
- 4. Personal protection or engineering control equipment appropriate to the potential hazards to which personnel performing the work under this Subcontract shall be provided by the Subcontractor.
- 5. A heat stress hazard may exist at this site due to the warm, humid climate, the personal protective equipment requirements and the prohibition on eating and drinking in certain areas of the site. The Subcontractor is responsible for providing a heat stress management program for its personnel.
- 6. Biological hazards in the form of poison ivy, snakes, ticks, mosquitoes, and high airborne mold concentrations have been noted at this site.

SC-23 TRAINING AND INDUSTRIAL HYGIENE (Clause 3)

A. Training Requirements

 Once at the site, the PMC shall provide information and training on the known hazards at this site. This training will be included as a part of the initial orientation training session described in SC-11, Health Physics.

B. Work Conditions

1. Portions of the controlled access area in which this Subcontract is to be performed are known to be contaminated with the following materials:

Asbestos Uranium Chlorodiphenyls (PCBs) Nitroaromatics (TNT, 2,6-DNT, etc.) Mercury

The Subcontractor is responsible for providing adequate equipment and procedures for controlling exposure of its personnel to these materials and for controlling the spread of these materials from its work area to other areas of the site.

The Subcontractor shall be responsible for maintaining exposure levels below the established OSHA exposure limits specified in 29CFR 1926, Subpart D and 29CFR 1910, Subpart Z, as applicable.

- Known contaminated areas and health hazards are posted by the PMC. Subcontractor personnel shall obey posted warnings.
- 3. Any health hazards(s) created by the Subcontractor shall be posted by the Subcontractor. The PMC shall be notified immediately of all such hazards.

- 4. Personal protection or engineering control equipment appropriate to the potential hazards to which personnel performing the work under this Subcontract shall be provided by the Subcontractor.
- 5. A heat stress hazard may exist at this site due to the warm, humid climate, the personal protective equipment requirements and the prohibition on eating and drinking in certain areas of the site. The Subcontractor is responsible for providing a heat stress management program for its personnel.
- 6. Biological hazards in the form of poison ivy, snakes, ticks, mosquitoes, and high airborne mold concentrations have been noted at this site.

SC-24 THE IMMIGRATION AND CONTROL ACT OF 1986; PL-99603

The Subcontractor shall be responsible for implementation and compliance with all applicable provisions of the Federal Immigration and Naturalization Act of 1986 which are incorporated herein by reference and made a part of the duties of the Subcontractor under this subcontract.

MKE DOCUMENT NO. 5121-C. DW - D - 01-0117 -00

DEPARTMENT OF ENERGY

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CONSTRUCTION DRAWINGS

RIDGE, TENNESSEE

EXISTING SITE PLAN

DATE PMC ENG'G MGR

MORRISON-KNUDSEN ENGINEERS, INC.

180 HOWARD ST SAN FRANCISCO, CA 94105

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cc: D. R. Lewis

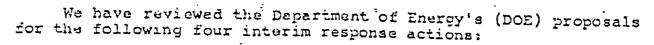
R. A. Nelson A. J. Stewart

UNITED STATES ENVIRONMENTAL PROTECTION AGENCY 3589-87-I. EAS. 010

HOV 5 1987

Mr. Rodney R. Nelson U.S. Department of Energy Weldon Spring Site Remedial Action Project/Office Route 2, Highway 94, South St. Charles, Missouri 63303





- Electric Power and Pole Removal,
- Overhead Piping/Asbastos Removal,
- Cleanup of Vicinity Property No. 7 on the Army Reserve Area, and
- Disposal of Containerized Chemicals.

Our comments on these proposals were sent to you earlier. You were also provided comments by the Missouri Department of Natural Resources (MDNR). No comments from the public were directed to the Environmental Protection Agency (EPA) and according to our records, there has been no public comment directed to MDNR or DOE.

We are in agreement these actions should proceed to ensure worker safety and reduce the further release of contaminants from this site. The EPA hereby approves these actions under the condition that the comments earlier provided by EPA and MDNR are adequately addressed. The MDNR has notified me they also concur with these actions. Please provide copies of any summary reports for these actions to EPA and MDNR.

We also received copies of the following four interim response actions:

- Dismantling of Building 401,
- Dismantling of Building 409,
- Removal of PCB Transformers, and
- Debris Consolidation.

My-C1-C1-C2

FILE NUMBER:

We will provide any comments on these within the agreed upon 21-day comment period. We are most pleased to see that activities are underway to stabilize the site and reduce contaminant release.

Sincerely yours,

Morris Kay Regional Administrator

cc: Dr. Fred Brunner, MDNR

bc: Robert Morby
Dan Shiel

Rowena Michaels

Ron Ritter

NOV 1 0 1987

Ms. B. Katherine Biggs
United States Environmental
Protection Agency
Region VII
726 Minnesota Avenue
Kansas City, Kansas 66101

.Dear Ms. Biggs:

USEPA COMMENTS ON INTERIM RESPONSE ACTIONS (IRA'S)

Enclosed is our response to the comments contained in your letter of October 8, 1987, regarding the following interim response actions:

- 1. Electric Power Line and Pole Removal
- 2. Overhead Piping/Asbestos Removal
- Army Reserve Area Vicinity Property No. 7
- 4. Disposal of Containerized Chemicals

We anticipate that this will adequately resolve the issues raised. We intend to proceed with action on these items in accordance with the enclosure.

If you have any questions, please give me a call.

Sincerely,
ORIGINAL SIGNED BY:

ORIGINAL SIGNED BY

R. R. Nelson Project Manager Weldon Spring Site Remedial Action Project

Enclosure: As stated

ACC: D. Bedan, MDNR
E. Brown, FLW
W/enclosure

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RESPONSES TO USEPA REVIEW COMMENTS ON IRA PACKAGES

Cleanup of Vicinity Property No. 7, Army Reserve Area

Comment: The proposal to cleanup this vicinity property calls for excavating the contaminated area to a depth of six (6) inches or where the radium concentration is below 15 pCi/g and then backfilling with clean material. The EPA suggests that while its criteria for residual radium in soil is satisfied by this approach, that we consider removing additional soil to reduce the concentration to below 5 pCi/g. As the area is small, little additional excavation would be required.

Response: Subsequent to the preparation of the IRA package for Army Reserve Vicinity Property #7, the Deparment of Army requested that the area not be backfilled upon completion of the cleanup as proposed by the DOE. The DOE will leave the excavation area open and apply the suface criteria of 5 pCi/g to this particular vicinity property.

Disposal of Containerized Chemicals

Comment: It is suggested that the specifications for this work might be strengthened by adding waste characterization procedures into Section 2.0 (Scope) of the document. The procedures are those which may be required under 40 CFR 260-268, or others required by the permit held by the Treatment, Storage and/or Disposal Facility.

Response: Procedures required in 40 CFR 260-268 will be referenced in Section 2.0 of the Request for Proposal. In addition, it will be emphasized that all waste characterization procedures which are required by the successful bidder's treatment, storage and/or disposal facilities permit must be satisfied. It will be required that these procedures (if applicable) be presented in the subcontractor's work plan.

Comment: EPA recommends that the specific subcontractor qualifications and experience in handling known and unknown potentially hazardous wastes be defined in the document.

Response: We are in agreement with the EPA that the Request for Proposal should contain subcontractor

qualifications and experience clauses. The

appropriate clauses will be added to the document.

Comment: The EPA recommends that the specification require

the successful bidder to identify the specific waste disposal facilities which will accept the containerized chemical waste, in the work plan

phase.

Response: The specification will be modified to include provision for certification by the subcontractor

that the waste disposal facilities meet the requirements when hazardous wastes are involved. The land disposal ban provision of RCRA will also be addressed as part of the subcontractor's work

plan.

General

Comment: The EPA review states that there is one(1)

deficiency common to the four proposals and that is that plans for onsite handling and storage of radioactive contaminated materials should be

developed.

Response: Plans for onsite handling and storage of

radioactive contaminated materials are currently

being finalized and will be provided under

separate cover.

NGV 10 1987

Mr. David E. Bedan
Missouri Department of
Natural Resources
Post Office Box 176
Jefferson City, Missouri 65102

Dear Mr. Bedan:

MISSOURI DNR COMMENTS ON INTERIM RESPONSE ACTIONS (IRA'S)

Enclosed is our response to the comments contained in Dr. Frederick A. Brunner's letter of October 26, 1987, regarding the following interim response actions:

- 1. Electric Power Line and Pole Removal
- 2. Overhead Piping/Asbestos Removal
- 3. Army Reserve Area Vicinity Property No. 7
- 4. Disposal of Containerized Chemicals

We anticipate that this will adequately resolve the issues raised and we intend to proceed with these actions in accordance with the enclosure.

If you have any questions, please give me a call.

Sincerely,
CRIGINAL SIGNED BY:
R. R. NELSON

R. R. Nelson
Project Manager
Weldon Spring Site
Remedial Action Project

Enclosure: As stated

cc: B. K. Biggs, USEPA
E. Brown, FLW
w/enclosure

FILE NUMBER:

PEER: JCoyne: x41:mw:11/9/87: (c:DNRCom.Ltr.)

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RESPONSES TO MDNR COMMENTS ON INITIAL FOUR (4) IRA PROPOSAL PACKAGES

I. Removal of Overhead Piping and Asbestos Removal

Comment: The DNR states that Missouri has adopted the Federal Clean Air Act standards for asbestos handling and has been delegated responsibility for implementing these standards and that asbestos and piping removal activities are subject to both the Missouri Air Conservation Law and the Missouri Solid Waste Management Law. DNR recommends that we maintain close contact with the Air Pollution Control Program to insure compliance with these standards.

Response: DNR Air Pollution Control Program office will be kept apprised of plans for asbestos removal work at the Weldon Spring Site. The WSSRAP will comply with requirements for disposal of asbestos and other demolition wastes in accordance with the Missouri Solid Waste Management Act.

Comment: The DNR has determined that the overhead piping and asbestos should be handled as a "special waste".

Response: We are proceeding to include the Special Waste Disposal Request form in the Request for Proposal for this work.

Comment: The DNR states that onsite handling of asbestos and other demolition waste may also be subject to Missouri Solid Waste Management Law requirements and requests that we furnish information on the size, design, location of the staging area and the amounts and methods of handling for the materials to be handled in the materials staging area.

Response: Information on handling and staging of the materials will be furnished to the DNR prior to issuing requests for proposals for this work.

modnrrsp,txtsheil

II. Disposal of Containerized Chemicals

Comment: Define specific levels at which the containerized wastes are considered radioactive.

Response: WSSRAP is developing concentration levels for wastes containing natural uranium for review and acceptance by concerned federal agencies. We will advise the DNR of this determination as it comes available. Until this determination is made we will retain on site containerized chemical materials which contain detectable levels of radioactive materials as determined by our onsite instruments.

Comment: What are removal plans for underground storage tanks on site?

Response: The underground tanks at the WSS have been sampled and found to contain only rainwater with trace amounts of motor fuel. The drainage and removal of the underground tanks is not part of the containerized chemical inventory and removal IRA Scope of Work. They may be removed as part of a subsequent IRA.

Comment: The document appears to be a generic outline for removal of waste. Items such as disposal facilities, transporters, waste characterization procedures, waste treatment procedures, etc. are not detailed.

Response: The IRA documentation represents a request for proposal to be sent to potential removal subcontractors. The responsibility for developing a detailed work plan addressing such items as disposal facilities, transporters, waste characterization procedures, waste treatment procedures, etc. rests with the successful bidder. requirement of the subcontract specification is the development of the subcontractor's work plan which must be approved by the WSSRAP before the work may begin. The DNR will be provided a copy of the subcontractor work plan when it becomes available for review. The WSSRAP office requests that the State provide a timely review (14 calendar days) to avoid delaying the subcontractor's

efforts. The State will be given 2-weeks notice of the interval at which time the work plan will be available for review.

III. Remedial Action on Army Vicinity Property

Comment: DNR states that the interim measure suggested is lacking in detail and should contain information on:

- How the removal is to be conducted?
- How and where excavated material will be contained?
- '- Health and safety plans for the work.
- Why is DOE recommending only remedial action for this vicinity property?
- When does DOE plan to remove the additional contamination in the other six locations?

Response: The technical requirements, i.e. specifications, drawings, special conditions, etc., did not accompany this IRA package for review as they were incomplete at the time of package submittal. Requirements for removal, containment and storage, and health and safety plans will be included in the Request for Proposal for this work.

The very small quantity (less than two cubic yards) of contaminated material can be removed manually and will not require any significant mechanical equipment. The technical specification developed for this work will provide required direction to the subcontractor.

We plan to excavate and place the contaminated soil in 55 gallon drums. The drums will be sealed, placed and stored in a dry, concrete floored building at the Weldon Spring Site awaiting final disposition. Total volume is anticipated to be less than 5 drums.

No unusual hazards are anticipated for this activity. The contract will require workers to abide by the WSSRAP Environmental, Safety and Health Plan.

DOE proposes performing remedial action of Army Vicinity Property No. 7 to remove contamination from an area where the Army has imminent construction plans. Cleanup of other vicinity properties, containing larger volumes of contaminated material, will be recommended based upon urgency of cleanup needs and development of storage facilities on the WSS.

Remediation of the remaining Army Vicinity Properties is tentatively scheduled for the first and second quarters of fiscal 1989.

IV. Power Line/Pole Removal

Response: The plan should address PCB contamination in regard to pole removal if transformers/capacitors containing PCB's were mounted on them.

Comment: The power poles to be removed in the IRA were specifically not associated with transformers containing PCBs. Those poles which have oil-cooled transformers mounted on them will be surveyed and removed at a later date.

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IRA-500-505



Department of Energy

Oak Ridge Operations
Weldon Spring Site
Remedial Action Project Office
Route 2, Highway 94 South
St. Charles, Missouri 63303

November 3, 1989

Mr. Gale Wright, Chief
Remedial Section/Superfund Branch
U. S. Environmental Protection
Agency
Region VII
726 Minnesota Avenue
Kansas City, Kansas 66101

Dear Mr. Wright:

MODIFICATION TO THE CONTAINERIZED CHEMICALS ENGINEERING EVALUATION/COST ANALYSIS (EE/CA)

The original containerized chemicals Engineering Evaluation/Cost Analysis (EE/CA) package stated that the RCRA waste, which could not be released off site, would be stored in Building 406. Since that time, it has been determined that a larger storage area is needed. Therefore, the purpose of this letter is to amend the EE/CA to show that the RCRA waste will be stored in Building 434.

Building 434 is undergoing modifications to meet RCRA storage requirements. Modifications should be completed by the end of 1989 and at that time, the contents of Building 406 will be moved.

This letter is to advise you of the modification to the containerized chemicals EE/CA. Please contact Ken Lawver if you have any questions.

Sincerely,

Acting Project Manager Weldon Spring Site Remedial Action Project

cc: Dave Bedan, MDNR

FILE NUMBER:



Department of Energy

Oak Ridge Operations
Weldon Spring Site
Remedial Action Project Office
Route 2, Highway 94 South
St. Charles, Missouri 63303

May 7, 1990

Mr. Dan Wall Remedial Project Manager U. S. Environmental Protection Agency Region VII 726 Minnesota Avenue Kansas City, Kansas 66101

Dear Mr. Wall:

RCRA COMPLIANCE FOR CONTAINERIZED CHEMICALS

In carrying out the Containerized Chemical Interim Response Action, we have found a disconnect between the requirements of RCRA (both State and EPA) and what we consider to be best conduct of operations for expediting secure storage of chemicals.

The enclosed flow chart illustrates the sequence of activities for collecting, sampling, repackaging and storing potentially hazardous chemicals at the site. Basically, the concept is to move chemicals that we find into the RCRA storage area as quickly as possible. To accomplish this we do a gross field analysis; move the material into the RCRA storage area; and then we do a detailed chemical analysis as required in order to fully meet the requirements of 40 CFR 264.13 and 10 CSR 25.5.262 (2) (c). These regulations require, however, that the detailed analysis be performed prior to placing the material in the RCRA storage area and that the containers be properly labeled during the entire storage period. Therefore, there is a period of time that we are in non-compliance. We do label all containers based on all information currently available and the containers will be relabeled as the detailed chemical analyses becomes available. For the most part, all required information for labeling is provided by the gross field analyses. In addition, all chemical waste is being packaged in DOT-approved containers regardless of its RCRA standing.

It is our belief that the approach we have adopted is preferable to leaving chemicals in an "as is" condition until detailed analysis is performed. I understand that this issue has been discussed with Dan Wall and Joe Davis and this letter is to solicit concurrence.

FILE NUMBER:

If you have any questions please give me or Ken Lawver a call.

Sincerely,

Stephen H. McCracken Project Manager

Weldon Spring Site Remedial Action Project

cc: Joe Davis, MDNR R. E. Hlavacek, PMC Peter Gross, SE-31



Department of Energy

Oak Ridge Operations Weldon Spring Site Remedial Action Project Office Route 2, Highway 94 South St. Charles, Missouri 63303

May 7, 1990

Mr. David E. Bedan Missouri Department of Natural Resources 205 Jefferson St. Jefferson State Office Bldg., 12th Floor P.O. Box 176 Jefferson City, MO 65102

Dear Mr. Bedan:

RCRA COMPLIANCE FOR CONTAINERIZED CHEMICALS

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Sincerely,

Stephen H. McCracken

Project Manager

Weldon Spring Site

Remedial Action Project

cc: Joe Davis, MDNR
R. E. Hlavacek, PMC

Peter Gross, SE-31

INTERIM RESPONSE ACTION (IRA) ADMINISTRATIVE RECORD FILE ARFS FILE # IR-0600

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604		Engineering Ev	valuations/Cost Analysis Approval or Decision
į	IR-0	600-604-1.01	EPA APPROVAL OF ELECTRICAL LINE/POLE REMOVAL
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IRA-600-601

IRA-600-602

IRA-600-603



Department of Energy

Oak Ridge Operations
Weldon Spring Site
Remedial Action Project Office
Route 2, Highway 94 South
St. Charles, Missouri 63303

September 03, 1987

Ms. Katherine Biggs United States Environmental Protection Agency, Region VII 726 Minnesota Avenue Kansas City, Kansas 66101

Dear Ms. Biggs:

Enclosed is the information regarding the removal of abandoned power lines and poles from the Weldon Spring Site, which we discussed in our telephone conference on July 24, 1987.

The site contains about 150,000 linear feet of electrical and communications wire and cable and 300 timber utility poles. Many of these poles are rotten and several have fallen during the past year. They present a safety hazard to site personnel.

We propose to remove the lines and poles. These will be disposed of off site. However, any radiologically contaminated material will be retained on site as described in the enclosed material. The work will be accomplished by a subcontractor to MK-Ferguson Company, our Project Management Contractor.

The enclosed index lists five (5) attachments including sampling information and drawings and specifications. If you have any questions please contact Jim Coyne of PEER Consultants, our support services contractor at (314) 441-8472.

Sincerely,

R. R. Nelson

Project Manager Weldon Spring Site Remedial Action Project

Enclosure: As stated

cc: Dave Bedan, MDNR

LIST OF ATTACHMENTS

Attachment 1

- Interim Measure - Electrical Line and Pole Removal - Summary

Attachment 2

- Specifications for Electric Power and Pole Removal

Attachment 3

- Special Conditions - Electrical Power and Pole Removal

Attachment 4

- Drawings - Chemical Plant Electrical Lines and Pole Removal

Attachment 5

- Radiological Survey of Chemical Plant Electrical Poles

Note - See also Addendum No. 1 dated 8/27/87

FILE NUMBER

INTERIM MEASURE

ELECTRICAL LINE AND POLE REMOVAL

SUMMARY

This task consists of removing abandoned power line and support poles at the Weldon Spring Site. About 300 timber utility poles and 150,000 lf of electrical and communications cable and wire are involved.

We propose to do this work now to improve safety conditions at the site. Many poles are rotted at ground level and pose a potential hazard to personnel working in the chemical plant area. In fact, a number of poles have failed and fallen during the past year.

Surveys of utility poles (see Attachment No. 5) show levels of contamination, where present, to be well within limits acceptable for release of these materials from the site. The plan (see Attachment No. 2) calls for survey of the poles and lines to verify they are uncontaminated immediately prior to release. Any poles found contaminated will be segregated and stored for later disposal. We expect these materials to be releasable. Should we find during removal that a pole or section thereof is contaminated above release levels we plan to leave that pole or portion thereof in place until we have constructed a suitable on-site interim storage area. Contamination, if present, is expected to occur at or near ground level on the poles. Thus, cutting those poles off above ground level would alleviate the immediate safety hazard. The butt end could remain in-situ for later removal.

SET I.D

WSSRA PROJECT REVIEWS AND APPROVALS

Subject:	WSSRA PROJECT - CP	03 в≭ 5121-С:SP-S-01-0190- 02	}
	Specification Section 02053 Demolition of Electrical Line and Poles	(DOCUMENT NO.)	
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	FILE NUMBER:	O.A. REVIEW ENTRY NO. NA	

SECTION 02053

DEMOLITION OF ELECTRICAL LINES AND POLES

PART 1 - GENERAL

1.1 SCOPE OF WORK

- A. This Section describes the requirements for the removal of existing timber poles, and above-ground electrical, power, and communications lines at a uranium feed materials processing plant as located on the Subcontract Drawings.
- B. Transformers and substations will be removed by others.

1.2 DEFINITIONS

Utility: For the purpose of this Section, above ground utility means any service such as electric power systems, lighting systems, telephone, and associated support structures.

1.3 JOBSITE CONDITONS

See Special Conditions for restrictions applicable to work in areas of potential radiation hazard.

PART 2 - PRODUCTS

(Not Used)

PART 3 - EXECUTION

3.1 GENERAL

A. Removal of the utility shall not be undertaken until written approval for such work has been obtained from the Contractor.

FILE NUMBER

O.A. REVIEW
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MIT AND DATE PAGE 7

Document No. 5121-C:SP-S-01-0190-03
Re-Issued for Construction-Revision 3
Demolition of Electrical Lines and Poles
02053 - 1

- B. Except for the utility lines shown to remain, all on-site above-ground electric power, lighting, and communication lines shall be removed, together with all poles and supporting structures. Conduit and cable risers on poles shall be cut off at ground level. Poles, grounding rods and guy wire anchors shall be pulled and the voids filled to the ground surface with tamped earth. Acceptable backfill material is available on site. Relocation of lines, if required, will be performed by others.
- C. To assure uninterrupted service to off-site customers, the Subcontractor shall obtain approval from the affected utility companies before disturbing utilities. Utilities shall be protected from damage by demolition operations until they are removed from service.
- D. Although Contractor intends to de-energize all wire to be removed, Subcontractor shall verify that all electrical items within Subcontractor's Scope of Work are de-energized prior to commencing work.
- E. Backfilled holes and unpaved areas disturbed by demolition operations shall be seeded as specified in Section 02930.

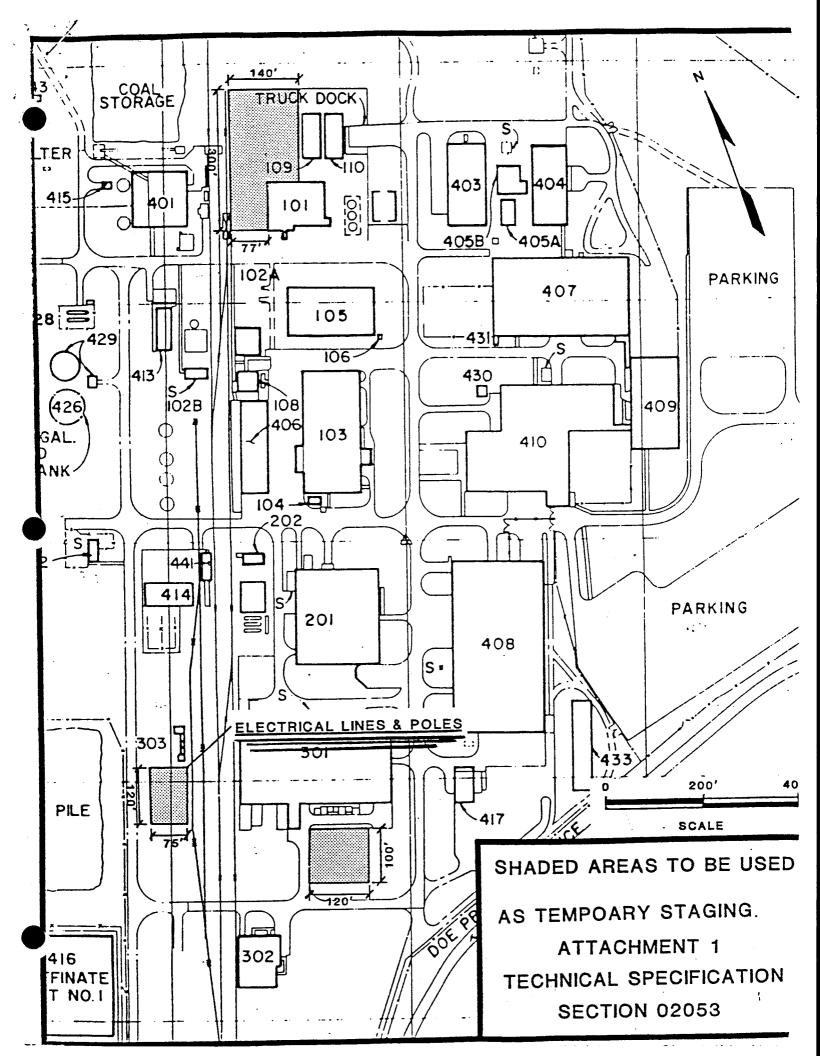
3.2 DISPOSAL OF DEMOLITION DEBRIS

- A. All materials will be subject to classification by the Contractor. The materials will be classified as contaminated and uncontaminated. No contaminated material is expected; however, any contaminated material, if encountered and so classified, shall be stacked in the on-site storage area shown on Figure 02053-1. Uncontaminated material shall be disposed of off-site.
- B. After a pole is pulled, the lower part of the pole shall be further checked and classified with regard to contamination. Where a pole is removed from a location where the soil is found to be contaminated, the pole shall be cut off two feet above ground contact level.

END OF SECTION 02053

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WSSRA PROJECT REVIEWS AND APPROVALS

Subject:	WSSRA PROJECT - CP Specification Section 02930 Seeding	_5121-Ç:SP-S-01-0 (DOCUMENT	
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FILE NUMBER

SECTION 02930

SEEDING

PART 1 - GENERAL

1.1 SCOPE

- A. This Specification Section describes the requirements for seeding of areas disturbed by demolition operations.
- B. The following areas shall be seeded as specified herein: areas where poles, yard structures, and foundation of any description have been removed, all backfilled areas, and all areas disturbed by demolition operations; except that surfaced areas and solid rock shall not be seeded.

PART 2 - PRODUCTS

2.1 SEED

- A. All seed shall comply with the requirements of the Missouri Seed Law. Minimum purity shall be 90 percent. Minimum germination including hard seed shall be 80 percent. Maximum weed seed shall be 1.0 percent.
- B. The seed mixture from August 1 through April 15 shall be Annual Rye Grass 10 lb/acre; Orchard Grass 2 lb/acre; Lespedeza 5 lb/acre; and Switch Grass 3 lb/acre.
- C. The seed mixture from April 16 through July 31 shall be Big Blue Stem - 10 lb/acre; Indian Grass - 10 lb/acre; and Little Blue Stem - 10 lb/acre.

2.2 FERTILIZER

Fertilizer shall contain equal percentages of nitrogen, phosphorous, and potash, applied at a rate of 65 pounds of each ingredient per acre.

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2.3 MULCH

Mulch shall be the cereal straw from stalks of oats, rye, wheat, or barley. The straw shall be clean and bright, relatively free of foreign material, and be dry enough to spread properly. If the above straw specifications cannot be met practicably, the foliage of the following plants may, with the Contractor's approval, be substituted: smooth brome, timothy, orchard grass, reed canary grass, red top, millet, blue stem, indian grass, alfalfa, birdsfoot trefoil, and vetch. The foliage shall be taken from areas of relatively pure stands of plants of the current season's growth.

PART 3 - EXECUTION

3.1 GENERAL

- A. The seedbed shall be prepared and fertilized and shall be in a firm but uncompacted condition with a relatively fine texture at the time of seeding.
- B. Seeding shall be done before the proposed seedbed becomes eroded, crusted over, or dried out and shall not be done when the ground is in a frozen condition or covered with snow.

3.2 HYDRAULIC SEEDING AND FERTILIZING

- A. In lieu of mechanical application of seed and fertilizer, hydraulic application may be used. On slopes steeper than 2:1, or when seeding is applied to a previously seeded and mulched area, seed and fertilizer may be applied hydraulically in a single operation and incorporation into the soil will not be required. On all other slopes, seed and fertilizer may be applied hydraulically provided the seed and fertilizer are applied separately. The seed and fertilizer shall be incorporated into the soil as specified under mechanical seed application.
- B. Seed and fertilizer, separately or in combination, shall be mixed with water and constantly agitated so that a uniform mixture can be applied hydraulically to the specified areas. The ratio of seed and fertilizer to water shall be calculated by determining the number of square feet covered by a given quantity of water. Seed shall not be added to the water more than 4 hours before application.

3.3 DRY SEEDING

Dry seeding shall be done mechanically with equipment designed for even distribution of dry seed. The equipment may either be hand operated, such as knapsack seeder, or be tractor-drawn, such as seed drill, except that tractor-drawn equipment will not be permitted on a previously seeded and mulched area. Seed scattered on the surface shall be covered with approximately 1/4 inch of soil by raking or other approved methods except that raking will not be required when seeding a previously seeded and mulched area. Seed placed in the soil shall be approximately 1/4 inch below the surface. After completing the seeding operation, the contractor shall firm the area by rolling, if in the judgment of the Contractor the seedbed is either too loose or contains clods which would reduce the germination of the seed. When rolling is required, a lawn-type roller shall be used and care shall be taken to avoid over-compacting the soil.

3.4 MULCHING

Mulch shall be applied after seeding at the rate of 2 tons per acre.

END OF SECTION 02930



SPECIAL CONDITIONS
SUBCONTRACT NO. 3589-SC-WP018

ELECTRICAL POWER AND POLE REMOVAL

MK-FERGUSON COMPANY
WELDON SPRING REMEDIAL ACTION PROJECT
ST. CHARLES, MISSOURI

Dated: 07-28-87

3589-SC-WP018

SPECIAL CONDITIONS

MK-FERGUSON CONSTRUCTION SUBCONTRACTS

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WP018s-c.txt Dated: July 28, 1987

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SC-1 WORK TO BE ACCOMPLISHED

The work scope consists of furnishing all labor, supervision, equipment and materials (except as otherwise specified), and performing all work in strict accordance with Federal, State and Local laws, codes and regulations. The work in general consists of:

Removal of all overhead wiring (power, telephone, etc.) from the site yard areas outside of the building. Poles are to be removed, cut and placed with rolled wire in an area designated on site.

The above description of work is for general information only and in no way limits the responsibility of the Subcontractor for constructing the Work in strict accordance with the subcontract drawings and specifications.

SC-2 DEFINITIONS

See GP2 For General Definitions

SC-3 LOCATION

The work to be accomplished is located at the abandoned Uranium Feed Materials Processing Plant at Weldon Spring, Missouri.

SC-4 HOLIDAYS

The following will be observed as holidays by the Contractor:

- * Christmas Day Columbus Day
- * New Year's Day Presidents' Day Memorial Day
- * Independence Day Labor Day Thanksgiving Day Day Following Thanksgiving
- * Holidays occurring on Saturday or Sunday will be on Friday or Monday.

Upon written notice from the Subcontractor 5 days in advance, the Contractor will adequately man the job on Contractor holidays not observed by the Subcontractor.

SC-5 SECURITY REQUIREMENTS

A. Access

- 1. Access to the site is via the main gate on Highway 94 South. The gate is normally open from 7 a.m. to 6 p.m. and the normal work day is from is 8 a.m. to 4:30 p.m., Monday thru Friday.
- 2. Special controls are established at the Weldon Spring site to govern personnel access. A guard station is located approximately 50 feet inside the main gate.
- 3. All privately and Subcontractor owned vehicles shall stop at the guard station.
- 4. The Subcontractor will have a specific identification symbol prominently displayed on each vehicle for entry to the site. A sample of this symbol will be given to MK-F prior to starting work.
- 5. If no specific identification symbol is displayed on a vehicle, each person shall sign in and out with the guard before entry to the site.

SC-6 TEMPORARY UTILITIES

A. <u>Telephone Service</u>

Telephone service will not be provided by the Contractor. The Subcontractor shall make his own arrangements for any telephone service desired.

B. Electrical Power for Construction

Electrical power for construction will not be provided by the Contractor. The Subcontractor shall make his own arrangements for construction power.

C. Toilet and Washroom Facilities

Toilet and washroom facilities shall be provided by the Contractor. Emergency showers are available in Trailer 7 and 8.

D. <u>Temporary Water Facilities</u>

- 1. Drinking water shall be furnished by the Subcontractor.
- 2. If authorized by the Contractor, temporary supply lines, standpipes, and connections over and above those facilities presently existing and available shall be furnished and installed by the Subcontractor and shall be removed upon the completion of the need for such temporary facilities.

SC-7 CONSTRUCTION RESTRAINTS

There are no known restraints at this time. The Subcontractor should be aware that more than one subcontractor may be working in the areas identified in the Work Scope. These areas are to be used jointly and the Subcontractor shall be responsible to coordinate his work activities with ongoing activities.

SC-8 TEMPORARY STORAGE OF CONSTRUCTION MATERIAL

The Contractor will provide space for storage of materials and equipment during performance of the Work Scope.

The Subcontractor is responsible for the security of his equipment and material.

SC-9 CLEANUP AND WASTE DISPOSAL

A. Cleanup

The Subcontractor shall maintain a neat and orderly work area. Cleanup of the construction work areas shall be required on a daily basis. Waste and debris shall not be allowed to accumulate in such quantities as to create an unsightly appearance, a safety or fire hazard. All construction areas shall be thoroughly cleaned to the satisfaction of the Contractor prior to final acceptance of the completed project(s).

B. Waste Disposal

The Subcontractor shall provide suitable receptacles and dispose of all waste material

such as paper, discarded containers, scrap lumber, scrap metals, etc., by removal from the work area to on-site disposal or storage areas.

SC-10 CONSTRUCTION HEALTH AND SAFETY

A. Contractor Safety Program

 Subcontractors shall comply with all applicable Local, State, DOE/WSSRAP and Federal Safety Codes, Regulations, Standards and Special Procedures.

A Subcontractor may submit his own Safety Program to the Contractor for review. The Subcontractor's program must be equal to or more stringent than the WSSRAP Construction Safety and Health Program for acceptance and use. The Subcontractor's Safety and Health Program shall be an integral part of this Subcontract including mandatory implementation and compliance by the Subcontractor.

2. During the Pre-Bid Meeting a copy of the WSSRAP Construction Safety and Health Management Program will be made available for review per request. The successful bidder will be issued a copy with the Notice of Award by the Contractor's Subcontract Administrator. A Table of Contents for the WSSRAP Safety and Health Program is presented on Figure 1.

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Appendix C	PROJECT FORMS AND DISTRIBUTION TABLE	

B. CONSTRUCTION SAFETY & HEALTH INITIAL INDOCTRINATION & TRAINING

- 1. All Subcontractor personnel working on the site shall receive the basic Construction Safety and Health Initial Indoctrination and Training from the Construction Safety and Health Manager. The Indoctrination will be oral or written and will take 30 minutes. No Subcontractor personnel will be permitted to work without having received this basic indoctrination.
- 2. Indoctrination requirements will not apply to routine delivery men who will be escorted while on site by Contractor personnel.

C. Smoking, Drinking, Eating and Chewing Restrictions

No smoking will be permitted in the immediate vicinity or any flammable liquids, gases or highly combustible material, or in any plant area posted as a non-smoking area. No smoking, eating or chewing will be permitted in any controlled area.

SC-11 HEALTH PHYSICS

All work performed under this subcontract shall conform to the Construction Safety & Health Management Program-SEC XIII, SEC XIV, SEC XV.

The Subcontract work area is located within a radiologically controlled area. The Contractor will provide formal radiation training (2 hours in duration) and dosimetry requirements of bioassay and TLD prior to commencement of the work.

A. Work Conditions

- 1. Some areas beyond the control point have special Health Physics restrictions. These are controlled by ribbons, signs, and tags. Such restrictions shall be observed by the Subcontractor and applicable precautions taken.
- Radiation exposure on this subcontracting effort will be maintained well within allowable radiation exposures.

- 3. Personnel protection equipment: The level of protection will generally require coveralls, cotton gloves, and rubber boots. These items will be provided by the Contractor.
- 4. Personnel Monitoring: All personnel will be monitored for radioactive contamination upon leaving radiologi cally controlled area. Personnel will be instructed in self-monitoring procedures if necessary.
- 5. Vehicles Monitoring: All vehicles, tools, & equipment, used inside a controlled area will be monitored for radioactive contamination by ES&H personnel before leaving the area.
- 6. ES&H Monitoring: The ES&H personnel will provide radiological surveillance over work activities and advise the Subcontractor on matters concerning radiation safety.
- B. Warning Signals: In emergency cases, the Subcontractor may depend on direct verbal information from the Contractor's personnel for warning signals. The Subcontractor's foreman and employees shall take action as directed. The Subcontractor's representative shall obtain the name, position, and agency of the messenger providing such direction.
- C. Disposition of Contaminated Equipment, Tools and Material:
 - 1. The Subcontractor shall use his own equipment in performing the required work under this Subcontract. All tools, vehicles, equipment and material will be inspected for radioactive contamination by Contractor personnel prior to removal from the construction area. Site experience has shown that no decontamination problems have occurred.
 - 2. Should the Subcontractor's tools, material, or equipment become

contaminated, they will have to be decontaminated before removal from the area. If decontamination becomes necessary, the Contractor will provide instructions for decontamination which may consist of steam cleaning, dry brushing, or washing with appropriate liquids. Decontamination required beyond these described will be handled under Article 59, "CHANGES" of the General provisions.

- 3. If decontamination proves impracticable or impossible, the tools, material, or equipment in question will be retained and an equitable adjustment of the value of the equipment for same will be negotiated with the Subcontractor provided that:
 - a. There is no fault or negligence of the Subcontractor contributing to the contamination;
 - b. The Subcontractor has followed all the specific instructions of the authorized HP personnel who have surveillance over the worker;
 - c. Item or equipment confiscated from the Subcontractor is documented by a Confiscation Notice furnished to the Subcontract Administrator by the Contractor and signed by HP personnel and the Operations Manager or his designee.
 - d. The Subcontractor allows reasonable time (a maximum of five (5) working days, excluding weekends and holidays) in which to attempt decontamination of the item(s) in question.
- 4. If reimbursement is required for tools, material or equipment, the following compensation will be made:
 - a. Tools valued less than \$300.00: 95% of replacement cost.

b. Tools/equipment \$300.00 and up:
If less than one (1) year old or at
top of depreciation schedule, at
75% of replacement cost; if at
bottom of or off the depreciation
schedule, at 50% of replacement
cost.

SC-12 TEMPORARY HEAT AND COLD WEATHER ENCLOSURES

Temporary heating if needed shall be Α. conducted in strict accordance with the applicable section of the Construction Safety and Health Management Program. The Subcontractor shall provide all temporary heat and/or heating equipment required for his use. The Subcontractor shall provide a watchman to check the heater when they are in operation and construction crews are not working at the job site. The use of open-type or oilpot salamanders, and gasoline-fueled heaters is prohibited. All products of combustion-type heaters shall be directly vented to the exterior of the structure and/or temporary enclosure.

SC-13 PARKING FACILITIES

A. Construction personnel shall park their vehicles in the designated parking area.

SC-14 EATING FACILITIES

A. No lunch room or cafeteria facilities are available.

SC-15 DAILY REPORT

- A. The subcontractor shall submit a daily report to the Operations Department stating the following information:
 - 1. Work Accomplished
 - 2. Manpower
 - 3. Equipment
 - 4. Problems or delays

SC-16 <u>INSURANCE</u> (Reference GP-33)

A. Subcontractor shall procure and maintain, during the period that this order remains in force, insurance coverages and limits of not less than those set forth in this agreement. Subcontractor will require all insurance companies issuing policies of insurance for Subcontractor, to certify to the Contractor in writing prior to commencement of any work, that such policies have been issued and are currently in effect.

Subcontractor agrees to waive any subrogation rights against Contractor.

Policies issued for Subcontractor shall be endorsed to include the following for the benefit of the Contractor.

- * A 30 day advance written notice in the event of cancellation, non-renewal or material change of any policy.
- * Contractor named as Additional Insured, as interests appear (Coverage B below).
- * Subcontractor's insurance is primary and any insurance maintained by Contractor is excess and non-contributory.
- * Cross liability or severability of interest clause (liability policies only).

INSURANCE COVERAGES

Type of Coverage: A

Worker's Compensation and, where an exposure exists Federal Employees Liability Act, U.S. Longshoremen & Harbor Workers', Jones Act and Employer's Liability.

Policy Limits:

State and Federal Acts - Statutory Employer's Liability - \$1,000,000

Type of Coverage: B

Comprehensive General Liability including

coverages for Independent Contractors, Products and Completed Operations (extending for at least 24 months after completion of operations) Blanket or Broad Form Contractual, Personal Injury Liability, Broad Form Property Damage, and where an exposure exists and explosion, collapse and underground (XCU) hazard exclusions deleted.

Policy Limits:

\$1,000,000 each occurrence Bodily Injury.

\$500,000 each occurrence Property Damage: or \$1,000,000 Combined Single Limit.

Type of Coverage: C

Comprehensive Automobile Liability including coverage for owned, non-owned and hired vehicles.

Policy Limits:

\$500,000 each person/\$1,000,000 each occurrence.

Bodily Injury/\$500,000 each occurrence Property Damage: Or \$1,000,000 Combined Single Limit.

- B. In the event any work to be performed under this subcontract is sublet, the Subcontractor will require the same insurance coverage and limits from its lower tier Subcontractors or suppliers and will require said lower tier Subcontractors or suppliers to certify insurance coverage to the Contractor prior to the commencement of any sale or work.
- C. If Subcontractor shall fail to certify required insurance coverage to the Contractor as set forth above, before commencing work hereunder the Contractor may, at its option, place insurance of the character, nature and limits described above to cover the operations of the Subcontractor, paying the premiums for same and charging same to the Subcontractor.

- D. The Contractor, by requiring the foregoing minimum insurance coverages, will not be deemed to limit any of the other obligations or liabilities of the Subcontractor, deductibles if any, will be solely for the account of the Subcontractor.
- E. Should Contractor require additional insurance subsequent to the acceptance of this subcontract by Subcontractor, the net cost thereof shall be an addition to the Subcontract price. Such request must be made in writing to Subcontractor.
- F. Notwithstanding any trade practice or custom, neither the Subcontractor nor any lower tier subcontractor or supplier shall be entitled to the benefit of any insurance Contractor or its Customer have in effect of which either might have obtained.

SC-17 QUALITY ASSURANCE

All work performed under this Subcontract shall conform to the requirements of the Contractor's Quality Assurance Program Plan. The Subcontractors work will be subject to inspection and Quality Assurance audits by the Contractor. A copy of the Quality Assurance Program Plan is available upon request from the contractor's office at the Weldon Spring Site.

SC-18 RESERVED

SC-19 SUPERINTENDENCE BY SUBCONTRACTOR

- A. SEE GP-46 For General Requirements
- B. In accordance with Article GP-46, a full-time working supervisor shall be required.

SC-20 DISPOSITION OF REMOVED EQUIPMENT, MATERIAL, AND SCRAP METAL

All material or equipment removed from any existing system (plant site or quarry), or structure, shall be transported to a storage location as specified by the Construction Engineer, and turned over to MK-F for disposition for the Government.

All potentially reusable material or equipment shall be properly identified, tagged, and checked for contamination before transporting to a storage location identified by the Construction Engineer.

SC-21 SCHEDULE/SCHEDULE OF VALUES

- A. SEE GC-6 For General Requirements
- B. In General, schedule items should be limited to approximately two (2) weeks duration and limited to approximately ten (10%) percent of the total project value.

SC-22 <u>DISPOSITION OF EXCAVATED SOIL</u>

A. Excess Soil

Soil removed during excavation that is needed for backfill shall be stacked in such a manner as to allow the Contractor and others free and clear access (including road access) to all existing buildings and other operating facilities. In those cases where the stacking of soil near the excavation would restrict vehicle traffic or prohibit access to the buildings and operating facilities, or create congestion of the work area, the Subcontractor shall temporarily stockpile the soil in an area identified by the Construction Engineer away from the excavation until backfill operations are designated. All excess soil removed from excavations (soil displaced in excess of that needed for backfill) shall be disposed of as directed by the Construction Engineer. All soil to be removed to the disposal site shall be free of non-soil materials such as concrete, wood, asphalt, metal, plastic, etc.

B. Radioactively Contaminated Soil

Radioactively contaminated soil will be encountered during excavations required for this Subcontract. Disposal of said soil shall be at the direction of the Construction Engineer.

SC-23 TRAINING AND INDUSTRIAL HYGIENE

A. Training Requirements

- 1. All personnel who will work in access controlled areas of this site must have undergone a health and safety training program in accordance with 29 CFR 1910.120. This training shall include appropriate respiratory protection training. The training shall be provided by the Subcontractor for its own personnel. Supervisory personnel shall receive additional training in health and safety in accordance with the same regulation.
- 2. Documentation of completion of this training must be provided to the PMC prior to or upon the arrival of each employee at the site. This documentation must consist of, at a minimum, the dates of training, a description of the contents of the course, a copy of a letter or certificate signed by the person conducting the training course and any additional supporting information which the PMC may deem necessary. Personnel will not be allowed to enter the site until this documentation is received.
- 3. The Subcontractor shall provide training which addresses the major hazards at the WSS. The Health and Safety Guidebook to the Weldon Spring Site included with this request for proposal presents further details on the known hazards at this site and their locations.
- 4. Once at the site, personnel shall be briefed by the PMC on the known hazards at this site. This briefing will require approximately four hours per employee during an initial orientation to the site. Locations of WSS hazards will be discussed during this orientation.

B. Medical Surveillance

- 1. All personnel who work in access controlled areas of the site must be participants in a medical surveillance program. This program must be designed, at a minimum, to detect the effects of exposure to the major hazards listed above.
- 2. Documentation of the fitness of each employee to work and, where applicable, to wear a respirator at a hazardous waste site or near the major hazards listed above must be received by the PMC upon or prior to the arrival of the employee at the site.

C. Work Conditions

The work area in which this contract is to be performed is known to be contaminated with the following materials:

> Asbestos Uranium Chlorodiphenyls (PCBs) Nitroaromatics (TNT, 2,6-DNT, etc.) Mercury

The Subcontractor is responsible for providing adequate equipment and procedures for controlling exposure of its personnel to these materials and for controlling the spread of these materials from its work area to other areas of the site.

- Known contaminated areas and health hazards are posted by the PMC. Subcontractor personnel shall obey posted warnings.
- 3. Any health hazard(s) created by the Subcontractor shall be posted by the Subcontractor. The PMC shall be notified immediately of all such hazards.

- 4. Personal protection or engineering control equipment appropriate to the potential hazards to which personnel performing the work under this subcontract shall be provided by the Subcontractor.
- 5. Heat stress has been noted at this site due to the warm, humid climate, the personal protective equipment requirements and the prohibition on eating and drinking in certain areas of the site. The Subcontractor is responsible for providing a heat stress management program to its personnel.
- 6. Biological hazards in the form of poison ivy, snakes, ticks, mosquitoes, and high airborne mold concentrations have been noted at this site.

W.Y.

EE DRAWING NO.5121E - CP - 525 OR GENERAL NOTES, REFERENCE RAWINGS AND LEGEND.



MKE DOCUMENT NO. 5121-C: DW-D-01-0115-02

DEPARTMENT OF ENERGY OAK RIDGE, TENNESSEE

CONSTRUCTION DRAWINGS
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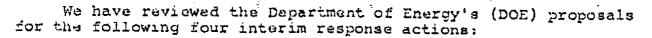
UNITED STATES ENVIRONMENTAL PROTECTION AGENCY A. J. Stewart

3589-87-I.EPA-010

1987 MOV 5

Mr. Rodney R. Nelson U.S. Department of Energy Weldon Spring Site Remedial Action Project/Office Route 2, Highway 94, South St. Charles, Missouri 63303

Dear Mr. Nelson:



Electric Power and Pole Removal,

Overhead Piping/Asbestos Removal,

Cleanup of Vicinity Property No. 7 on the Army Reserve Area, and

Disposal of Containerized Chemicals.

Our comments on these proposals were sent to you earlier. You were also provided comments by the Missouri Department of Natural Resources (MDNR). No comments from the public were directed to the Environmental Protection Agency (EPA) and according to our records, there has been no public comment directed to MDNR or DOE.

We are in agreement these actions should proceed to ensure worker safety and reduce the further release of contaminants from this site. The EPA hereby approves these actions under the condition that the comments earlier provided by EPA and MDNR are adequately addressed. The MDNR has notified me they also concur with these actions. Please provide copies of any summary reports for these actions to EPA and MDNR.

We also received copies of the following four interim response actions:

Dismantling of Building 401,

Dismantling of Building 409,

Removal of PCB Transformers, and

Debris Consolidation.

DOCUMENT NUMBER: T. GOO -604-1.0

We will provide any comments on these within the agreed upon 21-day comment period. We are most pleased to see that activities are underway to stabilize the site and reduce contaminant release.

Sincerely yours,

Morris Kay Regional Administrator

cc: Dr. Fred Brunner, MDNR

bc: Robert Morby Dan Shiel

Rowena Michaels

Ron Ritter

CONCURRENCES

NOV 1 0 1987

Ms. B. Katherine Biggs
United States Environmental
Protection Agency
Region VII
726 Minnesota Avenue
Kansas City, Kansas 66101

Dear Ms. Biggs:

USEPA COMMENTS ON INTERIM RESPONSE ACTIONS (IRA'S)

Enclosed is our response to the comments contained in your letter of October 8, 1987, regarding the following interim response actions:

- 1. Electric Power Line and Pole Removal
- Overhead Piping/Asbestos Removal
- Army Reserve Area Vicinity Property No. 7
 Disposal of Containerized Chemicals

We anticipate that this will adequately resolve the issues raised. We intend to proceed with action on these items in accordance with the enclosure.

If you have any questions, please give me a call.

Sincerely,
ORIGINAL SIGNED BY:
R. R. NELSON

R. R. Nelson Project Manager Weldon Spring Site Remedial Action Project

Enclosure: As stated

cc: D. Bedan, MDNR
E. Brown, FLW
w/enclosure

DOCUMENT NUMBER: T-400-604-1.02

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RESPONSES TO USEPA REVIEW COMMENTS ON IRA PACKAGES

Cleanup of Vicinity Property No. 7, Army Reserve Area

Comment: The proposal to cleanup this vicinity property calls for excavating the contaminated area to a depth of six (6) inches or where the radium concentration is below 15 pCi/g and then backfilling with clean material. The EPA suggests that while its criteria for residual radium in soil is satisfied by this approach, that we consider removing additional soil to reduce the concentration to below 5 pCi/g. As the area is small, little additional excavation would be required.

Response: Subsequent to the preparation of the IRA package for Army Reserve Vicinity Property #7, the Deparment of Army requested that the area not be backfilled upon completion of the cleanup as proposed by the DOE. The DOE will leave the excavation area open and apply the suface criteria of 5 pCi/g to this particular vicinity property.

Disposal of Containerized Chemicals

Comment: It is suggested that the specifications for this work might be strengthened by adding waste characterization procedures into Section 2.0 (Scope) of the document. The procedures are those which may be required under 40 CFR 260-268, or others required by the permit held by the Treatment, Storage and/or Disposal Facility.

Response: Procedures required in 40 CFR 260-268 will be referenced in Section 2.0 of the Request for Proposal. In addition, it will be emphasized that all waste characterization procedures which are required by the successful bidder's treatment, storage and/or disposal facilities permit must be satisfied. It will be required that these procedures (if applicable) be presented in the subcontractor's work plan.

Comment: EPA recommends that the specific subcontractor qualifications and experience in handling known and unknown potentially hazardous wastes be defined in the document.

DOCUMENT NUMBER: 4-600-604-102

Response: We are in agreement with the EPA that the Request for Proposal should contain subcontractor qualifications and experience clauses. The appropriate clauses will be added to the document.

Comment: The EPA recommends that the specification require the successful bidder to identify the specific waste disposal facilities which will accept the containerized chemical waste, in the work plan phase.

Response: The specification will be modified to include provision for certification by the subcontractor that the waste disposal facilities meet the requirements when hazardous wastes are involved. The land disposal ban provision of RCRA will also be addressed as part of the subcontractor's work plan.

General

Comment: The EPA review states that there is one(1) deficiency common to the four proposals and that is that plans for onsite handling and storage of radioactive contaminated materials should be developed.

Response: Plans for onsite handling and storage of radioactive contaminated materials are currently being finalized and will be provided under separate cover.

NOV 1 0 1987

Mr. David E. Bedan Missouri Department of Natural Resources Post Office Box 176 Jefferson City, Missouri 65102

Dear Mr. Bedan:

MISSOURI DNR COMMENTS ON INTERIM RESPONSE ACTIONS (IRA'S)

Enclosed is our response to the comments contained in Dr. Frederick A. Brunner's letter of October 26, 1987, regarding the following interim response actions:

- 1. Electric Power Line and Pole Removal
- 2. Overhead Piping/Asbestos Removal
- 3. Army Reserve Area Vicinity Property No. 7
- 4. Disposal of Containerized Chemicals

We anticipate that this will adequately resolve the issues raised and we intend to proceed with these actions in accordance with the enclosure.

If you have any questions, please give me a call.

Sincerely,
ORIGINAL SIGNED BY:
R. R. NELSON

R. R. Nelson Project Manager Weldon Spring Site Remedial Action Project

Enclosure: As stated

cc: B. K. Biggs, USEPA

E. Brown, FLW

w/enclosure

DOCUMENT NUMBER:

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RESPONSES TO MDNR COMMENTS ON INITIAL FOUR (4) IRA PROPOSAL PACKAGES

I. Removal of Overhead Piping and Asbestos Removal

Comment: The DNR states that Missouri has adopted the Federal Clean Air Act standards for asbestos handling and has been delegated responsibility for implementing these standards and that asbestos and piping removal activities are subject to both the Missouri Air Conservation Law and the Missouri Solid Waste Management Law. DNR recommends that we maintain close contact with the Air Pollution Control Program to insure compliance with these standards.

Response: DNR Air Pollution Control Program office will be kept apprised of plans for asbestos removal work at the Weldon Spring Site. The WSSRAP will comply with requirements for disposal of asbestos and other demolition wastes in accordance with the Missouri Solid Waste Management Act.

Comment: The DNR has determined that the overhead piping and asbestos should be handled as a "special waste".

Response: We are proceeding to include the Special Waste Disposal Request form in the Request for Proposal for this work.

Comment: The DNR states that onsite handling of asbestos and other demolition waste <u>may</u> also be subject to Missouri Solid Waste Management Law requirements and requests that we furnish information on the size, design, location of the staging area and the amounts and methods of handling for the materials to be handled in the materials staging area.

Response: Information on handling and staging of the materials will be furnished to the DNR prior to issuing requests for proposals for this work.

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DOCUMENT NUMBER: _____

II. Disposal of Containerized Chemicals

Comment: Define specific levels at which the containerized wastes are considered radioactive.

radioactive.

Response: WSSRAP is developing concentration levels for wastes containing natural uranium for review and acceptance by concerned federal agencies. We will advise the DNR of this determination as it comes available. Until this determination is made we will retain on site containerized chemical materials which contain detectable levels of radioactive materials as determined by our onsite instruments.

Comment: What are removal plans for underground storage tanks on site?

Response: The underground tanks at the WSS have been sampled and found to contain only rainwater with trace amounts of motor fuel. The drainage and removal of the underground tanks is not part of the containerized chemical inventory and removal IRA Scope of Work. They may be removed as part of a subsequent IRA.

Comment: The document appears to be a generic outline for removal of waste. Items such as disposal facilities, transporters, waste characterization procedures, waste treatment procedures, etc. are not detailed.

Response: The IRA documentation represents a request for proposal to be sent to potential removal subcontractors. The responsibility for developing a detailed work plan addressing such items as disposal facilities, transporters, waste characterization procedures, waste treatment procedures, etc. rests with the successful bidder. requirement of the subcontract specification is the development of the subcontractor's work plan which must be approved by the WSSRAP before the work may begin. The DNR will be provided a copy of the subcontractor work plan when it becomes available for review. The WSSRAP office requests that the State provide a timely review (14 calendar days) to avoid delaying the subcontractor's

efforts. The State will be given 2-weeks notice of the interval at which time the work plan will be available for review.

III. Remedial Action on Army Vicinity Property

Comment: DNR states that the interim measure suggested is lacking in detail and should contain information on:

- How the removal is to be conducted?
- How and where excavated material will be contained?
- Health and safety plans for the work.
- Why is DOE recommending only remedial action for this vicinity property?
- When does DOE plan to remove the additional contamination in the other six locations?

Response: The technical requirements, i.e. specifications, drawings, special conditions, etc., did not accompany this IRA package for review as they were incomplete at the time of package submittal. Requirements for removal, containment and storage, and health and safety plans will be included in the Request for Proposal for this work.

The very small quantity (less than two cubic yards) of contaminated material can be removed manually and will not require any significant mechanical equipment. The technical specification developed for this work will provide required direction to the subcontractor.

We plan to excavate and place the contaminated soil in 55 gallon drums. The drums will be sealed, placed and stored in a dry, concrete floored building at the Weldon Spring Site awaiting final disposition. Total volume is anticipated to be less than 5 drums.

No unusual hazards are anticipated for this activity. The contract will require workers to abide by the WSSRAP Environmental, Safety and Health Plan.

IRA-600-605

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INTERIM RESPONSE ACTION (IRA) ADMINISTRATIVE RECORD FILE ARFS FILE # IR-0700

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0700	IRA #09 DEBRIS	CONSOLIDATION
	DOCUMENT NUMBERS	DOCUMENT TITLE
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702	Sampling and A	nalysis Data/Chain of Custody Forms
703	Engineering Ev	valuations/Cost Analysis
	IR-0700-703-1.01	DOCUMENTATION FOR DEBRIS CONSOLIDATION IRA PACKAGES
	IR-0700-703-1.02	ENGINEERING EVALUATION/COST ANALYSIS FOR THE PROPOSED MANAGEMENT OF CONTAMINATED STRUCTURES AT THE WSCP
704	Engineering Ev Document	valuations/Cost Analysis Approval or Decision
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705	Amendments to	Decision Document

IRA-700-701

IRA-700-702

IRA-700-703



Department of Energy

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Oak Ridge Operations
Weldon Spring Site
Remedial Action Project Office
Route 2, Highway 94 South
St. Charles, Missouri 63303

October 16, 1987

Ms. Katherine Biggs
United States Environmental
Protection Agency
Region VII
726 Minnesota Avenue
Kansas City, Kansas 66101

Dear Ms. Biggs:

INTERIM RESPONSE ACTIONS (IRA'S)

Enclosed are six (6) copies of the documentation for the following four (4) Interim Response Actions:

- 1. Dismantling of Building 401
- 2. Dismantling of Building 409
- 3. Removal of PCB Transformers
- 4. Debris Consolidation

In addition, we are sending under separate cover, six (6) copies of the technical specifications and drawings from each of the four (4) proposed bid packages.

It is our intention to have copies of these documents in place in the repositories for public inspection, and to provide public notice of their availability on October 19, 1987. This will initiate the twenty one (21) day comment period.

If you have any questions, please give me a call.

Sincerely,

Rod Nelson

Project Manager
Weldon Spring Site

Remedial Action Project

Enclosures: As stated

cc w/enclosures:
D. Bedan, MDNR

DOCUMENT NUMBER:

The public comment period on this interim remedial action ends on November 9, 1987. Comments may be sent to any of the following:

- Ms. Katherine Biggs
 U. S. Environmental Protection Agency
 Region VII
 726 Minnesota Avenue
 Kansas City, Kansas 66101
- 2. Mr. David Bedan Missouri Department of Natural Resources Post Office Box 176 Jefferson City, Missouri 65102
- 3. Mr. Rodney R. Nelson
 Weldon Spring Site Remedial Action Project
 Route 2, Highway 94 South
 St. Charles, Missouri 63303

DEBRIS CONSOLIDATION

Site Background

The Weldon Spring site is located in St. Charles County, Missouri, about 48 km (30 mi) west of St. Louis. From 1941 to 1944, the U.S. Department of the Army operated the Weldon Spring Ordnance Works at the site for production of trinitrotoluene and dinitrotoluene. In the mid 1950s, a portion of the property was transferred to the U.S. Atomic Energy Commission (AEC), a predecessor of the U.S. Department of Energy (DOE).

From 1957 to 1966, the AEC operated a uranium processing facility at the Weldon Spring site. Impure uranium ore concentrates and some scrap uranium metal were processed at the chemical plant, and thorium-containing materials were also processed on an intermittent basis. Following closure by the AEC, the Army reacquired the chemical plant in 1967 and began converting the facilities to produce herbicides. The buildings were partially decontaminated and some equipment was dismantled. In 1969, prior to becoming operational, the herbicide project was canceled. Since that time, the plant has remained essentially unused and in caretaker status. The Army returned a portion of the Ordnance Works property to the AEC in 1971 but retained control of the chemical plant buildings. In 1984, the Army repaired several of these buildings; decontaminated some of the floors, walls, and ceilings; and removed some contaminated equipment to areas outside of the buildings. In 1985, custody of the chemical plant property was transferred to DOE.

Miscellaneous debris is randomly scattered throughout the 88-ha (220-acre) Weldon Spring site. If consolidated, the debris would cover an area of about 0.4 ha (1 acre). This debris consists of numerous articles ranging from small pieces of trash to abandoned vehicles.

Site Characterization

Site debris consists primarily of wooden and metal materials and includes such items as wooden pallets, railroad ties and hardware, assorted steel piping, lockers, tables, empty metal containers and cabinets, wheelbarrows, drinking fountains, water pumps, forklifts, and assorted trucks and trailers. Much of this material may be radioactively contaminated because a considerable amount of the debris was formerly associated with processing operations at the chemical plant.

Threat to Public Health and the Environment

A detailed characterization of site debris has not yet been performed. It is likely that the debris poses a potential radiological health threat, and uncontrolled releases of potentially hazardous materials from the debris could be occurring. In addition, these materials represent a potential safety hazard to workers on the site.

DOCUMENT NUMBER: 1-700 -701-1.0Z

Response Objectives

The objectives of this response action are as follows:

- 1. Perform a detailed chemical and radiological characterization of the debris to determine appropriate handling and storage procedures in order to minimize exposure hazards and the likelihood of contaminant releases to the environment;
- 2. Isolate the debris from the work environment to ensure the safety of on-site personnel; and
- 3. Store the debris in a manner that minimizes the potential for exposure threats and the release of contaminants to the environment.

Proposed Response Action Alternatives

Interim response actions are designed to ensure the health and safety of on-site personnel and to minimize or preclude off-site releases of contaminants. These actions are limited to those that can be performed under the Comprehensive Environmental Response, Compensation, and Liability Act/Superfund Amendments and Reauthorization Act and remain within the constraints of the Council on Environmental Quality's regulations for the National Environmental Policy Act (i.e., actions will be limited to those that do not have an adverse environmental impact nor limit the choice of reasonable alternatives).

The management of scattered debris at the Weldon Spring site is a two-phased process. The first phase consists of debris characterization and consolidation. The second phase consists of off-site transport to a licensed disposal facility of all material not exceeding chemical and radiological limits for unrestricted release. This proposed interim response action addresses the initial phase of the debris management process.

Alternative response actions identified for debris consolidation are:

- 1. No action;
- 2. Consolidation and storage of the debris at several locations throughout the site; and
- 3. Consolidation and storage of the debris at a single site location.

Analysis of Alternatives

Alternative 1 affords no reduction in the potential safety threat posed by the debris scattered throughout the site. There would be no improvement in environmental conditions at the site if no action were taken. This alternative presents no technical

barriers and costs nothing in the short term. However, the debris is scheduled for eventual disposal and its random placement negatively affects groundskeeping and other ongoing activities, as well as associated costs.

Alternatives 2 and 3 are technically feasible, and both will reduce the potential hazards associated with the debris. However, due to the multiple storage locations, Alternative 2 is less desirable based on both environmental and cost considerations. Alternative 3 is more consistent with DOE's intention to facilitate cleanup of the site. Therefore, following the screening and analysis process for interim response action alternatives, Alternative 3 has been identified as the preferred alternative.

Description of Proposed Action

The proposed interim response action involves the following operations.

- 1. Detailed chemical and radiological characterization of all debris;
- Separation of radioactive debris from nonradioactive debris, with subsequent transport of the radioactive debris to a designated material staging area located at the Weldon Spring site, pending a decision on final disposal; and
- 3. Transport of nonradioactive debris to the designated on-site staging area for interim storage and scrap recovery prior to off-site disposal.

Debris consolidation will be performed in compliance with all applicable regulations and procedures. All debris will be radiologically surveyed. Any material exhibiting surface contamination levels in excess of 1,000 disintegrations per minute (dpm)/100 cm² removable or 5,000 dpm/100 cm² total (fixed plus removable) alpha contamination (U.S. Department of Energy 1987) will be segregated and stored at the designated consolidation area.

Characterization and consolidation of this material would reduce maintenance costs and exposure hazards, as well as the potential for releases of contaminants to the environment, thereby improving environmental conditions at the site.

The total volume of debris scattered throughout the site is estimated to be 9,000 m³ (12,000 yd³). Some of this material may be radioactively contaminated in excess of the above criteria or chemically contaminated in excess of applicable levels for disposal in a sanitary landfill. The radiological and chemical characterization of site debris is part of the proposed action. Therefore, volumes of radioactive and hazardous waste cannot be estimated until this work is completed.

References

U.S. Department of Energy, 1987, U.S. Department of Energy Guidelines for Residual Radioactivity at Formerly Utilized Sites Remedial Action Program and Remote Surplus Facilities Management Program Sites (Revision 2, March).

MORRISON-KNUDSEN ENGINEERS, INC.

WSSRA PROJECT

DRAWING NO

DE-ACO5 - 860R21548

BO HOWARD ST. SAN FRANCISCO. CA 94105

MKE DOCUMENT NO. 5121-C: DW - D - 01 - 0110 - 01

DEPARTMENT

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ENERGY

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CP - DEBRIS REMOVAL

5121E -CP -513

DOE/OR/21548-159

Engineering Evaluation/Cost Analysis for the Proposed Management of Contaminated Structures at the Weldon Spring Chemical Plant

May 1991

See Document in 12A 18

DOCUMENT NUMBER: T-100-703-1.01(R)



U.S. Department of Energy Oak Ridge Operations Office Weldon Spring Site Remedial Action Project

IRA-700-704



UNITED STATES ENVIRONMENTAL PROTECTION AGENCY

REGION VII 726 MINNESOTA AVENUE KANSAS CITY, KANSAS 66101

NOV 18 1987

OFFICE OF THE REGIONAL ADMINISTRATOR

Mr. Rodney R. Nelson
U.S. Department of Energy
Weldon Spring Site Remedial
Action Project/Office
Route 2, Highway 94, South
St. Charles, Missouri 63303

Dear Mr. Nelson:

We have reviewed the Department of Energy's (DOE) proposals for the following four interim response actions:

- Dismantling of Building 401,
- Dismantling of Building 409,
- Removal of PCB Transformers, and
- · Debris Consolidation.

Our comments on these proposals were sent to you earlier. You were also provided comments by the Missouri Department of Natural Resources (MDNR). No comments from the public were directed to the Environmental Protection Agency (EPA) and according to our records, there has been no public comment directed to MDNR or DOE.

We are in agreement these actions should proceed to ensure worker safety and reduce the further release of contaminants from this site. The EPA hereby approves these actions under the condition that the comments earlier provided by EPA and MDNR are adequately addressed. The MDNR has notified me they also concur with these actions. Please provide copies of any summary reports for these actions to EPA and MDNR.

We also received copies of the interim response action for construction of the Ash Pond Dike. We will provide any comments on this proposed action within the agreed upon 21-day comment period. We are most pleased to see that activities are underway to stabilize the site and reduce contaminant release.

Sincerely yours

Morris Kay

Regional Administrator

DOCUMENT NUMBER: 1-700-704-1.01

cc: Dr. Fred Brunner, MDNR

11. 23-87 Tapit milon 11-20-87 DEC 0 9 1987

Ms. B. Katherine Biggs
United States Environmental
Protection Agency
Region VII
726 Minnesota Avenue
Kansas City, Kansas 66101

Dear Ms. Biggs:

USEPA COMMENTS ON INTERIM RESPONSE ACTIONS (IRA'S)

Enclosed is our response to the comments contained in your letter of November 9, 1987, regarding the following interim response actions:

- 1. Dismantling of Building #401
- 2. Dismantling of Building #409
- 3. Removal of PCB Transformers
- 4. Debris Consolidation

We anticipate that this will adequately resolve the issues raised. We intend to proceed with action on these items in accordance with the enclosure.

If you have any questions, please give me a call.

Sincerely,

ORIGINAL SIGNED BY:

Rod Nelson Project Manager Weldon Spring Site Remedial Action Project

Enclosure:
As stated

cc: Dave Bedan, MDNR

DOCUMENT NUMBER: I-700-704-1.02

PEER: JCoyne: x41: mw: 12/04/87: (c: EPA-IRA'.Ltr.)

CONCURRENCES

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RESPONSIVENESS SUMMARY

B. Katherine Biggs letter to Rodney R. Nelson, dated November 9, 1987 re:

Interim Response Actions

- 1. Dismantling of Building #401
- Dismantling of Building #409
- 3. Removal of PCB Transformers
- 4. Debris Consolidation

General

Comment: Generally, a more thorough analysis and screening of response alternatives would be appropriate.

Response: This comment was reviewed with the EPA (telecon from Rod Nelson to Dan Wall dated 11-17-87). The EPA agreed that while additional analysis and screening is not required for the four (4) IRA proposals addressed herein, future proposals such as the Ash Pond Isolation Dike will present a more thorough analysis of response alternatives.

Comment: The documents do not contain sufficient detail of the work to be done to stand alone without the support of the technical specifications and drawings.

Response: Technical specifications and drawings will continue to accompany the IRA proposal packages submitted for review.

Building Demolition

Comment: Specifics of handling, storage, and ultimate disposal of radioactively contaminated waste should be presented.

Response: Radioactively contaminated waste from demolition of Buildings #401 and #409 will be segregated and stored on-site in a dry, concrete floored building, Building #434 and/or Building #406. Ultimate disposal will be in accordance with the RI Plan/EIS. Specifics of handling this waste will be covered in the Contractor's operational work plan which will integrate the specification and drawings, the WSSRAP Construction Safety and Health Management Program, applicable WSSRAP Standard Operating Procedures and Plans along with the subcontractor dismantling plan. This work plan will be finalized prior to the Subcontractor(s) starting demolition work.

response, txtsheil

Comment: What guidelines will be used by the demolition subcontractor to determine the hazard potential of unknown materials encountered in the work?

Response: Subcontractor personnel who will work on the site will be required to undergo a minimum of 40 hours of initial instruction in hazardous waste operations prior to starting work on site in accordance with 29CFR1910.120. In addition Subcontractor personnel will receive indoctrination training in the known hazards in the work area prior to start of work in accordance with the WSSRAP Construction Safety and Health Management Program and Special Conditions requirement of the subcontract. Unknown (unidentified or unmarked) chemical substances encountered in the work shall be considered potential hazards and shall be reported to the Contractor in accordance with the requirements of the specifications.

The Contractor will also provide health physics, construction safety and industrial hygiene surveillance on a routine basis during all stages of the work. This will include inspections of all work areas to identify potential hazards. Where required, the Contractor will collect bulk samples to identify any unknown or suspected substances. The Contractor will also perform air monitoring, as necessary and prudent, to assess exposure levels of hazardous substances in the workplace.

Comment: The responsibility for determining whether a pollution condition has or will be created should be clearly specified.

Response: The WSSRAP Construction Safety and Health Management Program which is an integral part of site subcontracts assigns responsibility for the identification of potential pollution (environmental) conditions to the Project Management Contractor. The Subcontractor is contractually required to comply with the requirements of the Clean Air Act and the Clean Water Act.

Comment: The specification does not state the health and safety requirements for the subcontractor.

Response: Subcontractor health and safety requirements are defined in the Special Conditions to the subcontract. The Special Conditions bind the Subcontractor to compliance with the WSSRAP Construction Safety and Health Management Program and all applicable Federal, State, and local health and safety regulations and standards listed therein. The Special Conditions are a supplement to the General Conditions and General Provisions which also contain basic health and safety requirements.

PCB Transformer Removal

Comment: In this case, more detail in the site characterization section of the text would be appropriate. For instance, the PCB transformers are categorized as those containing PCBs at concentrations greater than 500 ppm. It may be somewhat misleading not to indicate in the text that the concentrations in these transformers are in excess of 350,000 ppm.

Response: The final subcontract work package includes a table on the subcontract drawings listing each electrical component in the scope of work. This table includes the PCB concentration and volume capacity, in gallons, of each electrical component.

Comment: Disposal facilities under consideration for receipt of these wastes must provide certification that they meet the Superfund offsite policy.

Response: The Work Plan specified in Section 1.2A of Specification Section 02090 includes provision for meeting all requirements of 40CFR761. The Subcontractor's Work Plan in section 1.4A will be required to contain certification that the facilities selected for disposal of the waste material (1) have received written approval from the U.S. Environmental Protection Agency as required under 40CFR Part 761.70 or 761.75, as applicable, and (2) are not under a state or federal compliance order under CERCLA or RCRA.

Debris Consolidation

Comment: It is stated in the description of the response action that one of the response objectives is to "Perform a detailed chemical and radiological characterization of the debris...". The description of the response action and specifications document contain no guidelines, references or information which would allow the Subcontractor to complete this objective.

Response: This objective is to be completed by the Contractor and does not require any special activity by the Subcontractor. Radiological guidelines to be used by the Contractor in performing this characterization are as defined in Draft DOE Order 5480.11 and applicable WSS operating procedures. All debris will be visually inspected for potential chemical contamination. Where chemical contamination is observed or suspected, sampling and analyses will be performed to identify the characteristics of the chemical.

Comment: More specifics regarding the handling, storage and

ultimate disposal of radioactive contaminated wastes are

needed.

Response: Specifics of handling the radiologically contaminated

debris will be finalized upon submittal of the

Subcontractor's work plan. That plan will be integrated with applicable WSSRAP Standard Operating Procedures and Plans, the WSSRAP Construction Safety and Health Management Program, and the specifications and drawings. The debris will be consolidated for temporary storage in a materials staging area. Details of the materials staging area will be presented in a separate IRA under preparation. Ultimate disposal of radioactive contaminated debris waste will be in accordance with the RI Plan/EIS.



Department of Energy

Oak Ridge Operations
Weldon Spring Site
Remedial Action Project Office
Route 2, Highway 94 South
St. Charles, Missouri 63303

December 21, 1987

Ms. B. Katherine Biggs
United States Environmental
Protection Agency
Region VII
726 Minnesota Avenue
Kansas City, Kansas 66101

Dear Ms. Biggs:

MDNR COMMENTS ON INTERIM RESPONSE ACTIONS (IRA'S)

Enclosed is our responsiveness summary for the comments contained in Dave Bedan's letter of November 12, 1987, regarding the following interim response actions:

- Dismantling of Building #401
- 2. Dismantling of Building #409
- 3. Removal of PCB Transformers
- 4. Debris Consolidation

We anticipate that this will adequately resolve the issues raised. We intend to proceed with action on these items in accordance with the enclosure.

If you have any questions, please give me a call.

Sincerely,

Rod Nelson

Project Manager

Weldon Spring Site

Remedial Action Project

Enclosure: As stated

cc: Dave Bedan, MDNR, w/enclosure Jack Hammond, MK-F, w/o enclosure

DOCUMENT NUMBER: 2-700-704-1.03

RESPONSIVENESS SUMMARY

B. Katherine Biggs letter to Rodney R. Nelson, dated 11-13-87 re: MoDNR comments on:

Interim Response Actions

- 1. Dismantling and Disposal of Building #401
- 2. Dismantling and Disposal of Building #409
- 3. Removal of PCB Transformers
- 4. Debris Consolidation

1. Dismantling and Disposal of Buildings #401 and #409

Comment: The DOE and its Contractors should develop and maintain close contact with the Missouri Air Pollution Control Program to assure compliance with Missouri Air Conservation Law and Missouri Solid Waste Management Law in carrying out these activities.

Response: The DOE and its subcontractor(s) will continue to keep the DNR Air Pollution Control Program office apprised of plans for work at the site involving removal, handling, storage, and/or disposal of asbestos materials.

Comment: Missouri Solid Waste Management Law requires demolition waste to be disposed of in a permitted sanitary or demolition landfill. Asbestos waste must be disposed of in a permitted sanitary landfill.

Response: The Specifications for this work will require that asbestos and other demolition debris be disposed of in accordance with the requirements of the Missouri Solid Waste Management Law.

Comment: DNR maintains that because of the special concerns relating to the volume of waste and to the possible contamination of the asbestos and the other demolition material with hazardous wastes or radioactive wastes, these materials should be handled as "special wastes".

Response: The DOE concurs that there are special circumstances that require handling of asbestos as "special waste". Specifications for the asbestos subcontracts contain this provision.

The pending subcontracts contain the "special waste" forms which will be included in the

DOCUMENT NUMBER: 1-700-704-1.03

subcontract work packages as matter of comity. Should subcontract efforts, cost or progress on these IRA's be impacted by this provision, the DOE will revisit this issue with the MDNR.

Comment: The DNR cannot approve the disposal of the asbestos and other demolition wastes until a procedure is in place to assure us that no radioactive or hazardous materials are being disposed of in Missouri solid waste landfills.

Response: Release standards are in place for controlling release of the rubble off site. Radiological survey and release plans will be developed for each work package involving removal and off-site disposal of materials to insure compliance with the standards.

Comment: DOE should provide justification for its policy to dispose of all non-radioactive building waste off site.

Response: The DOE policy is based on volume reduction and cost effectiveness. By disposal of nonradiological material in a sanitary or demolition landfill, there is a reduction in the amount of material (Volume Reduction) that will be encapsulated in any disposal cell. Secondly, costs for on-site disposal cells are high in comparison to disposal in sanitary or demolition landfills. Also, as an aside to the technical and cost effectiveness issues, the DOE currently has funding available. The site is still to be fully characterized and to delay demolition and disposal of clean materials would not allow these funds to be utilized and also would lead to overall slipping of the schedule.

2. Removal of PCB Transformers

Comment: MDNR recommends that if Alternative #5 is used, during the "flushing" process care should be taken to contain any spilled material. Also, "flushing" should be continued until PCB levels are less than 2 ppm, if transformer and switch carcasses are going to be disposed into a permitted sanitary landfill.

Response: The subcontract specifications for this interim response action include spill control provisions for draining and flushing operations. Spill control pans are specified to collect any spilled liquids. PCB transformers and other electrical

equipment which have been drained and flushed, as stated in the specifications, will be disposed of at an EPA approved PCB disposal facility, not a sanitary landfill.

Comment: If the PCB liquids are being transported to a disposal facility within Missouri, a licensed hazardous waste transporter must be used. If the PCBs are being transported to an out-of-state facility MDNR recommends that a licensed transporter be used although it is not a requirement.

Response: The specifications state that the transporter of the PCB liquids-and drained electrical equipment shall be licensed.

Comment: In the preamble to 40 CFR 761, unless otherwise tested, all dielectric transformers are assumed to contain 50-500 ppm PCB, therefore untested transformers (22, 32, and 45) should be "flushed" with other transformers.

Response: The three transformers which have not been sampled for PCBs will be treated as PCB-contaminated units unless future sampling is performed to otherwise classify them as non-PCB transformers or PCB transformers. Irrespective of the classification, these units will be drained and flushed on site, unless the disposal facility intends to incinerate them as intact units, as indicated in the subcontract specifications.

Comment: External pad, poles, and adjacent areas should be tested to determine if PCB contamination exists.

Response: Additional sampling for PCB contamination, in areas from which the PCB-containing transformers and other electrical components are to be removed, is planned as part of future chemical characterization activities at the site.

3. <u>Debris Consolidation</u>

Comment: The storage of solid waste on site may be subject to the requirements of the Missouri Solid Waste Management Law. Please contact the Missouri Waste Management Program for assistance in determining whether these requirements apply.

Response: An interim response action proposal is being prepared which will present plans for materials staging and interim storage of solid waste on site. We will contact the Missouri Waste Management Program Office for assistance in determining applicability of the Missouri Solid Waste Management Law to this work.

IRA-700-705

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INTERIM RESPONSE ACTION (IRA) ADMINISTRATIVE RECORD FILE ARFS FILE # IR-0800

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Department of Energy

Oak Ridge Operations
Weldon Spring Site
Remedial Action Project Office
Route 2, Highway 94 South
St. Charles, Missouri 63303

October 16, 1987

Ms. Katherine Biggs
United States Environmental
Protection Agency
Region VII
726 Minnesota Avenue
Kansas City, Kansas 66101

Dear Ms. Biggs:

INTERIM RESPONSE ACTIONS (IRA'S)

Enclosed are six (6) copies of the documentation for the following four (4) Interim Response Actions:

- Dismantling of Building 401
- 2. Dismantling of Building 409
- 3. Removal of PCB Transformers
- 4. Debris Consolidation

In addition, we are sending under separate cover, six (6) copies of the technical specifications and drawings from each of the four (4) proposed bid packages.

It is our intention to have copies of these documents in place in the repositories for public inspection, and to provide public notice of their availability on October 19, 1987. This will initiate the twenty one (21) day comment period.

If you have any questions, please give me a call.

Sincerely,

Rod Nelson

Project Manager Weldon Spring Site

Remedial Action Project

Enclosures: As stated

cc w/enclosures:
D. Bedan, MDNR

The public comment period on this interim remedial action ends on November 9, 1987. Comments may be sent to any of the following:

- Ms. Katherine Biggs
 U. S. Environmental Protection Agency
 Region VII
 726 Minnesota Avenue
 Kansas City, Kansas 66101
- 2. Mr. David Bedan Missouri Department of Natural Resources Post Office Box 176 Jefferson City, Missouri 65102
- 3. Mr. Rodney R. Nelson
 Weldon Spring Site Remedial Action Project
 Route 2, Highway 94 South
 St. Charles, Missouri 63303

DISMANTLING OF BUILDING 409

Site Background

The Weldon Spring site is located in St. Charles County, Missouri, about 48 km (30 mi) west of St. Louis. From 1941 to 1944, the U.S. Department of the Army operated the Weldon Spring Ordnance Works at the site for production of trinitrotoluene and dinitrotoluene. In the mid 1950s, a portion of the property was transferred to the U.S. Atomic Energy Commission (AEC), a predecessor of the U.S. Department of Energy (DOE).

From 1957 to 1966, the AEC operated a uranium processing facility at the Weldon Spring site. Impure uranium ore concentrates and some scrap uranium metal were processed at the chemical plant, and thorium-containing materials were also processed on an intermittent basis. Following closure by the AEC, the Army reacquired the chemical plant in 1967 and began converting the facilities to produce herbicides. The buildings were partially decontaminated and some equipment was dismantled. In 1969, prior to becoming operational, the herbicide project was canceled. Since that time, the plant has remained essentially unused and in caretaker status. The Army returned a portion of the Ordnance Works property to the AEC in 1971 but retained control of the chemical plant buildings. In 1984, the Army repaired several of these buildings; decontaminated some of the floors, walls, and ceilings; and removed some contaminated equipment to areas outside of the buildings. In 1985, custody of the chemical plant property was transferred to DOE.

Building 409 was used as an administrative office building during the operational period of the chemical plant. It is located in the eastern portion of the Weldon Spring complex (Fig. 1) about 90 m (300 ft) southwest of the plant, from which it is separated by a former laboratory building. Since the termination of site activities, Building 409 has remained unoccupied.

Site Characterization

The administration building is a two-story structure designed to house 250 persons. It measures $59 \text{ m} \times 29 \text{ m} \times 10 \text{ m}$ (192 ft \times 96 ft \times 32 ft) and encloses about 3,500 m² (38,000 ft²) of floor space. Office furniture and support equipment have been removed from the building but piping remains intact. Both floors are partitioned into numerous small work areas, including individual offices, telephone/teletype rooms, an equipment room, a vault, and a lobby.

The framework of Building 409 consists of structural steel and concrete. Interior walls are paneled with sheet metal, and exterior walls are composed of masonry blocks. The floor is tiled, and the roof is constructed of metal decking, insulation, and a built-up layer of tar and gravel.

The building was serviced by steam, water, and ethylene glycol lines, and these lines are insulated with asbestos-containing material. The bulk of the insulating material

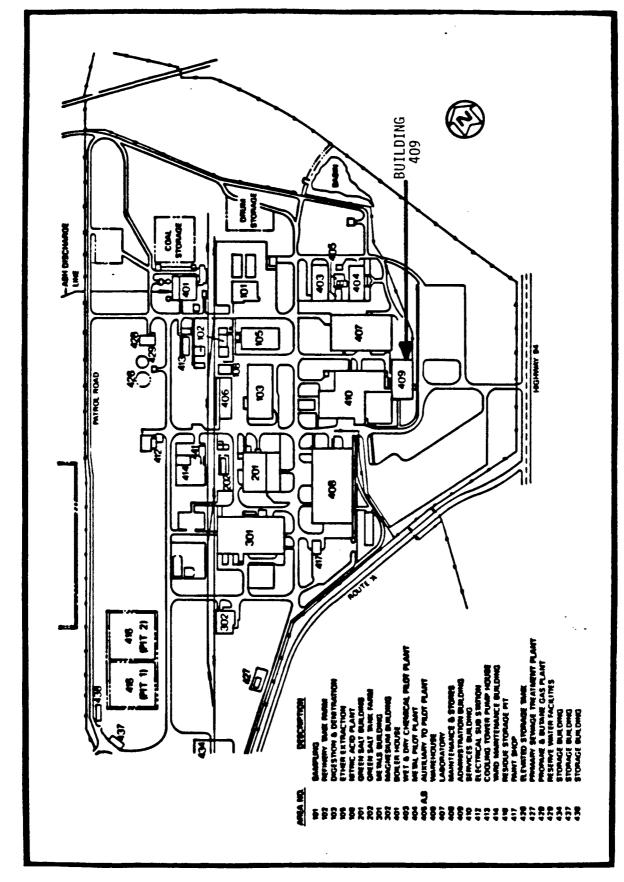


FIGURE 1 Location of Major Structures at the Weldon Spring Site (Source: Bechtel National 1986)

is located in ceiling space between the first and second floors. Although most of the insulation is currently in good condition, i.e., intact and contained by exterior wrappings, there are several localized areas in which friable asbestos has loosened.

A preliminary survey for polychlorinated biphenyls (PCBs) indicated the presence of detectable concentrations of PCBs in the enclosed corridor immediately west of Building 409. Swipe sampling of a 0.6 m \times 0.6 m (2 ft \times 2 ft) section of the concrete floor surface at this location indicated a PCB level of approximately 100 µg/100 cm². Further sampling will be performed to determine the presence and extent of PCB contamination in Building 409.

Results of a radiological survey of the building, presented in Table 1, indicate that contamination levels of the building's floors, walls, and equipment are below the release limits for unrestricted use. These limits are 1,000 disintegrations per minute (dpm)/100 cm² removable alpha contamination and 5,000 dpm/100 cm² total (fixed plus removable) alpha contamination (U.S. Department of Energy 1987). Since many of the surface areas were wet and muddy when surveyed (resulting in attenuation of alpha particles), beta-gamma measurements were used to estimate the level of alpha activity. Use of beta-gamma measurements will tend to overestimate the actual amount of alpha activity for uranium contamination. This will result in a conservative estimate of the amount of material contaminated in excess of DOE limits for unrestricted release.

Results of an initial survey indicated that three rooms in the building exhibit total beta-gamma contamination levels in excess of the DOE limit for removable alpha contamination (i.e., 1,000 dpm/100 cm²) but below the limit for total alpha contamination (i.e., 5,000 dpm/100 cm²). Subsequent sampling of the rooms indicated that the contamination is not removable. Thus, from a radiological standpoint, all of the material inside the building can be released for unrestricted use.

Results of the roof survey given in Table 1 indicate the presence of radioactive contaminants in the tar and gravel layer. The contamination resulted from airborne releases of radioactive dust (containing primarily uranium and its decay products) during the operational period of the chemical plant. Analysis of two samples from the tar/gravel layer identified uranium-238 concentrations of 35.3 and 107.7 pCi/g. (Two samples of the underlying insulation layer had only background levels of uranium.) These measurements indicate that the contamination levels in the built-up tar/gravel roofing material will require that this material be disposed of as radioactive waste.

Threat to Public Health and the Environment

Since its abandonment, the administration building has undergone substantial deterioration. Floor tiles have loosened and begun to break apart, and the roof has weakened to the extent that it leaks badly during rainstorms. Postponed dismantling of the building will result in increased occupational hazards to workers on-site. As an example of the hazards associated with this building, it was necessary to safety-rope a recent survey team during their roof characterization efforts because of unstable structural conditions.

TABLE 1 Summary of Radiological Survey Results for Building 409

	Measured Total (Fixed plus Removable) Beta-Gamma Activity			
Location	Range (dpm/100 cm ²)	Average (dpm/100 cm ²)		
First-story floor	0-1,037	257		
First-story walls	0-921	253		
Second-story floor	0-1,037	212		
Second story walls	0-921	297		
Roof	2,112-25,147	9,403		

Source: Data from MK-Ferguson and Jacobs (1987).

The potential for an asbestos-related health hazard will also increase if the deterioration of Building 409 remains unchecked because the protective coverings that isolate the asbestos from the environment will deteriorate further. In addition, possible PCB contamination of flooring within the building and the radiological contamination of the roof's tar/gravel layer may pose potential exposure hazards to on-site personnel.

Response Objectives

The objectives of this response action are as follows:

- Reduction of the potential health hazard due to asbestos exposure from asbestos-containing material in Building 409;
- 2. Reduction of the potential health hazard due to radiation exposure associated with uranium contamination of the roof's tar/gravel layer;
- 3. Reduction of the potential health hazard due to exposure to floor surfaces that may be contaminated with PCBs; and
- 4. Removal of the potential safety hazard to on-site personnel due to deterioration of the building.

Proposed Response Action Alternatives

Interim response actions are designed to ensure the health and safety of on-site personnel and to minimize or preclude off-site releases of contamination. These actions are limited to those that can be performed under the Comprehensive Environmental Response, Compensation, and Liability Act/Superfund Amendments and Reauthorization Act and remain within the constraints of the Council on Environmental Quality's regulations for the National Environmental Policy Act (i.e., limited to those that do not have an adverse environmental impact nor limit the choice of reasonable alternatives).

Alternative response actions identified for Building 409 are:

- 1. No action;
- 2. Removal of the tar/gravel roof layer and all PCB-contaminated materials for on-site storage, in-situ stabilization of asbestos-containing material, and repair of the building's structural deficiencies;
- Removal of the tar/gravel roof layer for on-site storage, removal
 of all PCB-contaminated material for off-site treatment/disposal,
 removal of asbestos-containing material for off-site disposal, and
 repair of the building's structural deficiencies;
- 4. Dismantlement of Building 409, with on-site disposal of all material except that which exceeds the radiological criteria for unrestricted release (i.e., the tar/gravel roof layer, which will be stored on-site), and transport of all PCB-contaminated material to an off-site treatment/disposal facility; or
- 5. Dismantlement of Building 409, with off-site disposal of all material except that which exceeds the radiological criteria for unrestricted release (i.e., the tar/gravel roof layer, which will be stored on-site), transport of all PCB-contaminated material to an off-site treatment/disposal facility; transport of the remainder of the waste to a sanitary landfill for disposal; and reclamation of reusable materials that are not radiologically or chemically contaminated for salvage or on-site use.

Analysis of Alternatives

Alternative 1 affords no reduction in the potential health threat posed by the radioactive material and the PCB- and asbestos-contaminated material associated with Building 409. There would be no improvement in environmental conditions at the site if no action were taken. This alternative presents no technical barriers and costs nothing in the short term. However, the building is scheduled for eventual demolition. The costs associated with deferred dismantlement would be higher than those for dismantlement at

the current time, due to periodic maintenance activities required until future dismantlement. Most importantly, Alternative 1 is effectively precluded by institutional factors related to the community's strong desire for timely response actions at the Weldon Spring site.

Alternatives 2 through 5 are all technically feasible. Each of these alternatives reduces the potential hazards associated with exposure to PCBs, asbestos, and radiation. Implementation of Alternatives 2 and 3 would be more expensive in the long term, due to the need to repair structural deficiencies and perform future maintenance activities at Building 409. In addition, Alternatives 2 and 3 do not fully address the public sentiment for expedited response at the site. Even though Alternative 4 would be less expensive than Alternative 5, it is not consistent with DOE's intention to dispose of all nonradioactive waste off-site. Therefore, following the screening and analysis process for interim response action alternatives, Alternative 5 has been identified as the preferred alternative.

Description of Proposed Action

The proposed interim response action involves demolition of Building 409 with off-site disposal of all material meeting the criteria for unrestricted radiological release, including PCB-contaminated and asbestos-containing material. The response action will include the following operations.

- 1. Removal of all PCB-contaminated material for transport to a licensed off-site treatment/disposal facility;
- Removal of the tar/gravel roofing material to a depth of approximately 5 cm (2 in.) for controlled on-site storage in a dry, concrete-floored building currently located at the Weldon Spring site; and
- 3. Removal of all asbestos-containing material and dismantlement of the remainder of Building 409, followed by scrap recovery and off-site disposal of the resultant waste material at a licensed sanitary landfill in Missouri.

The foundation and below-grade piping are not part of this action and will be addressed at a later date.

Under the proposed action, Building 409 will be dismantled in full compliance with all applicable regulations and procedures, with off-site disposal of all nonradioacative material (material that exceeds the radiological criteria for unrestricted release will be stored on-site). A representative fraction of material to be disposed of off-site will be radiologically surveyed prior to release. Asbestos removal, removal of the radioactively contaminated roof layer, and PCB decontamination/removal operations will also be performed in accordance with all applicable requirements. This compliance will ensure protection of the safety and health of on-site workers as well as limiting off-site releases of contaminants.

Demolition of Building 401 will proceed in accordance with all safety requirements and practices. Demolition at this time will preclude the associated adverse impacts on health and the environment resulting from continued deterioration of the building. Removal of the demolition debris will be consistent with DOE's goal of removing all nonradioactive waste from the site.

The total waste volume associated with this proposed action is estimated to be approximately 2,000 m³ (2,700 yd³), of which about 100 m³ (130 yd³) is asbestoscontaminated insulation material and 90 m³ (120 yd³) is radioactively contaminated roofing material (the approximate volume of PCB-contaminated material has not yet been determined). The nonradioactive waste will be shipped to a licensed sanitary landfill in Missouri, requiring an estimated 190 truckloads.

References

- Bechtel National, Inc., 1986, Characterization Plan for the Weldon Spring Chemical Plant, DOE/OR/20722-85, prepared by Advanced Technology Division for U.S. Department of Energy, Oak Ridge Operations Office, Oak Ridge, Tenn. (Draft, Feb.).
- MK-Ferguson Company and Jacobs Engineering Group, 1987, Radiological Survey of Building 409, Draft, prepared for U.S. Department of Energy, Oak Ridge Operations Office, Oak Ridge, Tenn. (Sept.).
- U.S. Department of Energy, 1987, U.S. Department of Energy Guidelines for Residual Radioactivity at Formerly Utilized Sites Remedial Action Program and Remote Surplus Facilities Management Program Sites (Revision 2, March).



UNITED STATES ENVIRONMENTAL PROTECTION AGENCY

REGION VII 726 MINNESOTA AVENUE KANSAS CITY, KANSAS 66101

NOV 18 1987

OFFICE OF THE REGIONAL ADMINISTRATOR

Mr. Rodney R. Nelson
U.S. Department of Energy
Weldon Spring Site Remedial
Action Project/Office
Route 2, Highway 94, South
St. Charles, Missouri 63303

Dear Mr. Nelson:

We have reviewed the Department of Energy's (DOE) proposals for the following four interim response actions:

- pismantling of Building 401,
- Dismantling of Building 409,
- · Removal of PCB Transformers, and
- Debris Consolidation.

Our comments on these proposals were sent to you earlier. You were also provided comments by the Missouri Department of Natural Resources (MDNR). No comments from the public were directed to the Environmental Protection Agency (EPA) and according to our records, there has been no public comment directed to MDNR or DOE.

We are in agreement these actions should proceed to ensure worker safety and reduce the further release of contaminants from this site. The EPA hereby approves these actions under the condition that the comments earlier provided by EPA and MDNR are adequately addressed. The MDNR has notified me they also concur with these actions. Please provide copies of any summary reports for these actions to EPA and MDNR.

We also received copies of the interim response action for construction of the Ash Pond Dike. We will provide any comments on this proposed action within the agreed upon 21-day comment period. We are most pleased to see that activities are underway to stabilize the site and reduce contaminant release.

Sincerely yours

Morris Kay

Regional Administrator

cc: Dr. Fred Brunner, MDNR

11. =3-87 Fark mila-11-20-87

CONCURRENCES

DEC 0 9 1987

Ms. B. Katherine Biggs
United States Environmental
Protection Agency
Region VII
726 Minnesota Avenue
Kansas City, Kansas 66101

Dear Ms. Biggs:

USEPA COMMENTS ON INTERIM RESPONSE ACTIONS (IRA'S)

Enclosed is our response to the comments contained in your letter of November 9, 1987, regarding the following interim response actions:

- Dismantling of Building #401
- 2. Dismantling of Building #409
- 3. Removal of PCB Transformers
- Debris Consolidation

We anticipate that this will adequately resolve the issues raised. We intend to proceed with action on these items in accordance with the enclosure.

If you have any questions, please give me a call.

Sincerely,

ORIGINAL SIGNED BY:

Rod Nelson Project Manager Weldon Spring Site Remedial Action Project

Enclosure: As stated

cc: Dave Bedan, MDNR

PEER: JCoyne: x41: mw: 12/04/87: (c: EPA-IRA'.Ltr.)

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RESPONSIVENESS SUMMARY

B. Katherine Biggs letter to Rodney R. Nelson, dated November 9, 1987 re:

Interim Response Actions

- Dismantling of Building #401
- Dismantling of Building #409
- 3. Removal of PCB Transformers
- 4. Debris Consolidation

General

Comment: Generally, a more thorough analysis and screening of response alternatives would be appropriate.

Response: This comment was reviewed with the EPA (telecon from Rod Nelson to Dan Wall dated 11-17-87). The EPA agreed that while additional analysis and screening is not required for the four (4) IRA proposals addressed herein, future proposals such as the Ash Pond Isolation Dike will present a more thorough analysis of response alternatives.

Comment: The documents do not contain sufficient detail of the work to be done to stand alone without the support of the technical specifications and drawings.

Response: Technical specifications and drawings will continue to accompany the IRA proposal packages submitted for review.

Building Demolition

Comment: Specifics of handling, storage, and ultimate disposal of radioactively contaminated waste should be presented.

Response: Radioactively contaminated waste from demolition of Buildings #401 and #409 will be segregated and stored on-site in a dry, concrete floored building, Building #434 and/or Building #406. Ultimate disposal will be in accordance with the RI Plan/EIS. Specifics of handling this waste will be covered in the Contractor's operational work plan which will integrate the specification and drawings, the WSSRAP Construction Safety and Health Management Program, applicable WSSRAP Standard Operating Procedures and Plans along with the subcontractor dismantling plan. This work plan will be finalized prior to the Subcontractor(s) starting demolition work.

response, txtsheil

Comment: What guidelines will be used by the demolition subcontractor to determine the hazard potential of unknown materials encountered in the work?

Response: Subcontractor personnel who will work on the site will be required to undergo a minimum of 40 hours of initial instruction in hazardous waste operations prior to starting work on site in accordance with 29CFR1910.120. In addition Subcontractor personnel will receive indoctrination training in the known hazards in the work area prior to start of work in accordance with the WSSRAP Construction Safety and Health Management Program and Special Conditions requirement of the subcontract. Unknown (unidentified or unmarked) chemical substances encountered in the work shall be considered potential hazards and shall be reported to the Contractor in accordance with the requirements of the specifications.

The Contractor will also provide health physics, construction safety and industrial hygiene surveillance on a routine basis during all stages of the work. This will include inspections of all work areas to identify potential hazards. Where required, the Contractor will collect bulk samples to identify any unknown or suspected substances. The Contractor will also perform air monitoring, as necessary and prudent, to assess exposure levels of hazardous substances in the workplace.

Comment: The responsibility for determining whether a pollution condition has or will be created should be clearly specified.

Response: The WSSRAP Construction Safety and Health Management Program which is an integral part of site subcontracts assigns responsibility for the identification of potential pollution (environmental) conditions to the Project Management Contractor. The Subcontractor is contractually required to comply with the requirements of the Clean Air Act and the Clean Water Act.

Comment: The specification does not state the health and safety requirements for the subcontractor.

Response: Subcontractor health and safety requirements are defined in the Special Conditions to the subcontract. The Special Conditions bind the Subcontractor to compliance with the WSSRAP Construction Safety and Health Management Program and all applicable Federal, State, and local health and safety regulations and standards listed therein. The Special Conditions are a supplement to the General Conditions and General Provisions which also contain basic health and safety requirements.

PCB Transformer Removal

Comment: In this case, more detail in the site characterization section of the text would be appropriate. For instance, the PCB transformers are categorized as those containing PCBs at concentrations greater than 500 ppm. It may be somewhat misleading not to indicate in the text that the concentrations in these transformers are in excess of 350,000 ppm.

Response: The final subcontract work package includes a table on the subcontract drawings listing each electrical component in the scope of work. This table includes the PCB concentration and volume capacity, in gallons, of each electrical component.

Comment: Disposal facilities under consideration for receipt of these wastes must provide certification that they meet the Superfund offsite policy.

Response: The Work Plan specified in Section 1.2A of Specification Section 02090 includes provision for meeting all requirements of 40CFR761. The Subcontractor's Work Plan in section 1.4A will be required to contain certification that the facilities selected for disposal of the waste material (1) have received written approval from the U.S. Environmental Protection Agency as required under 40CFR Part 761.70 or 761.75, as applicable, and (2) are not under a state or federal compliance order under CERCLA or RCRA.

<u>Debris Consolidation</u>

Comment: It is stated in the description of the response action that one of the response objectives is to "Perform a detailed chemical and radiological characterization of the debris...". The description of the response action and specifications document contain no guidelines, references or information which would allow the Subcontractor to complete this objective.

Response: This objective is to be completed by the Contractor and does not require any special activity by the Subcontractor. Radiological guidelines to be used by the Contractor in performing this characterization are as defined in Draft DOE Order 5480.11 and applicable WSS operating procedures. All debris will be visually inspected for potential chemical contamination. Where chemical contamination is observed or suspected, sampling and analyses will be performed to identify the characteristics of the chemical.

Comment: More specifics regarding the handling, storage and

ultimate disposal of radioactive contaminated wastes are

needed.

Response: Specifics of handling the radiologically contaminated

debris will be finalized upon submittal of the

Subcontractor's work plan. That plan will be integrated with applicable WSSRAP Standard Operating Procedures and Plans, the WSSRAP Construction Safety and Health Management Program, and the specifications and drawings. The debris will be consolidated for temporary storage in a materials staging area. Details of the materials staging area will be presented in a separate IRA under preparation. Ultimate disposal of radioactive contaminated debris waste will be

in accordance with the RI Plan/EIS.

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3589-87-I-DOE-241



Department of Energy

Oak Ridge Operations Weldon Spring Site Remedial Action Project Office Route 2, Highway 94 South St. Charles, Missouri 63303

December 21, 1987

Ms. B. Katherine Biggs
United States Environmental
Protection Agency
Region VII
726 Minnesota Avenue
Kansas City, Kansas 66101

Dear Ms. Biggs:

MDNR COMMENTS ON INTERIM RESPONSE ACTIONS (IRA'S)

Enclosed is our responsiveness summary for the comments contained in Dave Bedan's letter of November 12, 1987, regarding the following interim response actions:

- Dismantling of Building #401
- Dismantling of Building #409
- 3. Removal of PCB Transformers
- Debris Consolidation

We anticipate that this will adequately resolve the issues raised. We intend to proceed with action on these items in accordance with the enclosure.

If you have any questions, please give me a call.

Sincerely,

Rod Nelson

Project Manager Weldon Spring Site

Remedial Action Project

Enclosure:
As stated

cc: Dave Bedan, MDNR, w/enclosure

Jack Hammond, MK-F, w/o enclosure

RESPONSIVENESS SUMMARY

B. Katherine Biggs letter to Rodney R. Nelson, dated 11-13-87 re: MoDNR comments on:

Interim Response Actions

- Dismantling and Disposal of Building #401
- 2. Dismantling and Disposal of Building #409
- 3. Removal of PCB Transformers
- 4. Debris Consolidation

1. Dismantling and Disposal of Buildings #401 and #409

Comment: The DOE and its Contractors should develop and maintain close contact with the Missouri Air Pollution Control Program to assure compliance with Missouri Air Conservation Law and Missouri Solid Waste Management Law in carrying out these activities.

Response: The DOE and its subcontractor(s) will continue to keep the DNR Air Pollution Control Program office apprised of plans for work at the site involving removal, handling, storage, and/or disposal of asbestos materials.

Comment: Missouri Solid Waste Management Law requires demolition waste to be disposed of in a permitted sanitary or demolition landfill. Asbestos waste must be disposed of in a permitted sanitary landfill.

Response: The Specifications for this work will require that asbestos and other demolition debris be disposed of in accordance with the requirements of the Missouri Solid Waste Management Law.

Comment: DNR maintains that because of the special concerns relating to the volume of waste and to the possible contamination of the asbestos and the other demolition material with hazardous wastes or radioactive wastes, these materials should be handled as "special wastes".

Response: The DOE concurs that there are special circumstances that require handling of asbestos as "special waste". Specifications for the asbestos subcontracts contain this provision.

The pending subcontracts contain the "special waste" forms which will be included in the

subcontract work packages as matter of comity. Should subcontract efforts, cost or progress on these IRA's be impacted by this provision, the DOE will revisit this issue with the MDNR.

Comment: The DNR cannot approve the disposal of the asbestos and other demolition wastes until a procedure is in place to assure us that no radioactive or hazardous materials are being disposed of in Missouri solid waste landfills.

Response: Release standards are in place for controlling release of the rubble off site. Radiological survey and release plans will be developed for each work package involving removal and off-site disposal of materials to insure compliance with the standards.

Comment: DOE should provide justification for its policy to dispose of all non-radioactive building waste off site.

Response: The DOE policy is based on volume reduction and cost effectiveness. By disposal of nonradiological material in a sanitary or demolition landfill, there is a reduction in the amount of material (Volume Reduction) that will be encapsulated in any disposal cell. Secondly, costs for on-site disposal cells are high in comparison to disposal in sanitary or demolition landfills. Also, as an aside to the technical and cost effectiveness issues, the DOE currently has funding available. The site is still to be fully characterized and to delay demolition and disposal of clean materials would not allow these funds to be utilized and also would lead to overall slipping of the schedule.

2. Removal of PCB Transformers

Comment: MDNR recommends that if Alternative #5 is used, during the "flushing" process care should be taken to contain any spilled material. Also, "flushing" should be continued until PCB levels are less than 2 ppm, if transformer and switch carcasses are going to be disposed into a permitted sanitary landfill.

Response: The subcontract specifications for this interim response action include spill control provisions for draining and flushing operations. Spill control pans are specified to collect any spilled liquids. PCB transformers and other electrical

equipment which have been drained and flushed, as stated in the specifications, will be disposed of at an EPA approved PCB disposal facility, not a sanitary landfill.

Comment: If the PCB liquids are being transported to a disposal facility within Missouri, a licensed hazardous waste transporter must be used. If the PCBs are being transported to an out-of-state facility MDNR recommends that a licensed transporter be used although it is not a requirement.

Response: The specifications state that the transporter of the PCB liquids-and drained electrical equipment shall be licensed.

Comment: In the preamble to 40 CFR 761, unless otherwise tested, all dielectric transformers are assumed to contain 50-500 ppm PCB, therefore untested transformers (22, 32, and 45) should be "flushed" with other transformers.

Response: The three transformers which have not been sampled for PCBs will be treated as PCB-contaminated units unless future sampling is performed to otherwise classify them as non-PCB transformers or PCB transformers. Irrespective of the classification, these units will be drained and flushed on site, unless the disposal facility intends to incinerate them as intact units, as indicated in the subcontract specifications.

Comment: External pad, poles, and adjacent areas should be tested to determine if PCB contamination exists.

Response: Additional sampling for PCB contamination, in areas from which the PCB-containing transformers and other electrical components are to be removed, is planned as part of future chemical characterization activities at the site.

3. <u>Debris Consolidation</u>

Comment: The storage of solid waste on site may be subject to the requirements of the Missouri Solid Waste Management Law. Please contact the Missouri Waste Management Program for assistance in determining whether these requirements apply.

Response: An interim response action proposal is being prepared which will present plans for materials staging and interim storage of solid waste on site. We will contact the Missouri Waste Management Program Office for assistance in determining applicability of the Missouri Solid Waste Management Law to this work.

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IRA-800-805



Department of Energy

Oak Ridge Operations
Weldon Spring Site
Remedial Action Project Office
Route 2, Highway 94 South
St. Charles, Missouri 63303

January 10, 1990

Mr. Dan Wall
Remedial Project Manager
U. S. Environmental Protection
Agency
Region VII
726 Minnesota Avenue
Kansas City, Kansas 66101

Dear Mr. Wall:

CONTRACT NO. DE-AC05-860R21548 - MODIFICATIONS TO THE BUILDING 409 DISMANTLING AND BUILDING 401 DISMANTLING EE/CAS

Part of the work performed for IRA 10, Building 409 Dismantling, and IRA 11, Building 401 Dismantling, was conducted in a manner other than that originally planned (as documented in the respective EE/CAs). The original plan for building demolition material was to use a sanitary landfill in Missouri for off site disposal. However, it was later determined that the specified landfill was not in compliance with applicable regulations; therefore, the waste was disposed of in an Illinois sanitary landfill that was in regulatory compliance. It was originally planned to store the tar and gravel roofing material on site in a dry concrete-floored building. Actually, the gravel has been stored in a dry concrete building. The tar and felt material has been stored on pad 303 under a synthetic membrane cover, in a manner that achieves a similar level of protection.

Please accept this letter as a supplement to the Building 409 Dismantling and Building 401 Dismantling EE/CAs.

Sincerely,

R. R. Nelson

Project Manager Weldon Spring Site

Remedial Action Project

cc: Susan Meyers, PMC

INTERIM RESPONSE ACTION (IRA) ADMINISTRATIVE RECORD FILE ARFS FILE # IR-0900

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903 Engineering Evaluations/Cost Analysis						
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904	Engineering Eva	aluations/Cost Analysis Approval or Decision				
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4



Department of Energy

Oak Ridge Operations
Weldon Spring Site
Remedial Action Project Office
Route 2, Highway 94 South
St. Charles, Missouri 63303

October 16, 1987

Ms. Katherine Biggs
United States Environmental
Protection Agency
Region VII
726 Minnesota Avenue
Kansas City, Kansas 66101

Dear Ms. Biggs:

INTERIM RESPONSE ACTIONS (IRA'S)

Enclosed are six (6) copies of the documentation for the following four (4) Interim Response Actions:

- 1. Dismantling of Building 401
- 2. Dismantling of Building 409
- 3. Removal of PCB Transformers
- 4. Debris Consolidation

In addition, we are sending under separate cover, six (6) copies of the technical specifications and drawings from each of the four (4) proposed bid packages.

It is our intention to have copies of these documents in place in the repositories for public inspection, and to provide public notice of their availability on October 19, 1987. This will initiate the twenty one (21) day comment period.

If you have any questions, please give me a call.

Sincerely,

Rod Nelson

Project Manager

Weldon Spring Site Remedial Action Project

Enclosures: As stated

cc w/enclosures: D. Bedan, MDNR

DOCUMENT NUMBER: I 900 -901-1.01

The public comment period on this interim remedial action ends on November 9, 1987. Comments may be sent to any of the following:

- 1. Ms. Katherine Biggs
 U. S. Environmental Protection Agency
 Region VII
 726 Minnesota Avenue
 Kansas City, Kansas 66101
- 2. Mr. David Bedan Missouri Department of Natural Resources Post Office Box 176 Jefferson City, Missouri 65102
- 3. Mr. Rodney R. Nelson
 Weldon Spring Site Remedial Action Project
 Route 2, Highway 94 South
 St. Charles, Missouri 63303

DISMANTLING OF BUILDING 401

Site Background

The Weldon Spring site is located in St. Charles County, Missouri, about 48 km (30 mi) west of St. Louis. From 1941 to 1944, the U.S. Department of the Army operated the Weldon Spring Ordnance Works at the site for production of trinitrotoluene and dinitrotoluene. In the mid 1950s, a portion of the property was transferred to the U.S. Atomic Energy Commission (AEC), a predecessor of the U.S. Department of Energy (DOE).

From 1957 to 1966, the AEC operated a uranium processing facility at the Weldon Spring site. Impure uranium ore concentrates and some scrap uranium metal were processed at the chemical plant, and thorium-containing materials were also processed on an intermittent basis. Following closure by the AEC, the Army reacquired the chemical plant in 1967 and began converting the facilities to produce herbicides. The buildings were partially decontaminated and some equipment was dismantled. In 1969, prior to becoming operational, the herbicide project was canceled. Since that time, the plant has remained essentially unused and in caretaker status. The Army returned a portion of the Ordnance Works property to the AEC in 1971 but retained control of the chemical plant buildings. In 1984, the Army repaired several of these buildings; decontaminated some of the floors, walls, and ceilings; and removed some contaminated equipment to areas outside of the buildings. In 1985, custody of the chemical plant property was transferred to DOE.

Building 401, the steam plant, is located in the northwest section of the Weldon Spring site (Fig. 1). The coal-fired steam plant previously provided the energy necessary to support uranium—and thorium—processing activities at the site.

Site Characterization

Building 401 is a three-story rectangular structure with approximate dimensions of 33 m \times 30 m \times 12 m (107 ft \times 98 ft \times 38 ft) and nearly 1,600 m² (17,500 ft²) of floor space. Equipment and piping within the plant are generally in place and intact.

The first floor of the building contains numerous pieces of process equipment. The second floor contains small offices and an instrument control room and provides access to three external coal-fired boilers. Structural steel frames support the boilers and the induced-draft fans that previously discharged exhaust gases to the atmosphere via two external steel stacks. The building's third floor houses equipment related to its previous use as a service area for the overhead crane and storage tanks.

The roof of Building 401 consists of three tiers constructed of metal decking, insulation, and built-up tar and gravel. The floors are concrete, and the walls are composed of corrugated cement-asbestos siding attached to structural steel and concrete. An associated coal conveyor system also contains asbestos siding, and a major portion of the plant's boilers and steam and process equipment lines are insulated with

DOCUMENT NUMBER: 7-900-901-1.07

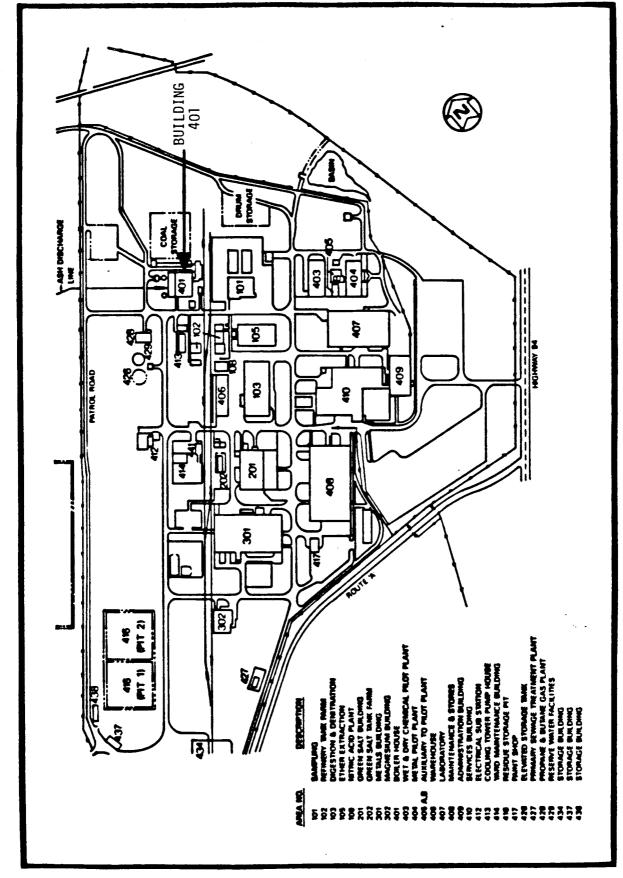


FIGURE 1 Location of Major Structures at the Weldon Spring Site (Source: Bechtel National 1986)

asbestos-containing materials. Although the siding is currently intact and most of the insulating material is in good condition, portions of the insulation have deteriorated and fallen to the floor in some areas.

Steam was produced in Building 401 by the combustion of coal. Coal combustion involves no radioactive material other than the very low concentrations of radioactive materials that occur naturally in coal. Therefore, the potential for radiological contamination of the building's interior would have resulted only from the movement of personnel and equipment into and out of the building, and the radiological survey of the plant was conducted with emphasis on traffic areas and portable equipment.

Results of the radiological survey, summarized in Table 1, indicate that contamination levels of the building's floors, walls, and equipment are below DOE's release limits for unrestricted use. These limits are 1,000 disintegrations per minute (dpm)/100 cm² removable alpha contamination and 5,000 dpm/100 cm² total alpha contamination (U.S. Department of Energy 1987). In fact, the highest reported value for total (fixed plus removable) contamination is below the lower removable contamination limit.

Results of a beta-gamma radiological survey of the roof, summarized in Table 2, indicate the presence of radioactive contaminants in the tar and gravel layer. The contamination resulted from airborne releases of radioactive dust (primarily containing uranium and its decay products) that occurred during the chemical plant processing period. Because the contamination has likely migrated into the tar (resulting in attenuation of alpha particles), beta-gamma activity was measured to provide an indication of the alpha activity associated with the roof layer. Survey results indicate that the contamination levels in the built-up tar/gravel roofing material will require that these materials be disposed of as radioactive waste. To confirm the need to treat the roofing material as radioactive waste, two samples of the tar/gravel layer were collected and analyzed by gamma spectrometry. The results, presented in Table 3, show that the primary contaminant is uranium.

Threat to Public Health and the Environment

Although most of Building 401 remains intact, some deterioration has occurred during the past two decades of disuse. At certain locations, asbestos-containing material has begun to deteriorate and fall to the floor. Interior air monitoring results indicate that concentrations of asbestos fibers do not currently pose a significant inhalation hazard. However, if deterioration of the building continues, the potential threat to public health posed by asbestos exposure will increase. In addition, the radiological contamination of the roof's tar/gravel layer poses an exposure hazard to workers in the area.

TABLE 1 Summary of Alpha Contamination in Building 401^a

Level	Surface	Measured Total (Fixed plus Removable) Alpha Activity		
		Range (dpm/100 cm ²)	Average (dpm/100 cm ²)	
1	Floor	0-224	81	
	Walls	0-143	75	
	Equipment	0-265	118	
2	Floor	20-673	200	
	Walls	41-673	206	
	Equipment	20-224	124	
3	Floor	61-388	226	
	Walls	41-224	135	
	Equipment	20-143	82	

aDoes not include the roof.

Source: Data from MK-Ferguson and Jacobs (1987b).

TABLE 2 Summary of Radioactivity Measurements on the Roof (Tar/Gravel) of Building 401

Tier	Beta-Gamma Measurements					
	Range (dpm/100 cm ²)	Average (dpm/100 cm ²)				
1	1,997-5,414	3,627				
2	3,341-5,261	4,077				
3	3,533-3,955	3,763				

Source: Data from MK-Ferguson and Jacobs (1987b).

TABLE 3 Gamma Spectrometry Measurements of the Tar/Gravel Roof Layer of Building 401

	Radionuclide	Concentration (pCi/g)	
Sample	Uranium-238	Radium-226	Radium-228
1	106.3	3.1	1.6
2	58.1	2.4	1.3

Source: Data from MK-Ferguson and Jacobs (1987a).

Response Objectives

The objectives of this response action are as follows:

- 1. Reduction of the potential health hazard due to asbestos exposure from asbestos-containing materials in Building 401;
- 2. Reduction of the potential health hazard due to radiation exposure associated with uranium contamination of the roof's tar/gravel layer; and
- 3. Removal of the potential safety hazard to on-site personnel associated with the deteriorating building.

Proposed Response Action Alternatives

Interim response actions are designed to ensure the health and safety of on-site personnel and to minimize or preclude off-site releases of contamination. These actions are limited to those that can be performed under the Comprehensive Environmental Response, Compensation, and Liability Act/Superfund Amendments and Reauthorization Act and remain within the constraints of the Council on Environmental Quality's regulations for the National Environmental Policy Act (i.e., actions will be limited to those that do not have an adverse environmental impact nor limit the choice of reasonable alternatives).

Alternative response actions identified for Building 401 are:

- 1. No action;
- 2. Removal of the tar/gravel roof layer for on-site storage, in-situ stabilization of asbestos-containing material, and repair of the building's structural deficiencies;

- Removal of the tar/gravel roof layer for on-site storage, removal
 of asbestos-containing material for off-site disposal, and repair of
 the building's structural deficiencies;
- 4. Dismantlement of Building 401, with on-site storage of all material that exceeds the radiological criteria for unrestricted release (i.e., the tar/gravel roof layer) and on-site disposal of all other material; or
- 5. Dismantlement of Building 401, with off-site disposal of all material except that which exceeds the radiological criteria for unrestricted release (i.e., the tar/gravel roof layer, which will be stored on-site), and reclamation of reusable materials that are not radiologically or chemically contaminated for salvage or on-site use.

Analysis of Alternatives

Alternative 1 affords no reduction in the potential health threat posed by radioactive and asbestos-containing material associated with Building 401. There would be no improvement in environmental conditions at the site if no action were taken. This alternative presents no technical barriers and costs nothing in the short term. However, the building is scheduled for eventual demolition. The costs associated with deferred dismantlement would be higher than those for dismantlement at the current time, due to periodic maintenance activities required until future dismantlement. Most importantly, Alternative 1 is effectively precluded by institutional factors related to the community's strong desire for timely response actions at the Weldon Spring site.

Alternatives 2 through 5 are all technically feasible. Each of these alternatives reduces the potential hazard associated with asbestos and radiation exposure. Implementation of Alternatives 2 and 3 would be more expensive in the long term, due to the need to repair structural deficiencies and perform future maintenance activities at Building 401. In addition, Alternatives 2 and 3 do not fully address the public sentiment for expedited response at the site. Even though Alternative 4 would be less expensive than Alternative 5, it is not consistent with DOE's intention to dispose of all non-radioactive waste off-site. Therefore, following the screening and analysis process for interim response action alternatives, Alternative 5 has been identified as the preferred alternative.

Description of Proposed Action

The proposed interim response action involves demolition of Building 401 with off-site disposal of all material meeting the criteria for unrestricted release — including asbestos-containing material. The response action will include the following operations.

1. Removal of the tar/gravel roofing material to a depth of approximately 5 cm (2 in.) for controlled on-site storage in a dry,

concrete-floored building currently located at the Weldon Spring site; and

2. Removal of all asbestos-containing material, equipment, and piping, and dismantlement of the remainder of Building 401 and ancillary structures, i.e., the external stacks and boilers, followed by scrap recovery and off-site disposal of the resultant waste material at a licensed sanitary landfill in Missouri.

The foundation and below-grade piping are not part of this action and will be addressed at a later date.

Under the proposed action, Building 401 will be dismantled in full compliance with all applicable regulations and procedures, with off-site disposal of all nonradioactive material (material that exceeds the radiological criteria for unrestricted release will be stored on-site). A representative fraction of material to be disposed of off-site will be radiologically surveyed prior to release. Asbestos removal and removal of the radioactively contaminated roof layer will be also performed in accordance with all applicable requirements. This compliance will ensure protection of the safety and health of on-site workers as well as limit off-site releases of contaminants.

Demolition of Building 401 will proceed in accordance with all safety requirements and practices. Demolition at this time will preclude the associated adverse impacts on health and the environment resulting from continued deterioration of the building. Removal of the demolition debris will be consistent with DOE's goal of removing all nonradioactive waste from the site.

The total waste volume is estimated to be 2,800 m^3 (3,600 yd^3), of which about 400 m^3 (500 yd^3) is asbestos-containing material and 40 m^3 (50 yd^3) is radioactive waste. The nonradioactive waste will be shipped to a licensed sanitary landfill in Missouri, requiring an estimated 250 truckloads.

References

- Bechtel National, Inc., 1986, Characterization Plan for the Weldon Spring Chemical Plant, DOE/OR/20722-95, prepared by Advanced Technology Division for U.S. Department of Energy, Oak Ridge Operations Office, Oak Ridge, Tenn. (Draft, Feb.).
- MK-Ferguson Company and Jacobs Engineering Group, 1987a, Weldon Spring Site Remedial Action Project, Hp Ge Radio-Isotope Analysis, ES-19-01-12, prepared by Environmental, Safety, and Health Department, Weldon Spring, Mo. (July).
- MK-Ferguson Company and Jacobs Engineering Group, 1987b, Radiological Survey Report for the Weldon Spring Chemical Plant Steam Production Facility, prepared for U.S. Department of Energy, Oak Ridge Operations Office, Oak Ridge, Tenn. (Sept.).

U.S. Department of Energy, 1987, U.S. Department of Energy Guidelines for Residual Radioactivity at Formerly Utilized Sites Remedial Action Program and Remote Surplus Facilities Management Program Sites (Revision 2, March).



UNITED STATES ENVIRONMENTAL PROTECTION AGENCY

REGION VII 726 MINNESOTA AVENUE KANSAS CITY, KANSAS 66101

NOV 18 1987

OFFICE OF THE REGIONAL ADMINISTRATOR

Mr. Rodney R. Nelson
U.S. Department of Energy
Weldon Spring Site Remedial
Action Project/Office
Route 2, Highway 94, South
St. Charles, Missouri 63303

Dear Mr. Nelson:

We have reviewed the Department of Energy's (DOE) proposals for the following four interim response actions:

- o Dismantling of Building 401,
- o Dismantling of Building 409,
- Removal of PCB Transformers, and
- · Debris Consolidation.

Our comments on these proposals were sent to you earlier. You were also provided comments by the Missouri Department of Natural Resources (MDNR). No comments from the public were directed to the Environmental Protection Agency (EPA) and according to our records, there has been no public comment directed to MDNR or DOE.

We are in agreement these actions should proceed to ensure worker safety and reduce the further release of contaminants from this site. The EPA hereby approves these actions under the condition that the comments earlier provided by EPA and MDNR are adequately addressed. The MDNR has notified me they also concur with these actions. Please provide copies of any summary reports for these actions to EPA and MDNR.

We also received copies of the interim response action for construction of the Ash Pond Dike. We will provide any comments on this proposed action within the agreed upon 21-day comment period. We are most pleased to see that activities are underway to stabilize the site and reduce contaminant release.

Sincerely yours,

Morris Kay

Regional Administrator

cc: Dr. Fred Brunner, MDNR

DOCUMENT NUMBER: 2-900-904-1.0/

11. =3-87 Jane 11. =3-87 DEC 0 9 1987

Ms. B. Katherine Biggs United States Environmental Protection Agency Region VII 726 Minnesota Avenue Kansas City, Kansas 66101

Dear Ms. Biggs:

USEPA COMMENTS ON INTERIM RESPONSE ACTIONS (IRA'S)

Enclosed is our response to the comments contained in your letter of November 9, 1987, regarding the following interim response actions:

- Dismantling of Building #401
- Dismantling of Building #409 2.
- 3. Removal of PCB Transformers
- Debris Consolidation

We anticipate that this will adequately resolve the issues raised. We intend to proceed with action on these items in accordance with the enclosure.

If you have any questions, please give me a call.

Sincerely,

ORIGINAL SIGNED BY: R. R. NELSON

Rod Nelson Project Manager Weldon Spring Site Remedial Action Project

Enclosure: As stated

Dave Bedan, MDNR

FTG 3743CL

PEER: JCoyne: x41: mw: 12/04/87: (c: EPA-IRA'. Ltt.) OCUMENT NUMBER: T-900-904-1,02 ..- 9.3 93

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RESPONSIVENESS SUMMARY

B. Katherine Biggs letter to Rodney R. Nelson, dated November 9, 1987 re:

Interim Response Actions

- Dismantling of Building #401 Dismantling of Building #409 2.
- 3. Removal of PCB Transformers
- Debris Consolidation

General

Comment: Generally, a more thorough analysis and screening of response alternatives would be appropriate.

Response: This comment was reviewed with the EPA (telecon from Rod Nelson to Dan Wall dated 11-17-87). The EPA agreed that while additional analysis and screening is not required for the four (4) IRA proposals addressed herein, future proposals such as the Ash Pond Isolation Dike will present a more thorough analysis of response alternatives.

Comment: The documents do not contain sufficient detail of the work to be done to stand alone without the support of the technical specifications and drawings.

Response: Technical specifications and drawings will continue to accompany the IRA proposal packages submitted for review.

Building Demolition

Comment: Specifics of handling, storage, and ultimate disposal of radioactively contaminated waste should be presented.

Response: Radioactively contaminated waste from demolition of Buildings #401 and #409 will be segregated and stored on-site in a dry, concrete floored building, Building #434 and/or Building #406. Ultimate disposal will be in accordance with the RI Plan/EIS. Specifics of handling this waste will be covered in the Contractor's operational work plan which will integrate the specification and drawings, the WSSRAP Construction Safety and Health Management Program, applicable WSSRAP Standard Operating Procedures and Plans along with the subcontractor dismantling plan. This work plan will be finalized prior to the Subcontractor(s) starting demolition work.

response, txtsheil

Comment: What guidelines will be used by the demolition subcontractor to determine the hazard potential of unknown materials encountered in the work?

Response: Subcontractor personnel who will work on the site will be required to undergo a minimum of 40 hours of initial instruction in hazardous waste operations prior to starting work on site in accordance with 29CFR1910.120. In addition Subcontractor personnel will receive indoctrination training in the known hazards in the work area prior to start of work in accordance with the WSSRAP Construction Safety and Health Management Program and Special Conditions requirement of the subcontract. Unknown (unidentified or unmarked) chemical substances encountered in the work shall be considered potential hazards and shall be reported to the Contractor in accordance with the requirements of the specifications.

The Contractor will also provide health physics, construction safety and industrial hygiene surveillance on a routine basis during all stages of the work. This will include inspections of all work areas to identify potential hazards. Where required, the Contractor will collect bulk samples to identify any unknown or suspected substances. The Contractor will also perform air monitoring, as necessary and prudent, to assess exposure levels of hazardous substances in the workplace.

Comment: The responsibility for determining whether a pollution condition has or will be created should be clearly specified.

Response: The WSSRAP Construction Safety and Health Management Program which is an integral part of site subcontracts assigns responsibility for the identification of potential pollution (environmental) conditions to the Project Management Contractor. The Subcontractor is contractually required to comply with the requirements of the Clean Air Act and the Clean Water Act.

Comment: The specification does not state the health and safety requirements for the subcontractor.

Response: Subcontractor health and safety requirements are defined in the Special Conditions to the subcontract. The Special Conditions bind the Subcontractor to compliance with the WSSRAP Construction Safety and Health Management Program and all applicable Federal, State, and local health and safety regulations and standards listed therein. The Special Conditions are a supplement to the General Conditions and General Provisions which also contain basic health and safety requirements.

PCB Transformer Removal

Comment: In this case, more detail in the site characterization section of the text would be appropriate. For instance, the PCB transformers are categorized as those containing PCBs at concentrations greater than 500 ppm. It may be somewhat misleading not to indicate in the text that the concentrations in these transformers are in excess of 350,000 ppm.

Response: The final subcontract work package includes a table on the subcontract drawings listing each electrical component in the scope of work. This table includes the PCB concentration and volume capacity, in gallons, of each electrical component.

Comment: Disposal facilities under consideration for receipt of these wastes must provide certification that they meet the Superfund offsite policy.

Response: The Work Plan specified in Section 1.2A of Specification Section 02090 includes provision for meeting all requirements of 40CFR761. The Subcontractor's Work Plan in section 1.4A will be required to contain certification that the facilities selected for disposal of the waste material (1) have received written approval from the U.S. Environmental Protection Agency as required under 40CFR Part 761.70 or 761.75, as applicable, and (2) are not under a state or federal compliance order under CERCLA or RCRA.

Debris Consolidation

Comment: It is stated in the description of the response action that one of the response objectives is to "Perform a detailed chemical and radiological characterization of the debris...". The description of the response action and specifications document contain no guidelines, references or information which would allow the Subcontractor to complete this objective.

Response: This objective is to be completed by the Contractor and does not require any special activity by the Subcontractor. Radiological guidelines to be used by the Contractor in performing this characterization are as defined in Draft DOE Order 5480.11 and applicable WSS operating procedures. All debris will be visually inspected for potential chemical contamination. Where chemical contamination is observed or suspected, sampling and analyses will be performed to identify the characteristics of the chemical.

Comment: More specifics regarding the handling, storage and ultimate disposal of radioactive contaminated wastes are needed.

Response: Specifics of handling the radiologically contaminated debris will be finalized upon submittal of the Subcontractor's work plan. That plan will be integrated with applicable WSSRAP Standard Operating Procedures and Plans, the WSSRAP Construction Safety and Health Management Program, and the specifications and drawings. The debris will be consolidated for temporary storage in a materials staging area. Details of the materials staging area will be presented in a separate IRA under preparation. Ultimate disposal of radioactive contaminated debris waste will be in accordance with the RI Plan/EIS.



Department of Energy

Oak Ridge Operations
Weldon Spring Site
Remedial Action Project Office
Route 2, Highway 94 South
St. Charles, Missouri 63303

December 21, 1987

Ms. B. Katherine Biggs
United States Environmental
Protection Agency
Region VII
726 Minnesota Avenue
Kansas City, Kansas 66101

Dear Ms. Biggs:

MDNR COMMENTS ON INTERIM RESPONSE ACTIONS (IRA'S)

Enclosed is our responsiveness summary for the comments contained in Dave Bedan's letter of November 12, 1987, regarding the following interim response actions:

- 1. Dismantling of Building #401
- Dismantling of Building #409
- 3. Removal of PCB Transformers
- 4. Debris Consolidation

We anticipate that this will adequately resolve the issues raised. We intend to proceed with action on these items in accordance with the enclosure.

If you have any questions, please give me a call.

Sincerely,

Rod Nelson

Project Manager Weldon Spring Site

Remedial Action Project

Enclosure: As stated

cc: Dave Bedan, MDNR, w/enclosure Jack Hammond, MK-F, w/o enclosure

RESPONSIVENESS SUMMARY

B. Katherine Biggs letter to Rodney R. Nelson, dated 11-13-87 re: MoDNR comments on:

Interim Response Actions

- 1. Dismantling and Disposal of Building #401
- 2. Dismantling and Disposal of Building #409
- 3. Removal of PCB Transformers
- 4. Debris Consolidation

1. Dismantling and Disposal of Buildings #401 and #409

Comment: The DOE and its Contractors should develop and maintain close contact with the Missouri Air Pollution Control Program to assure compliance with Missouri Air Conservation Law and Missouri Solid Waste Management Law in carrying out these activities.

Response: The DOE and its subcontractor(s) will continue to keep the DNR Air Pollution Control Program office apprised of plans for work at the site involving removal, handling, storage, and/or disposal of asbestos materials.

Comment: Missouri Solid Waste Management Law requires demolition waste to be disposed of in a permitted sanitary or demolition landfill. Asbestos waste must be disposed of in a permitted sanitary landfill.

Response: The Specifications for this work will require that asbestos and other demolition debris be disposed of in accordance with the requirements of the Missouri Solid Waste Management Law.

Comment: DNR maintains that because of the special concerns relating to the volume of waste and to the possible contamination of the asbestos and the other demolition material with hazardous wastes or radioactive wastes, these materials should be handled as "special wastes".

Response: The DOE concurs that there are special circumstances that require handling of asbestos as "special waste". Specifications for the asbestos subcontracts contain this provision.

The pending subcontracts contain the "special waste" forms which will be included in the

subcontract work packages as matter of comity. Should subcontract efforts, cost or progress on these IRA's be impacted by this provision, the DOE will revisit this issue with the MDNR.

Comment: The DNR cannot approve the disposal of the asbestos and other demolition wastes until a procedure is in place to assure us that no radioactive or hazardous materials are being disposed of in Missouri solid waste landfills.

Response: Release standards are in place for controlling release of the rubble off site. Radiological survey and release plans will be developed for each work package involving removal and off-site disposal of materials to insure compliance with the standards.

Comment: DOE should provide justification for its policy to dispose of all non-radioactive building waste off site.

Response: The DOE policy is based on volume reduction and cost effectiveness. By disposal of nonradiological material in a sanitary or demolition landfill, there is a reduction in the amount of material (Volume Reduction) that will be encapsulated in any disposal cell. Secondly, costs for on-site disposal cells are high in comparison to disposal in sanitary or demolition landfills. Also, as an aside to the technical and cost effectiveness issues, the DOE currently has funding available. The site is still to be fully characterized and to delay demolition and disposal of clean materials would not allow these funds to be utilized and also would lead to overall slipping of the schedule.

2. Removal of PCB Transformers

Comment: MDNR recommends that if Alternative #5 is used, during the "flushing" process care should be taken to contain any spilled material. Also, "flushing" should be continued until PCB levels are less than 2 ppm, if transformer and switch carcasses are going to be disposed into a permitted sanitary landfill.

Response: The subcontract specifications for this interim response action include spill control provisions for draining and flushing operations. Spill control pans are specified to collect any spilled liquids. PCB transformers and other electrical

equipment which have been drained and flushed, as stated in the specifications, will be disposed of at an EPA approved PCB disposal facility, not a sanitary landfill.

Comment: If the PCB liquids are being transported to a disposal facility within Missouri, a licensed hazardous waste transporter must be used. If the PCBs are being transported to an out-of-state facility MDNR recommends that a licensed transporter be used although it is not a requirement.

Response: The specifications state that the transporter of the PCB liquids-and drained electrical equipment shall be licensed.

Comment: In the preamble to 40 CFR 761, unless otherwise tested, all dielectric transformers are assumed to contain 50-500 ppm PCB, therefore untested transformers (22, 32, and 45) should be "flushed" with other transformers.

Response: The three transformers which have not been sampled for PCBs will be treated as PCB-contaminated units unless future sampling is performed to otherwise classify them as non-PCB transformers or PCB transformers. Irrespective of the classification, these units will be drained and flushed on site, unless the disposal facility intends to incinerate them as intact units, as indicated in the subcontract specifications.

Comment: External pad, poles, and adjacent areas should be tested to determine if PCB contamination exists.

Response: Additional sampling for PCB contamination, in areas from which the PCB-containing transformers and other electrical components are to be removed, is planned as part of future chemical characterization activities at the site.

3. Debris Consolidation

Comment: The storage of solid waste on site may be subject to the requirements of the Missouri Solid Waste Management Law. Please contact the Missouri Waste Management Program for assistance in determining whether these requirements apply.

Response: An interim response action proposal is being prepared which will present plans for materials staging and interim storage of solid waste on site. We will contact the Missouri Waste Management Program Office for assistance in determining applicability of the Missouri Solid Waste Management Law to this work.



Department of Energy

Oak Ridge Operations
Weldon Spring Site
Remedial Action Project Office
Route 2, Highway 94 South
St. Charles, Missouri 63303

January 10, 1990

Mr. Dan Wall
Remedial Project Manager
U. S. Environmental Protection
Agency
Region VII
726 Minnesota Avenue
Kansas City, Kansas 66101

Dear Mr. Wall:

CONTRACT NO. DE-AC05-860R21548 - MODIFICATIONS TO THE BUILDING 409 DISMANTLING AND BUILDING 401 DISMANTLING EE/CAS

Part of the work performed for IRA 10, Building 409 Dismantling, and IRA 11, Building 401 Dismantling, was conducted in a manner other than that originally planned (as documented in the respective EE/CAs). The original plan for building demolition material was to use a sanitary landfill in Missouri for off site disposal. However, it was later determined that the specified landfill was not in compliance with applicable regulations; therefore, the waste was disposed of in an Illinois sanitary landfill that was in regulatory compliance. It was originally planned to store the tar and gravel roofing material on site in a dry concrete-floored building. Actually, the gravel has been stored in a dry concrete building. The tar and felt material has been stored on pad 303 under a synthetic membrane cover, in a manner that achieves a similar level of protection.

Please accept this letter as a supplement to the Building 409 Dismantling and Building 401 Dismantling EE/CAs.

Sincerely,

R. R. Nelson

Project Manager Weldon Spring Site

Remedial Action Project

cc: Susan Meyers, PMC

DOCUMENT NUMBER: 7-900-905-1.0/